

UNCLASSIFIED

AD NUMBER	
AD350590	
CLASSIFICATION CHANGES	
TO:	UNCLASSIFIED
FROM:	SECRET//RESTRICTED DATA
LIMITATION CHANGES	
TO: Approved for public release; distribution is unlimited.	
FROM: Distribution authorized to U.S. Gov't. agencies and their contractors; Administrative/Operational Use; FEB 1963. Other requests shall be referred to Aeronautical Systems Division, ATTN: Propulsion Laboratory, Wright-Patterson AFB, OH 45433. Restricted Data.	
AUTHORITY	
OSD/WHS memo dtd 17 Mar 2016; OSD/WHS memo dtd 17 Mar 2016	

THIS PAGE IS UNCLASSIFIED



DEPARTMENT OF DEFENSE
WASHINGTON HEADQUARTERS SERVICES
1155 DEFENSE PENTAGON
WASHINGTON, DC 20301-1155



MAR 17 2016

MEMORANDUM FOR DEFENSE TECHNICAL INFORMATION CENTER
(ATTN: DTIC-OQ INFORMATION SECURITY)
8725 JOHN J. KINGMAN ROAD, SUITE 0944
FORT BELVOIR, VA 22060-6218

SUBJECT: OSD MDR Case 13-M-3701

We have reviewed the attached document in consultation with the Department of Energy, Department of the Navy, and Department of the Air Force and have declassified it in full. If you have any questions please contact Mr. John D. Smith by phone at 571-372-0482 or by email at john.d.smith887.civ@mail.mil, john.d.smith887.civ@mail.smil.mil, or john.smith@osdj.ic.gov.

George R. Sturgis
Deputy Chief, Records and Declassification
Division

Attachments:

1. MDR request w/ document list
2. Document 2



~~SECRET~~
~~RESTRICTED DATA~~

AD 3 5 0 5 9 0

DEFENSE DOCUMENTATION CENTER

FOR

SCIENTIFIC AND TECHNICAL INFORMATION

CAMERON STATION, ALEXANDRIA, VIRGINIA

Office of the Secretary of Defense ^{5 US\$552}
Chief, RDD, ESD, WHS
Date: 02 Oct 2015 Authority: EO 13526
Declassify: X Deny in Full: _____
Declassify in Part: _____
Reason: _____
MDR: 13 -M- 3701

Department of the Navy	
DON/AA DRMD	
Date: <u>10 Oct 2015</u>	Authority: EO 13526
Declassify: <u>X</u>	Deny in Full: _____
Declassify in Part: _____	
Reason: _____	
MDR: <u>2015</u>	-M- <u>0001</u>

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



DEPARTMENT OF ENERGY DECLASSIFICATION REVIEW	
1 st Review Date: <u>10/17/13</u>	DETERMINATION (CIRCLE NUMBER(S))
Authority: ☐ DC ☒ DD	1. CLASSIFICATION RETAINED
Name: <u>R.L. Shankle</u>	2. CLASSIFICATION CHANGED TO: NSI
2 nd Review Date: <u>10/30/2013</u>	3. CONTAINS NO DOE CLASSIFIED INFO
Authority: <u>DD</u>	4. COORDINATE WITH:
Name: <u>S. Fivozinsky</u>	5. CLASSIFICATION CANCELED
	6. CLASSIFIED INFO BRACKETED
	7. OTHER (SPECIFY):

~~RESTRICTED DATA~~
~~SECRET~~

RI

13-M-3701

NOTICE: When government or other drawings, specifications or other data are used for any purpose other than in connection with a definitely related government procurement operation, the U. S. Government thereby incurs no responsibility, nor any obligation whatsoever; and the fact that the Government may have formulated, furnished, or in any way supplied the said drawings, specifications, or other data is not to be regarded by implication or otherwise as in any manner licensing the holder or any other person or corporation, or conveying any rights or permission to manufacture, use or sell any patented invention that may in any way be related thereto.

NOTICE:

THIS DOCUMENT CONTAINS INFORMATION
AFFECTING THE NATIONAL DEFENSE OF
THE UNITED STATES WITHIN THE MEAN-
ING OF THE ESPIONAGE LAWS, TITLE 18,
U.S.C., SECTIONS 793 and 794. THE
TRANSMISSION OR THE REVELATION OF
ITS CONTENTS IN ANY MANNER TO AN
UNAUTHORIZED PERSON IS PROHIBITED
BY LAW.

Page determined to be Unclassified
Reviewed Chief, RDD, WNS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

CATALOGED BY DDC 350590

AS AD 140.

2

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277
Volume IV

350590

(UNCLASSIFIED TITLE)
NUCLEAR RAMJET PROPULSION SYSTEM
APPLIED RESEARCH AND ADVANCED TECHNOLOGY
(PROJECT PLUTO)

VOLUME IV
PROPULSION SYSTEM DESIGN AND STRUCTURAL ANALYSIS

TECHNICAL DOCUMENTARY REPORT ASD-TDR-63-277, VOLUME V

15 February 1963

Directorate of Aeromechanics
Propulsion Laboratory
Aeronautical Systems Division
Air Force Systems Command
Wright-Patterson Air Force Base, Ohio

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

Project No. 655A, Tasks Nos. 1 and 5

(Prepared under Contract AF 33(657)-8123
by The Marquardt Corporation, Van Nuys, California
Author: R. D. Grossman)

~~EXCLUDED FROM AUTOMATIC
REGRADING; DOD DIR 5200.10
DOES NOT APPLY~~

JUN 6 1964

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

63 ASRM-36

1005 38

MAC 1473

NOTICES

~~RESTRICTED DATA~~

~~This document contains restricted data as defined in the Atomic Energy Act of 1954. Its transmittal or the disclosure of its contents in any manner to an unauthorized person is prohibited by law.~~

When US Government drawings, specifications, or other data are used for any purpose other than a definitely related government procurement operation, the government thereby incurs no responsibility nor any obligation whatsoever; and the fact that the government may have formulated, furnished, or in any way supplied the said drawings, specifications, or other data is not to be regarded by implication or otherwise, as in any manner licensing the holder or any other person or corporation, or conveying any rights or permission to manufacture, use, or sell any patented invention that may in any way be related thereto.

Copies of this report should not be returned to the Aeronautical Systems Division unless return is required by security considerations, contractual obligation, or notice on a specific document.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

UNCLASSIFIED

2

REPORT 6003

ASD-TDR-63-277, Vol. IV

FOREWORD

This report was prepared by The Marquardt Corporation, Van Nuys, California, on Air Force Contract AF 33(657)-8123, under Task Order 1 and No. 5 of Project No. 655A, "Nuclear Ramjet Propulsion Systems Research and Technology". The work was administered under the direction of the Propulsion Laboratory (Directorate of Aeromechanics), Aeronautical Systems Division. R. F. Latham was Project Engineer for the Laboratory.

The studies presented here were performed during the contract period 1 January-31 December 1962. The Marquardt Corporation activities were under the direction of A. O. Mooneyham, Senior Project Engineer. Chief contributors were J. G. Bendot, Aerothermodynamics; R. D. Grossman, Design and Development; R. K. Nuno, Controls.

This report is the final technical summary report and concludes the work on Contract AF 33(657)-8123. The contractor's report number is Marquardt Report 6003. The volumes of this report are as follows:

- Volume I: Summary
- Volume II: Propulsion System Performance and Aerothermodynamics
- Volume III: Propulsion System Controls
- Volume IV: Propulsion System Design and Structural Analysis
- Volume V: Propulsion System Test Planning and Ground Test Facility Studies
- Volume VI: Structural Materials Investigations

This report is ~~classified Secret-Restricted Data~~ in conformance with requirements contained in AF-provided security guidance (Form 151) and OC DOC 75.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

S	marquardt	C
E		N
C		T
R		R
E		O
T		L

UNCLASSIFIED

- 11 -
MAC 4672

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV . ~~ATOMIC ENERGY ACT OF 1954~~

PORT 6003

ABSTRACT

(This abstract is ~~classified SECRET~~)

This volume contains the results of design, structures and materials studies and structures component testing of a nuclear propulsion system in support of the Pluto reactor program. These studies include design concepts, structural analysis of steady state and dynamic loads, material evaluation, and recommended dynamic and structural test programs. The methods of analysis used have been outlined in each case for reference.

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 653

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

SUMMARY

The mechanical and structural design effort during this contract period (1 January 1962 through 31 December 1962) has been directed towards the design of a flight type propulsion system, designated the MA50-XB, which incorporates a reactor reflecting Tory IIC reactor technology. Effort has also been expended in design and fabrication of development test hardware for component testing.

Design layouts have been completed for the MA50-XB Propulsion System in addition to the major components that make up the entire system; namely, the inlet and diffuser duct, ejector exhaust nozzle, reactor control rods support system, reactor lateral and axial support structure, inlet spike translation and bypass door mechanism and exhaust nozzle attachment.

Design and fabrication have been completed for the direct-connect and free jet aerodynamic coupling test hardware, engine airframe lateral attachment test hardware, and reactor lateral support spring test hardware.

Test outlines and test results for the aerodynamic coupling tests are reported in Volume II of this report.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

NET 6003

CONTENTS

Section	Page
1.0 INTRODUCTION.	1
2.0 PROPULSION SYSTEM DESIGN CRITERIA	3
2.1 General Discussion	3
2.2 Mission Parameters	3
2.2.1 Contractual Requirements.	3
2.2.2 Flight Capability	3
2.3 Typical Mission Profiles	4
2.4 Flight Envelopes	4
2.5 Nomenclature	5
2.5.1 Operational Phases.	5
2.5.2 Reference Axes.	5
2.6 Basic Operational Factors.	6
2.6.1 Inertial Load Factors	6
2.6.2 Vibration Environment	6
2.7 Analytical Factors	6
2.7.1 Design Factors and Factors of Safety.	6
2.8 Margins of Safety.	7
2.9 Limits of Structural Deformation	7
2.10 Materials and Material Properties.	8
2.10.1 Choice of Material.	8
2.10.2 Over Aging and Cyclic Loading	8
2.11 Design Parameters.	8
3.0 INLET	9
3.1 Design	9
3.1.1 Discussion.	9
3.1.2 Boundary Layer Air Bleed.	9
3.1.3 Bypass Doors.	9
3.1.4 Spike Transition.	9
3.1.5 Weights and Centers of Gravity.	10
3.2 Structural Analysis.	10
3.2.1 Discussion.	10
3.2.2 Cowling.	11
3.2.3 Diffuser Skin	15

UNCLASSIFIED

MAC 6573

UNCLASSIFIED

REPORT 6003

ASD-TDR-63-277, Vol. IV

CONTENTS (Continued)

<u>Section</u>		<u>Page</u>
3.2.4	Load Paths of Unsymmetrical Airloads.	15
3.2.5	Strut	19
3.2.6	Innerbody Ring at Section (F-F)	22
3.2.7	Innerbody Skin.	24
3.2.8	Thrust Fitting.	27
4.0	SUBSONIC DIFFUSER AND REACTOR CONTROL SUPPORT STRUCTURE	29
4.1	Design.	29
4.1.1	Discussion.	29
4.1.2	Weights and Centers of Gravity.	30
4.2	Preliminary Analysis of Control Rod Support Structure	30
4.2.1	Loads	30
4.2.2	Deflection Check of Strut	32
5.0	REACTOR SUPPORT SYSTEM.	35
5.1	Side Support System Design.	35
5.1.1	Side Support System Function.	35
5.1.2	Side Support Annulus Thickness to Vehicle Drag Comparison	36
5.1.3	Reduction in Annulus Width.	36
5.1.4	Design Side Support System.	37
5.1.5	Operating and Installation Requirements	37
5.2	Side Support System Analysis.	38
5.2.1	Nomenclature.	38
5.2.2	Dynamics Model and Optimum Spring Rate.	40
5.2.3	Structural.	43
5.3	Front Support Structural Criteria	51
5.3.1	Discussion of Axial Support System.	51
5.3.2	Dynamic Analysis.	54
5.3.3	Structural Summary.	60
5.3.4	Conclusions and Recommendations	61
5.4	Side Support System Test.	61
5.4.1	Discussion.	61
5.4.2	High Temperature Springs.	62
5.4.3	Engine-Airframe Lateral Attachment Tests Phases I and II	67

34C 4672

UNCLASSIFIED

CONTENTS (Continued)

<u>Section</u>	<u>Page</u>
6.0 EXIT NOZZLE	73
6.1 Design.	73
6.1.1 Discussion.	73
6.1.2 Weights and Centers of Gravity.	73
6.1.3 Design Data	73
6.2 Structural Analysis	74
6.2.1 Nozzle Forward Cylinder and Convergent Cone (Primary Shell)	74
6.2.2 Cone-Cylinder and Cone-Cone Intersection.	78
6.2.3 Divergent Exit Cone	82
6.2.4 Ejector Shroud (Nozzle Inner Liner)	96
6.2.5 Analysis of Exit Nozzle Attach Joint.	100
7.0 MODELS.	107
7.1 Direct-Connect Aerodynamic Coupling	107
7.1.1 Design.	107
7.2 One-Third Scale Free Jet Model.	107
7.2.1 Design.	107
8.0 REFERENCES.	109
APPENDIX A--Classified Materials Studies.	300
DISTRIBUTION.	311

Page determined to be Unclassified
Reviewed Chet RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

- vii -

MAC 4673

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

ILLUSTRATIONS

<u>Figure</u>		<u>Page</u>
1.	Master Layout - Pluto Propulsion System - MA50-XCB.	141
2.	Installation - Pluto Propulsion System - MA50-XCB.	143
3.	Typical Mission Profile for 11,000 Nautical Miles (ICAO Standard Day)	145
4.	Typical Mission Profile for 11,000 Nautical Miles (ANA Hot Day)	146
5.	Typical Mission Profile for 11,000 Nautical Miles (ANA Cold Day)	147
6.	Estimated Operating Envelope for Marquardt Model MA50- (ICAO Standard Day)	B Ramjet 148
7.	Estimated Operating Envelope for Marquardt Model MA50- (ANA Hot Day)	B Ramjet 149
8.	Estimated Operating Envelope for Marquardt Model MA50- (ANA Cold Day)	B Ramjet 150
9.	Typical Pluto Boost Trajectory.	151
10.	MA50-XCB Propulsion System Net Flow Areas	153
11.	Internal Engine Parameters for Design Point Operation	155
12.	MA50-XCB Propulsion System Auxiliary Air Requirements	157
13.	Design Layout - Inlet and Duct Lines.	159
14.	Design Layout - Inlet and Duct Lines.	161
15.	Design Layout - Axisymmetric Inlet.	163
16.	Design Layout - Axisymmetric Inlet.	165
17.	Design Layout - Axisymmetric Inlet.	167
18.	Sketch of Inlet Assembly for Inlet Structural Analysis	169
19.	Sketch of Spike and Inner-Body for Inlet Structural Analysis	170
20.	MA50-XCA Inlet-Diffuser Static Pressure	171
21.	Varying Pressures on Inside and Outside Lip and Cowl S. Cases	172

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

OFF 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
22.	Throc Loadings for Analysis Structure	173
23.	Sketch of Cowl Lip for Inlet Structural Analysis.	174
24.	Spike Geometry.	175
25.	Cowl and Innerbody Unsymmetrical Loadings	176
26.	Cowl and Innerbody Load Paths (Unsymmetrical Loadings).	177
27.	Unsymmetrical Air Load Distribution	178
28.	Shears, Moments, and Torques from Y Direction	179
29.	Shears and Moments from Z Direction Components.	180
30.	Geometry and Pressures for Inlet Structural Analysis.	181
31.	Derived Load Distribution on Fittings	182
32.	Longitudinal Section Through Typical Strut.	183
33.	Load on Struts from Supporting Rings.	184
34.	Loads on Strut Spars from Unsymmetrical Loads	185
35.	Vertical and Horizontal Loads on Strut Spars.	186
36.	Load Paths from Spike and Drive Mechanism to Fixed Innerho	187
37.	Innerbody Ring Loads.	188
38.	Symmetrical and Unsymmetrical Loadings.	189
39.	Ring Section Properties	190
40.	Ring Section Properties	191
41.	Thrust Fitting Loads.	192
42.	Struts.	193
43.	Strut Cross Section	194
44.	Design Progress on Reactor Side Support Structure	195
45.	Design Layout - Reactor Installation.	197

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 673

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
46.	Assumed Side Support Temperatures (ANA Hot Day)	199
47.	Assumed Side Reflector Temperature Distribution	200
48.	Schematic of Radial and Tangential Support Structure	201
49.	Dynamic System Model.	202
50.	Pressure Distribution	203
51.	Required Energy Storage for Lateral Support System.	204
52.	Typical Section of Web-Shell Combination.	205
53.	Estimated Full Scale Reactor Drag Load Variation During Unstart Transient Flight Operation.	206
54.	Estimated Pressure Transient on Full Scale Reactor During Unstart	207
55.	Estimated Differential Pressure Variation on Full Scale Reactor During Unstart.	208
56.	Reactor Drag, 1,000-Ft Altitude	209
57.	Estimated Full Scale Reactor Drag Load Variation During Unstart Transient Ground Test Only.	210
58.	Estimated Reactor Displacement During Unstart Transient Ground Test Only--Mach 3, 1000-Ft Altitude, ANA Hot Day	211
59.	Estimated Reactor Velocity During Unstart Transient Ground Test Only--Mach 3, 1000-Ft Altitude, ANA Hot Day.	212
60.	Estimated Reactor Displacement During Unstart Transient Flight Operation with 3 g Vehicle Deceleration - Mach 3, 1000-Ft Altitude, ANA Hot Day	213
61.	Estimated Reactor Velocity During Unstart Transient Flight Operation with 3g Vehicle Deceleration - Mach 3, 1000-Ft Altitude, ANA Hot Day	214
62.	Belleville Springs--R-235 Material.	215
63.	Belleville Springs--R-235 Material (1/16-in. Full Radial & Both I.D. Edges)	216

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ORI 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
64.	Test Setup for Single Belleville Spring--R-235 Material	217
65.	Strain Gages Installed on Inner and Outer Radius of Modified Belleville Spring (1/16-in. Full Radius Both I.D. Edges)-- R-235 Material.	218
66.	Setup for Test of Single Belleville Spring with Strain Gage Installed	219
67.	Test Setup for Strain Gaged Belleville Springs (R-235 Material).	220
68.	Test Setup for 10-Spring Series Stack of Belleville Springs R-235 Material.	221
69.	Partial Setup for 1400°F Test of 10-Spring Series Stack of Belleville Springs--R-235 Material.	222
70.	Permanent Set vs. Deflection During Presetting of Single Belleville Springs (R-235 Alloy).	223
71.	Load vs. Deflection: No. 11, No. 109, and No. 111 Single Belleville Springs (R-235 Alloy).	224
72.	Permanent Set vs. Deflection During Presetting of Single Belleville Springs.	225
73.	Load vs. Deflection for Single Belleville Springs (R-235 Alloy)	226
74.	Load vs. Deflection for Single Belleville Springs (R-235 Alloy)--Ambient Temperature	227
75.	Load vs. Deflection: No. 95 Belleville Spring--1/16-in. Full Radius Both I.D. Edges (R-235 Alloy).	228
76.	Stress vs. Spring Radial Width -Belleville Springs No. 29 and No. 25 (R-235 Material, Ambient Temperature; Flat Plate Loading).	229
77.	Load vs. Deflection: No. 1 and No. 4 10-Spring Series Stack of Belleville Springs (R-235 Alloy)	230
78.	Load Loss vs. Time at Constant Spring Deflection: No. 4 10- Spring Series Stack of Belleville Springs (R-235 Alloy)	231
79.	Tapered Curved Plate Springs--R-235 Alloy.	232

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

MT 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
99.	Core Dynamic Response (Run No. 10)	252
100.	Lateral Attach Assembly-Engine-Airframe	253
101.	Test Item for Engine-Airframe Lateral Attachment Test (Phase II)	255
102.	Vibration Test Setup for Engine-Airframe Lateral Attachment Test (Phase II)	256
103.	Instrumentation Requirements for Engine-Airframe Lateral Attachment Test (Phase II)	257
104.	Spring Displacement vs. Frequency (15 psi Core Preload)	258
105.	Core Deflection Pattern, Run 4.	259
106.	Spring Displacement vs. Frequency (6g Force Input)	260
107.	Design Layout - Propulsion System Exit Nozzle	261
108.	Wall Pressure Distribution for MA50-XCB Ejector Nozzle (Altitude = 10,000 Feet)	263
109.	Wall Pressure Distribution for MA50-XCB Ejector Nozzle (Altitude = 35,000 Feet)	264
110.	Local Wall Pressure Distribution in Exhaust Region of Airframe Cooling Flow: MA50-XCB Ejector Nozzle	265
111.	MA50-XCB Ejector Nozzle Axial Loads (Drag)	266
112.	Estimated Temperature Distribution for MA50-XCB Ejector Nozzle (3.22 M)	267
113.	Estimated Temperature Distribution for MA50-XCB Ejector Nozzle (3.90 M)	268
114.	MA50-XCB Ejector Nozzle Contours (Based on 0.125 in. Nozzle Liner Thickness)	269
115.	MA50-XCB Ejector Nozzle Liner Exit Details	270
116.	Nozzle Load Parameters	271
117.	Doubler Ring	272

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 6073

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
118.	Liner Load Parameters	273
119.	Assumed Coupling Geometry	274
120.	Retaining Ring Geometry	275
121.	Airframe Fitting Shell.	276
122.	Lock Ring	277
123.	Exit Nozzle Attach Ring	278
124.	Testing of Aerodynamic Coupling with 1/3 Scale Engine (Phase I - Direct Connect Test)	279
125.	Aft Transition Section for Aerodynamic Coupling Test.	280
126.	Buildup of Simulated Reactor Segment for Aerodynamic Coupling Test (Closeup).	281
127.	Front Support Section of Aerodynamic Coupling (Looking Forward).	282
128.	Aft End of Simulated Reactor Segment (Viewed Up Exit Nozzle).	283
129.	Engine Assembly Aerodynamic Coupling Free Jet Test.	285
130.	Cowl Assembly for Free Jet Aerodynamic Coupling Test.	287
131.	Spike Adapter for Free Jet Aerodynamic Coupling Test.	288
132.	Display of Components for Free Jet Aerodynamic Coupling Test.	289
133.	Closeup of Components for Free Jet Aerodynamic Coupling Test.	290
134.	Spike Actuator and Housing Details for Free Jet Aerodynamic Coupling Test	291
135.	Spike Actuator and Housing Assembly for Free Jet Aerodynamic Coupling Test	292
136.	Simulated Reactor and Housing for Free Jet Aerodynamic Coupling Test	293
137.	Test Item Assembly (Axial Load Spring Retention) for Free Jet Aerodynamic Coupling Test	294

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 1677

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF . 6003

ILLUSTRATIONS (Continued)

<u>Figure</u>		<u>Page</u>
138.	Centerbody and Inlet Duct for Free Jet Aerodynamic Coupling Test	295
139.	Centerbody and Inlet Duct Assembly for Free Jet Aerodynamic Coupling Test	296
140.	Exposed Inlet Assembly for Free Jet Aerodynamic Coupling Test	297
141.	Inlet Assembly for Free Jet Aerodynamic Coupling Test (Three Quarter View)	298
142.	Setup of Pluto Propulsion System Model for Free Jet Aerodynamic Coupling Test	299
A-1.	Stress-Time Profile for Rene' 41.	305
A-2.	Creep of Rene' 41 Sheet (1400°F; 52 Ksi).	306
A-3.	Ultimate Tensile Strength and Yield Strength of Rene' 41 Sheet at Selected Strain Rates.	307
A-4.	Young's Modulus and Percent Elongation of Rene' 41 Sheet at Selected Strain Rates	308

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLES

<u>Table</u>		<u>Page</u>
I	Table of Weights and Centers of Gravity for MA50-XCB Propulsion System	111
II	Margins of Safety	112
III	Inertial Load Factors	114
IV	Inertia Load Factors Flight Maneuver Conditions	115
V	Vertical Load Factors	116
VI	Ground Handling Load Factors.	117
VII	Analytical Design Factors and Factors of Safety	118
VIII	Weights and Centers of Gravity for Axisymmetric Inlet	119
IX	Section Properties and Locations for Calculation of Stress.	120
X	Moments from Air Load	121
XI	Torque Loads at Various Sections.	122
XII	Shears, Moments, and Torques from Z Direction Components.	123
XIII	Pressure Calculations	124
XIV	Joint Loads	125
XV	Attachment Reactions from Airloads for the Unsymmetrical Condition	126
XVI	Symmetrical Load Portion of Steady State Flight Condition Ring Moments.	128
XVII	Steady State Flight Condition Ring Moments.	129
XVIII	Ring Moments from Short Time Maneuver Loads	130
XIX	Weight and Centers of Gravity for Control Rod Support Structure and Actuator Mechanism.	131
XX	Weights and Centers of Gravity for Reactor and Support System	132
XXI	Thermal Expansion of Tangentially and Radially Oriented Springs	133
XXII	Summary of Criteria	134
XXIII	Table of Dynamic Test Runs, "Nominal Conditions".	135

UNCLASSIFIED

MAC 4673

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

POST 6003

TABLES (Continued)

<u>Table</u>		<u>Page</u>
XXIV	Accelerometer Response, Run No. 4	136
XXV	Accelerometer Response, Run No. 10.	137
XXVI	Weights and Centers of Gravity for Ejector Exhaust Nozzle . .	138
XXVII	Required Material Thickness	139
XXVIII	Shroud Pressure and Temperature Data.	140
A-I	Creep of Rene' 41 During Cyclic Testing	301
A-II	Tensile Properties of Rene' 41 Sheet After Cyclic Testing . .	302
A-III	Creep-Rupture of Rene' 41 Sheet Control Specimens	303
A-IV	Tensile Properties of Rene' 41 Sheet at Selected Strain Ra: s .	304

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

1.0 INTRODUCTION

The model MA50-XCB nuclear ramjet propulsion system, as shown in Figures 1 and 2 consists of a variable geometry supersonic inlet with a modified isentropic spike, a subsonic diffuser incorporating a variable area bypass, a nuclear reactor similar in construction to the Tory II reactor with integrated control system, and a convergent-divergent ejector type exhaust nozzle.

The inlet, which is an underslung, axisymmetric, internal-internal compression type, has a translating centerbody spike with a maximum travel of seven inches. The spike actuation mechanism is housed within the centerbody structure and is air-operated. Air is supplied to the actuator through a slot located in the centerbody structure. The bypass doors are located in the vicinity of the aft centerbody structure and are an integral part of the supersonic inlet structure.

The subsonic diffuser duct, from aft of the supersonic inlet to the front face of the reactor, is an integral part of the missile airframe structure.

The nuclear reactor is composed of a series of individual ceramic elements that make up the fueled core and front rear and radial reflectors. The reactor is maintained in the form of a right circular cylinder by a series of spring-loaded expansion shell pads. A series of axial tie tubes (which pass through the reactor) collect all aft directional loads through rear bearing plates and transfer them to a front support structure. All axial loads imposed upon the reactor are transferred to the airframe through a shear ring located in the vicinity of the front support structure.

The reactor control rod translating mechanisms are mounted forward of the front support structure and housed within the inlet duct. Control rod actuators are mounted in the annulus between the diffuser duct and the missile airframe.

The exhaust nozzle, is a convergent-divergent ejector type with fixed primary and secondary nozzle flow areas. This design employs a convergent-divergent outer shell with an inner shell in the convergent section only. The annulus between the inner and outer shell is sized such that the engine cooling air (secondary flow) cools the convergent portion of the nozzle by forced convection. The divergent portion of the nozzle is then film-cooled by the air issuing from the annular passage just aft of the throat.

The weights and centers of gravity for the MA50-XCB and its components are shown in Table I. Margins of safety are presented in Table II.

Manuscript released by The Marquardt Corporation on 15 February 1963 for publication as an ASD Technical Documentary Report.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

2.0 PROPULSION SYSTEM DESIGN CRITERIA

2.1 General Discussion

Particular emphasis for efficient operation of any propulsion system designed for high speed flight must be placed on lightweight and reliable structure, which will provide the necessary performance. Structural design will determine the shape and size of individual structural members to obtain the minimum weight of these structures. Design criteria are concerned with the data associated with the operational life, the associated loads, temperatures, and times at load and temperature. Also included will be nomenclature and design factors for use in analysis together with the physical properties of the materials used in fabrication. Such data are presented in the pages following.

2.2 Mission Parameters

2.2.1 Contractual Requirements

The following typical mission parameters are specified in Reference 1, Paragraph 1.1, to be used as a basis for propulsion system design:

Minimum range 11,000 nautical miles

Maximum payload 10,000 pounds

2.2.2 Flight Capability

High altitude cruise at 35,000 feet at Mach 3.5 to 4.0:

Minimum time 4.0 hours

Maximum time 7.0 hours

Penetration at 500 to 1000 feet at Mach 2.8 to 3.0:

Minimum time 2.0 hours

Maximum time 3.0 hours

Component design lifetime to be adequate for high reliability in performing above maximum requirements.

The Pluto propulsion system is designated the Arquardt Model MA50-XCA Nuclear Ramjet Engine. Operating envelopes for this engine are presented in Reference 2. The envelope limits are associated with control and actuator requirements and structural limitations of materials. Due to the establishment of temperature and pressure limitations (inlet air total temperature of 1070°F and diffuser exit pressure of 420 psia), the system would operate at Mach 3.2, Sea Level, on an ICAO Standard Day. This point places the system in the most severe steady state operating condition (exceeding the requirements stated above) and should be taken as the structural design point.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

PRECEDING PAGE BLANK

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

6003

2.3 Typical Mission Profiles

Vehicle total mission flight profiles used by Marquardt for ICAO Standard and ANA 421 Hot and Cold Days, and presented in Figures 3, 4 and 5, are reproduced for reference from Ling-Temco-Vought data presented in Reference 3, and again in Reference 4. Critical Mach number vs. altitude combinations from these curves are as follows:

Day Condition	Operational Phase	Altitude (feet)	Mach No
ANA 421	High Altitude Cruise	32,000	3.55
Hot Day	Low Altitude Cruise	1,000	2.92
ANA 421	High Altitude Cruise	35,000	4.0
Cold Day	Low Altitude Cruise	1,000	3.1
ICAO	High Altitude Cruise	35,000	3.8
Std Day	Low Altitude Cruise	1,000	3.1

A typical regime represents a flight of 11,000 nautical miles in approximately 310 minutes. These regimes consist of launch and rocket boost to high altitude, steady state cruise at high altitude for approximately 1-2/3 hours, letdown to low altitude in approximately 2.0 minutes, and steady state cruise to target for the balance of flight, approximately 1-1/3 hours. During cruise phases intermittent maneuver and gust loadings will occur.

Contractual requirements of Section 2.2.0 specify a more critical design life of 10 hours. The flight envelope (Figure 6) for ICAO Standard Day specifies a more severe low altitude cruise combination of Mach 3.2 at Sea Level.

2.4 Flight Envelopes

The preliminary Pluto propulsion system operating envelopes for the Model MA50-XCB Nuclear Ramjet for ICAO Standard and ANA 421 Hot and Cold days were presented in Reference 2 and are reproduced in Figures 6, 7, and 8. A typical boost trajectory is shown in Figure 9. Limits for these envelopes are as follows:

1. Mach 2.0 lower limit is established by operation requirements on pneumatic components.
2. Upper altitude limit is established as the line of constant diffuser exit pressure of 45 psia (assures required to 1 pneumatic pressure ratio) up to 50,000 feet.
3. Mach number and altitude requirement is set either to ram air total temperature of 1070°F or a diffuser duct total pressure of 420 psia.

~~SECRET RESTRICTED DATA~~

~~Atomic Energy Act of 1954~~

MAC 4673

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

2.5

Nomenclature

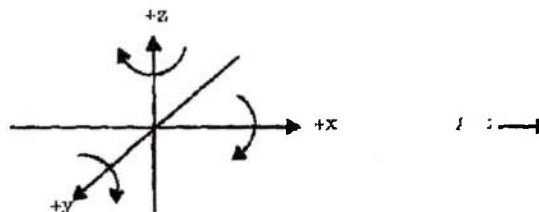
2.5.1 Operational Phases

As an aid to systematic analyses, proposed typical missile mission profiles are divided into a series of operational phases. These phases which are studied separately for individual maximum loadings and collectively for lifetime loading effects, are the following:

1. Ground handling
2. Boost
 - a. Burnout (launch to burnout)
 - b. Separation (booster separation to high altitude cruise)
3. High altitude cruise
4. Letdown
5. Low altitude (penetration) cruise
6. Weapons delivery (ejection of multiple stores during final phase of low altitude flight)

2.5.2 Reference Axes

The following reference axes notations were used in the analyses: (1) loads, linear accelerations, and dimensions positive when acting aft, up, and to the left (viewed from aft); (2) moments, angular accelerations, and angular velocities about reference axes follow the "left hand" rule:



~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 452

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

~~ATOMIC ENERGY ACT OF 1954~~

ONT 6003

2.6 Basic Operational Factors

2.6.1 Inertial Load Factors

2.6.1.1 Flight Maneuver Factors at Missile Center of Gravity

The inertial load factors presented in Table I represent the translational acceleration at the vehicle center of gravity and are reproduced from data presented by Ling-Temco-Vought in Reference 3. Rotational accelerations $\dot{W}_y$ and $\dot{W}_z$ are referenced to the vehicle center of gravity.

Dynamic load shears and moments for ejection of multiple warheads must be added to the static rigid body shears and moments derived from these load factors.

2.6.1.2 Flight Maneuver Translational Inertial Load Factors at Centers of Gravity of Individual Components

The component inertial loads shown in Table IV were calculated from data presented in Reference 3 (Figure 18).

2.6.1.3 Design Limit Load Factors for Reactor

The following inertial factors (Figure 19 of Reference 3) represent the combined inertial effects of both translational and rotational acceleration acting at the reactor center of gravity at vehicle Fuselage Station 842.48. The vertical load factors for the case of weapons ejection are presented in Table V for the reactor end stations.

2.6.1.4 Design Limit Ground Handling Inertial Factors

The following factors (Table VI) are in terms of translational acceleration at the missile center of gravity (refer to Reference 3, Figure 20).

2.6.2 Vibration Environment

- | | | |
|----------------------------------|--|---------------|
| 1. Launch-Boost | Average 3.0g RMS in all directions (180db) | 5 to 2000 cps |
| 2. Cruise (Boundary Layer Noise) | Average 2.25 g in all directions (164db) | 5 to 2000 cps |

2.7 Analytical Factors

2.7.1 Design Factors and Factors of Safety

Multiplying numerical factors used in the structural analysis design of structural components are classed as "Design Factor" and "Factors of Safety" (Table VII).

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 4873

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

2.7.1.1 Design Factors

Structural deformations alter propulsion stem performance and, if excessive, may result in functional failure. Design factors are utilized to provide greater assurance against deformations that are difficult to predict with confidence.

2.7.1.2 Factors of Safety

Factors of Safety are utilized to reduce the possibility of catastrophic structural failure, such as buckling or rupture

2.7.1.3 Limit Loads

A limit load is defined as the maximum value of the load to which a structural member will be subjected during a critical design condition.

2.7.1.4 Design Load

A design load is a limit load increased by a specified Design Factor or Factor of Safety.

2.7.1.5 Use of Factors

To avoid confusion, the limit values of loads and/or stresses shall be utilized in all calculations and design factors or factors of safety applied only in the calculation of margins of safety.

2.8 Margins of Safety

Calculated values of limit stresses increased by the appropriate multiplying design or safety factor are compared to the pertinent allowable mechanical strength properties and margins of safety derived as follows:

$$\text{Margin of Safety} = \left(\frac{\text{Allowable Stresses}}{\text{Limit Stresses} \times \text{factor}} \right) - .00$$

2.9 Limits of Structural Deformation

Material strain within the elastic action range rarely affects end product function, but additional progressive deformation associated with time and temperature will progressively alter propulsion system performance and may result in system malfunction.

For a given functional design the object of structural design, then, is to determine the amounts of deformation that can be tolerated without excessive functional loss and then to provide the minimum weight structure that will accomplish these requirements.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

CONF 6003

Deformation limits vary between individual structural components, depending upon their function in the system and the interaction between components.

Allowable limits of deformation are specified in the separate analyses of individual components presented in later sections of this report.

2.10 Materials and Material Properties

2.10.1 Choice of Material

A study of available materials suitable for high temperature operation indicates that nickel base alloys are the most suitable for the current load, time, and environment operational requirements of the turbo propulsion system.

Rene' 41, a precipitation-hardened type of nickel base alloy, the strength of which is developed by various solution and aging heat treatments, appears to be more attractive for use in the present thermal environment. However, if the thermal environment should become less severe, then 15-7PH may be considered.

2.10.2 Over Aging and Cyclic Loading

Long life at temperatures approaching the precipitation heat treatment range may result in excessive overaging and lower stress rupture life. In addition, the low cycle-high stress level fatigue life decreases rapidly at temperatures near the precipitation treatment range.

Since the operational temperatures of the inlet structures are in the 1000°F range, the chosen material should be investigated for cyclic loading.

2.11 Design Parameters

Engine design parameters are shown in Figures 10, 11, and 12 presenting net flow areas, Mach numbers, temperatures, pressures, and auxiliary air requirements.

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

3.0 INLET

3.1 Design

3.1.1 Discussion

Design layouts have been completed for the basic inlet. Basic lines are shown in Figures 13 and 14, and structural framing, the spike translation mechanism, and the bypass door actuation mechanism are shown in Figures 15, 16, and 17, respectively.

The inlet is a complete, self-contained assembly starting at Engine Station 49.348 (at tip of spike) and terminating at Engine Station 286.936. The inlet assembly is attached to the underside of the vehicle by bolts through oversized holes for vertical loads only and by a shear pin connection for forward and aft loads. Side loads are resisted by the shear pin connection and six shear pins equally spaced around the structural ring at Engine Station 286.936.

3.1.2 Boundary Layer Air Bleed

Provision for efficiently discharging inlet boundary layer bleed air is made by ducting the air through the two lower sections into the annulus between the inner and outer shells of the inlet diffuser such that the boundary layer bleed exits axially downstream through a stepped outlet.

3.1.3 Bypass Doors

Bypass doors are used to control the airflow to the reactor. Due to the control package, the doors are located in the supersonic section of the supersonic inlet diffuser where the nuclear heat generation rates are low and space restriction for the actuator mechanism is less severe. The doors are of the vertically hinged, counterbalanced shutter type, mounted on both sides of the inlet on the inlet horizontal centerline. The doors are designed to operate at a maximum pressure differential of 420 psi at design point, and are capable of opening and reclosing within three seconds when required. The doors are rotated by a means of a top crank operated from a single pneumatic actuator, thereby insuring synchronization.

3.1.4 Spike Translation

The inlet must be capable of start, restart, and shock positioning at Mach numbers below and above design point. The spike must move forward to start or restart the inlet and must retract for operation at higher Mach number. At reduced Mach numbers the intersection point of the two compression shocks moves forward, and it is necessary to extend the spike forward to keep these shocks positioned properly on the centerbody and boundary layer bleed. The translating spike is designed to retract and extend 6.00 inches forward from the design position by critical operation at Mach 2.8. Also, the spike is designed to retract and extend 7.00 inches in three seconds.

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ORT 6003

The spike translation mechanism is designed to operate against a maximum spike load of 58,000 pounds. A modified pneumatic gear motor with servo valve (input speed: 1200 rpm; output speed: 8.2 rpm) drives a cone drive worm gear (30:1 reduction), which drives a 19-tooth pinion which drives two racks (3 1/2 diametral pitch teeth with effective face width of four inches) that are attached to the inlet spike, thereby providing for spike translation.

The centerbody spike is mounted to a guide system, which consists of four one-inch rods (equally spaced circumferentially about and radially located 5.25 inches from the inlet horizontal centerline) and roller guide bushings affixed to the internal centerbody structure. Axial loads are resisted by the actuation mechanism, and the side loads are resisted by the guide system.

3.1.5 Weights and Centers of Gravity

Weights and centers of gravity for the inlet spike are listed in Table VIII.

3.2 Structural Analysis

3.2.1 Discussion

The following preliminary structural analysis of the inlet is concerned primarily with major structural items, (Figures 18 and 19). It has been found that in most cases the inertia forces counteract the primary air loads and are small in magnitude. It has therefore been considered sufficiently accurate to omit them from this analysis.

It has been assumed that the structure has been subjected to three possible conditions:

1. A steady state, long time (10 hours) flight regime near sea level with air entering the inlet at 1.0° pitch angle.
2. A short time flight regime (3 minutes) near sea level with air entering the inlet at 6.0° pitch angle and 3.6° yaw angle; this regime represents a "pull-up" condition.
3. A short time flight regime similar to (2) but with shock outside the cowl producing abnormal radial loads on the spike itself.

The following nomenclature is used in the structural analysis:

- F = Stress, psi
 Δp = Pressure differential, psi
I = Moment of inertia, in⁴

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

b = Width of section, in.

t = Thickness, in.

F_{cr} = Allowable crippling stress, psi

M.S. = Margin-of-safety

3.2.2 Cowl Lip

The pressures for a condition of Mach 3.0, 1000 feet from data given in Figure 20 are combined as follows to determine cowl stresses and deflections, as these were the only data available at the time of this analysis.

Pressures on inside and outside lip surface for an axial length of 4.8 inches are shown in Figure 21 (Sketch A).

Pressures on inside and outside cowl surface for an additional axial length of 8.4 inches are shown in Figure 21 (Sketch B).

These pressures are averaged as follows to find the average pressure on the whole 13.2 inches of axial length and a total loading per inch of circumference as shown in Figure 21 (Sketch C).

ΔP (psi)	Length (inches)	Loading Per Inch of Circumference (lb/inch)	
-37	(4.8)	- 177.5	Sketch
-96	(8.4)	- 806.0	Sketch
-74.5	(13.2)	- 983.5	

The foregoing loading of Figure 21 (Sketch C) is now broken into three portions to represent unsymmetrical loading conditions that could result from pitch or yaw angles of attack at the same altitude and speed.

It is assumed in this analysis that the outward loading on the windward side drops to one-half its original value, that the loading on the leeward side could increase to twice its original value and that the loadings on the neutral sides would remain unchanged. The three loadings which will be superimposed one on another for analysis of the structure then appear in Figure 22.

Stresses due to hoop tension and bending at the 0° and the 90° locations. Section properties and locations at which stresses are calculated are shown in Table IX and Figure 23. Moments combined at which stresses from Table X.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ORT 6003

3.2.2.1 Circumferential Location 0°

$$f("e", "g", \text{ or } "n") = F_{H.T.} + \frac{M\bar{a}}{I} = + \frac{984 \times 20}{3.94} + \frac{-17}{0.631} \frac{30(-0.52)}{0.631}$$

$$= 5000 + 11,000 = 16,000 \text{ psi}$$

$$f("s") = + \frac{984 \times 20}{3.94} + \frac{-13,250(0.50)}{0.631} = 5000 - 10,500 = -5500 \text{ psi}$$

$$f("a") = + \frac{984 \times 20}{3.94} + \frac{-13,250(0.147)}{0.631} = 5000 - 3080 = +1920 \text{ psi}$$

$$f("l") = + \frac{984 \times 20}{3.94} + \frac{-13,250(1.067)}{0.631} = 5000 - 22,400 = -17,400 \text{ psi}$$

3.2.2.2 Circumferential Location 90°

$$f("e", "g", \text{ or } "n") = F_{H.T.} + \frac{M\bar{a}}{I} = + \frac{984 \times 20}{3.94} + \frac{14,500(-0.522)}{0.631}$$

$$= +5000 - 12,000 = -7000 \text{ psi}$$

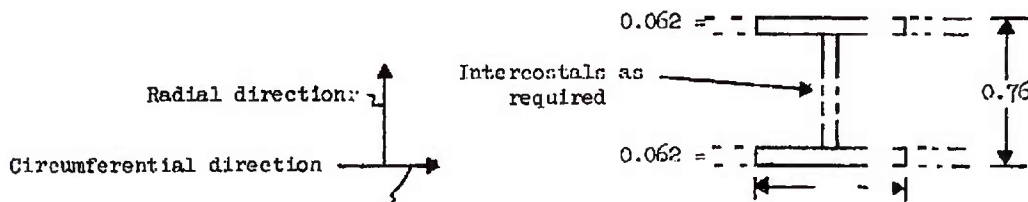
$$f("s") = + \frac{984 \times 20}{3.94} + \frac{14,500(0.50)}{0.631} = +5000 + 11,500 = +16,500 \text{ psi}$$

$$f("a") = + \frac{984 \times 20}{3.94} + \frac{14,500(0.147)}{0.631} = +5000 + 3370 = +8370 \text{ psi}$$

$$f("l") = + \frac{984 \times 20}{3.94} + \frac{14,500(1.067)}{0.631} = +5000 + 24,500 = +29,500 \text{ psi}$$

3.2.2.3 Longitudinal Lip Bending at Station 6.92 (Hole Section)

Bending Moment: 1989 inch-pounds



~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

$$I = (0.062 \times 1.0 \times 0.349^2)2 = 0.015 \text{ in}^4$$

$$f_b = \frac{M \bar{a}}{I} = \frac{1989 \times 0.349}{0.01512} = 45,800 \text{ psi (longitudinal tension on inside surface and longitudinal compression on outside surface)}$$

3.2.2.4 Longitudinal Lip Bending at Station 5.0 (Solid Section)

Depth is 0.40 inch. Take circumferential dimension as 1.00 inch, moment as 925 inch-pounds:

$$f_b = \frac{M \bar{a}}{I} = \frac{925 \times 6}{1.0 \times 0.40^2} = 34,700 \text{ psi}$$

3.2.2.5 Margins of Safety

The stresses of Para. 3.2.2.1, 3.2.2.2, and 3.2.2.3 are all considered as "short time" stresses occurring during a major upset and/or gust. For the purpose of design conservatism, the above stresses are raised by the factor of 1.25 and compared with material yield stresses for short times. At local points where buckling is not a problem, the operating stresses are raised by the factor of 1.10 and compared with the material yield stresses--short time.

3.2.2.6 Circumferential Bending and Tension, Location 0°

Temperature taken as 1100°F; material properties:

$$\text{Tension on outside: MS} = \frac{+105,000}{+16,000 \times 1.25} = 4.25$$

$$\text{Tension at splice: MS} = \frac{+105,000 \times 0.75}{+16,000 \times 1.10} = 4.06$$

$$\text{Compression on inside: MS} = \frac{-105,000}{-17,400 \times 1.10} = 5.82$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

~~ATOMIC ENERGY ACT OF 1954~~

INT 6003

Compression on leading edge (tangential direct n):

Take $b = 2.00$ in., take $t_{\text{effective}}$ as $0.057 + 143$
 $(0.25 - 0.057) = 0.0846$ in.

$$b/t = \frac{200}{0.0846} = 23.6$$

$$F_{cr} = 0.452 E \left(\frac{t}{b} \right)^2 = 0.452 \times 22.7 \times 10^6 \left(\frac{1}{23.6} \right)^2 = 8,400 \text{ psi}$$

(Reference 6, Table B 5.2)

$$MS = \frac{-18,400}{-1.25 \times 5500} - 1 = 1.67$$

3.2.2.7 Circumferential Bending and Tension, Location °

Temperature taken as 1100°F ; material Rene' 41

$$\text{Tension on inside surface: } MS = \frac{+105,000}{+1.25 \times 29,500} - 1 = 1.85$$

$$\text{Tension at splice: } MS = \frac{+105,000 \times 0.8}{+1.10 \times 29,500} - 1 = 1.75$$

3.2.2.8 Longitudinal Bending Stress at Location 0°

Temperature taken as 1100°F ; material Rene' 41

Compression on outside surface from Para. 3.2.: 3

There is a weld splice, so efficiency of joint taken as 0.85:

$$MS = \frac{105,000 \times 0.85}{45,800 \times 1.25} - 1 = +0.56$$

3.2.2.9 Summary Note

Since the above calculations were made there has been a slight modification of skin gages; however, it is believed that changes in gages will not appreciably affect the above analysis.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

3.2.3 Diffuser Skin

3.2.3.1 Bursting Effects

This applies to the inner of double skin

R = 17.88 inches; design pressure inside 400 psi
(maximum which would occur locally between frames)

Pressure outside = 15 psi; P = 30 psi

By conventional formula for bursting of cylinder,

$$f_{HT} = \frac{P \times R}{t} = \frac{385 \times 17.88}{0.10} = 68,750 \text{ psi}$$

Material: Rene' 41

Joint Allowable:

10 hours, 0.2% creep = 0.85 x 92,000 = 78,200 psi

$$MS = \frac{78,100}{1.1 \times 68,750} - 1 = +0.03$$

3.2.4 Load Paths of Unsymmetrical Airloads

3.2.4.1 Loads Normal to Centerline of Spike

The spike geometry and loads normal to the center are shown in Figure 24.

The relation of cone length to diameter:

$$f_n = \frac{1}{d} = \frac{46.1}{30.48} = 1.51$$

Although data are not available for values of f_n down to 1.51 (Reference 7, Tables I and II), it is conservative to use the shortest cone $f_n = 3.0$, to arrive at normal force derivatives and centers of pressure. The value of f_n , the ratio of length to diameter for the cylinder downstream of the cone, is conservatively assumed to be zero.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 452

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

From Reference 7 (Table I), with $f_n = 3.0$ and $f_a = 0$, the change in normal force coefficient, C_n , with angle of attack, α , using the second order expansion method, is

$$\frac{dC_n}{d\alpha} = 1.83 \text{ rad}^{-1}$$

From Table II, $f_n = 3.0$, $f_a = 0$, $X_{cp}/d = 2.00$ (second order expansion method). This places Center of Pressure at $2/3$ the distance from the tip to the base of the $f_n = 3.0$ cone.

At sea level and Mach 3 conditions, the dynamic pressure, q , is

$$q = \frac{\lambda}{2} \rho M_0^2 = \frac{1.41}{2} \times 14.7 \times 3.0^2 = 93.4 \text{ lb/ft}^2$$

The normal force on the cone = $C_n q S \alpha$, where S is the cross-sectional area of the cone base, and α is the angle of attack in radians.

$$\text{Take } S = (\pi) (30.48^2) / 4 = 731 \text{ in}^2$$

Using the coefficient C_n of the preceding page multiplied by 1.25 to cover discrepancies that may exist between the theoretical cone and the actual spike, the normal force per degree angle of attack is estimated to be:

$$F_N = 1.25 \times 1.83 \times 93.4 \times 731 \times \frac{1}{57.3^\circ/\text{rad}} = 2730 \text{ lbs/degree}$$

The fore and aft center-of-pressure location for this load will be taken at 60 percent of the distance from the tip to the base compared to the 67 percent for an $f_n = 3.0$ cone.

3.2.4.2 Cowl and Innerbody Unsymmetrical Loadings

The loads of Figure 25 appear in plan view in Figure 26 along with moment load calculations.

Design conditions selected (see Figure 27) are $\alpha = 6^\circ$ pitch angle of attack for the inlet relative to entering air and 3.6° angle of yaw for the inlet relative to entering air.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ONT 6003

$$R_{EE} = \frac{37,600 (127.86 + 32.60 + 39.06 - 10.4) - 10,840 (127.86 + 32.60) - 87,200}{127.86}$$

$$= (7,500,000 - 1,740,000 - 87,200) / 127.86 = 44,400 \text{ lbs.}$$

Shears, moments, and torques from Y direction components only are shown in Figure 28.

The torques are considered as reacted by the b ly in proportion to the distance from the centerline to the fitting squared at any one station. Such reaction distributions are tabulated in Table XI.

Shears, moments, and torques from Z direction components only are calculated below, and the results are shown in Figure 29.

Distance from (A-A) to centroid of group is

$$\frac{\sum x}{6} = \frac{464.22}{6} = -77.3 \text{ inches} = \text{centroid P of vertical reactions}$$

Loads of Figure 29 are transposed to the centroid and the moment distributed out to the various reaction points as

$$P_z = \frac{mx'}{\sum (x')^2} \quad \text{where minus below indicates compression at joint}$$

$$\sum (\text{vertical load}) = 62,800 - 18,080 = 44,720 \text{ lbs}$$

$$\begin{aligned} \sum (\text{moment about P}) &= 62,800 (199.52 - 77.3 - 10.4) \\ &\quad - 145,200 - 18,080 (199.52 - 77.3 - 39.06) \end{aligned}$$

$$\sum M_P = + 7,020,000 - 145,200 - 1,502,000 = 5,372,200 \text{ in-lbs}$$

Load at any station (from moment) is then

$$p = \frac{5,372,200 x'}{18,867} = 285 x'$$

The calculation for this load is summarized in Table XII.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV.

REPORT 6003

The vertical load is divided equally among six stations.

$$\frac{44,720}{6} = 7452 \text{ lbs/each}$$

To determine fore and aft loads, the structure is broken into segments and the radii and pressures of Figure 30 are applied. The pressures given in Figure 30 represent the symmetrical portion of the asymmetrical load system used for analysis of the cowl lip.

The inside pressure of the innerbody is taken as 62 instead of 110 to obtain conservative panel pressures. The pressure calculations appear in Table XIII.

Loads of Table XIII are now summarized (a p. : load is one acting aft):

Spike less skirt:

$$+7540 + 2400 - 3350 - 6580 = +10 \text{ lbs}$$

Spike including skirt attached to it:

$$+10 - 4370 = -4360 \text{ lbs}$$

Fixed part of innerbody complete:

$$-3500 - 28,100 - 56,600 - 56,300 = 144,000 \text{ lbs}$$

Complete innerbody, including spike:

$$-144,400 - 4360 = 148,760 \text{ lbs}$$

There will also be a drag on the wedge structure between the inlet and the vehicle. The half angle is 8° , and the average height is 5.0 inches. The base width at Section A-A of Figure 18 is 50 inches.

From Reference 8, the pressure ratio across two-dimensional shock at Mach 3.0 for a half angle of 8° is $p/p_o = 1.80$, so that at sea level conditions the pressure behind the shock is $1.80 \times 14.7 = 26.5$ psia. This pressure is considered constant from the entering edge back to Section A-A (Figure 18). The inside pressure will be considered as ambient, 14.7 psia, for the present such that total drag for the wedge structure aft to Section A-A becomes

$$(50 \text{ in.} \times 5 \text{ in.}) (26.5 - 14.7) = (250) (11.8) = 2950 \text{ lbs}$$

$$\left[q \text{ (ref. only)} = \frac{\rho}{2} P_o M_o^2 - \frac{1.41}{2} \times 14.7 \times 3.0 = 93.4 \text{ psi} \right]$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~Atomic Energy Act of 1954~~

ASD-TDR-63-277, Vol. IV.

REPORT 6003

Axial load from the inlet and wedge structure is then:

$$= 148,760 + 2950 = 151,710 \text{ lbs (acting forward)}$$

The effect of inlet forward load on the vertical fitting at the inlet vehicle joint is now considered. The analysis is presented in Figure 29. However, in this case, the couple which is reacted vertically at the six fittings, results from axial loading on the vertical centerline of Section E-E.

Figure 31 shows the fitting. Calculations are presented below.

The couple resulting from forward load on the inlet is:

$$151,710 \times 25.08 = 3,804,000 \text{ in./lbs}$$

Point P is the centroid of the vertical reactions (Figure 29 and Table XII) that must resist this moment.

The load at any station due to the applied moment is then

$$P = \frac{3,804,000 \times'}{18.867} = 199.0 \times'$$

This example is evaluated in Table XIV.

3.2.4.3 Summary Tables of Attachment Reactions

Following the nomenclature of Figure 18, the reactions from airloads of the unsymmetrical condition are listed in Table XV.

Loads marked with an asterisk would act on the aft portion of the split frame at Section (A-A), rather than directly at fitting points (k) and (l).

3.2.5 Strut

The forward load is 148,760 lbs from the aerodynamic pressure to the diffuser double wall for a normal cruising condition (Page

There are three struts to carry this shear load, so the load on each is:

$$\frac{148,760}{3} = 49,587 \text{ lbs}$$

~~SECRET RESTRICTED DATA~~
~~Atomic Energy Act of 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

Radial reactions induced at spar stations (E-E and (F-F)
are:

$$\frac{49,500 \times 6.6}{32.6} = 10,020 \text{ lbs total in radial direction}$$

These reactions are apportioned between inner and outer support rings in inverse proportion to the ring radii cubed for preliminary design purposes (see Figure 32). NOTE: Short time maneuver loads from the innerbody could produce loads of

$$\begin{array}{r} -204,400 \\ -148,760 \end{array}$$

or 1.38 times the above values (see Para. 3.2.5.2).

3.2.5.1 Struts (Normal Symmetrical Loads Alone)

Load on struts from supporting rings are shown in Figure 33.

3.2.5.2 Special Load Conditions Wherein Shock Has Moved Forward From Its Normal Location

With the shock moved well forward, the aft load on the spike is at 40,000 lbs. Using the normal cruising pressures for the fixed portion of the innerbody, the minimum forward load of the entire innerbody is now estimated as follows:

$$\text{Minimum Forward Load} = -148,760 + 4360 + 40,000 = -104,400 \text{ lbs (Reference Page)}$$

With the shock moved well aft, the spike forward load has been estimated at 60,000 lbs. Using the normal cruising pressures for the fixed portion of the innerbody, the maximum forward load of the innerbody is estimated as follows:

$$\text{Maximum Forward Load} = -148,760 + 4360 + 60,000 = -84,400 \text{ lbs}$$

The above may be compared with the normal cruising condition forward load:

$$\text{Normal Forward Load} = -144,400 + 4360 = -140,040 \text{ lbs (Reference Page)}$$

3.2.5.3 Loads on Strut Spars from Unsymmetrical Loads Normal to the Centerline

The total loads from the spike and the fixed portion of the innerbody, as shown on Figure 26, are the resultant loads on a plane rotated 30° 58' from a vertical plane.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV.

6003

$$X - \text{Direction Reaction "R" + "S"} = 600,000 \times \frac{2}{3} + \frac{564,000}{11.82} - \frac{18,100 (2.06)}{11.82}$$

$$= 40,000 + 47,700 - 59,800 = +27,900 \text{ lbs}$$

$$\text{Load "R"} = \text{Load on "S"} = \frac{27,900}{2} = 13,950 \text{ lbs Tension on Joint}$$

3.2.6 Innerbody Ring at Section (F-F)

The innerbody rings, the diffuser ring, and the struts form a redundant system of load paths. However, a simplified and conservative set of loads on the innerbody ring of Section (F-F) has been employed to find the maximum bending moment. The ring is subjected to both transient short time and steady state long time loads.

3.2.6.1 Steady State Long Time Loads

The innerbody forward load is taken as -148,760 lbs (Figures 32 and 33). The loads normal to the centerline of the innerbody are considered the same as those for the 1° angle of attack condition which is one sixth of those in Figure 36.

Steady State Flight Condition Ring Moments are calculated in Tables XVI and XVII.

For the unsymmetrical portion of moments, PR is 1,760 x 10.79 = 40,500 (see Figure 37). Moments for symmetrical portion of loads are carried over from preceding page.

3.2.6.2 Short Time Maneuver Loads

The innerbody forward load is taken as -204,000 lbs, which is 1.38 times normal thrust (see Figure 32 and Paragraph 3.2.5.2). The loadings then appear as shown in Figure 38.

The unsymmetrical load is taken as 7/6 times the vertical load shown in Figure 36.

Ring moments from short time maneuver loads are calculated in Table XVIII, and ring section properties are calculated in Figure 39.

3.2.6.3 Stresses and Margins

Using the ring section properties and the two basic loading condition ring moments determined in the previous pages, the margins of safety of this ring are calculated as follows (Material is Rene' 41):

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Steady State - Long Time Loading:

$$\text{Free Flange } f_{b_1} = \frac{Mc}{I} = \frac{-27,100 \times 1.84}{3.27} = -15,250 \text{ psi (Compression)}$$

$$\text{I.B. Skin near Flange } f_{b_o} = \frac{-27,110 (-1.45)}{3.27} = +12,000 \text{ psi (Tension)}$$

It is proposed to maintain a 10 percent safety factor to material yield and a 25 percent safety factor to structural failure. The yield allowable is taken as the stress level to force a yield of 0.2 percent with 25-hour exposure at 1100°F. The ultimate allowable is taken as the short time 0.2 percent yield stress at 1100°F.

$$\text{MS on Yield} = \frac{-92,000}{1.10 (-15,250)} - 1 = \text{Ample}$$

$$\text{MS on Failure} = \frac{-105,000}{1.25 (-15,250)} - 1 = \text{Ample}$$

Short Time Maneuver Loading:

$$\text{Free Flange } f_{b_1} = \frac{Mc}{I} = \frac{-92,060 \times 1.84}{3.27} = 51,800 \text{ psi (Compression)}$$

$$\text{I.B. Skin near Flange } f_{b_o} = \frac{-92,060 (-1.45)}{3.27} = +40,800 \text{ psi (Tension)}$$

It is proposed to maintain a 10 percent safety factor to material yield and a 25 percent safety factor to structural failure. Short time yield is employed here for the allowable, because buckling could occur when yielding commences.

$$\text{MS on Yield} = \frac{-105,000}{1.10 (51,800)} - 1 = .85$$

$$\text{MS Against Failure} = \frac{-105,000}{1.25 (51,800)} - 1 = +0.62$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

Item 6003

3.2.7 Innerbody Skin

At Section (F-F) of Figure 26, the following are the loads across the section:

Shear forward of section: 21,090 lbs (transverse)

Moment at section: 169,400 in.-lbs

Shear aft of section: $169,400/32.6 = 5,200$ lb (transverse)

Normal shear flow to each longeron from inner body thrust is $49,500/32.6 = 1520$ lbs/in. (see Figure 32).

Normal shear flow to each longeron from short time thrust with shock in abnormal position on the spike is $1.38 \times 1520 = 2100$ lbs/in. (see Figure 37).

Shear buckling forward of Section (F-F):

Panel width, 5.00 inches; radius, 12.5 inches
 t , 0.125 in.

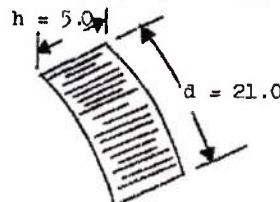
Maximum shear on side

$$\frac{V}{77 R t} = \frac{21,090}{77 \times 12.5 \times 0.125} = 4300 \text{ psi}$$

To estimate critical shear buckling, see Reference 10.

$$\begin{aligned} T_o &= \left\{ (0.10 E \frac{t}{R}) + 5 E \left(\frac{t}{d} \right)^2 \left[1 + 0.8 \left(\frac{d}{h} \right)^2 \right] \right\} 0.75 \\ &= \left\{ (0.10 \times 22.5 \times 10^6 \times \frac{0.125}{12.5}) + 5 \times 22 \times 10^6 \left(\frac{0.125}{21} \right)^2 \left[1 + 0.8 \left(\frac{21}{5} \right)^2 \right] \right\} 0.75 \\ &= \left\{ (22,500) + 3980 [15.1] \right\} 0.75 = 62,000 \text{ psi} \end{aligned}$$

The margin of safety against shear buckling: adequate.



~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

3.2.7.1 Collapse of Skin between Frame Support at Forward of Section (F-F)

Normal pressures are 311 psia outside and 2 psia inside (Figure 30). However, to account for possible higher values of pressure introduced by angles of attack and yaw, a maximum outside pressure of 400 psia with 62 psia inside will be employed here. Therefore, $\Delta P = 338$ (see Reference 11, Page 306, Table XVI, Cond. Q, Support 31):

p' = pressure of collapse from external pressure

$$= 0.807 \frac{Et}{lr} \left[\left(\frac{1}{1 - \nu^2} \right)^3 \frac{t^2}{r^2} \right]^{1/4} = 48 \text{ psia}$$

It is proposed to maintain a safety factor of 10 percent on the peak short time operating pressure:

$$MS = \frac{485}{1.10 \times 338} - 1 = +0.30$$

3.2.7.2 Shear Buckling of Skins between Sections (F-F) and (E-E)

Loading (short time maximum) from Paragraph 3.2.7 is 2100 lb/in. to allow for uneven distributions plus shear from inverse shear (skin shear to each side of strut is taken at 75 percent, or $0.75 \times 2100 = 1575$ lb/in. To estimate critical shear buckling stress as in Paragraph 3.2.7 (except frame spacing is 7.7 and R is 11.50 in.):

$$T_o = \left\{ (0.10 E \frac{t}{R}) + 5E \left(\frac{t}{R} \right)^2 \left[1 + 0.8 \left(\frac{d}{R} \right) \right] \right\}^{0.75}$$

$$= 39,000 \text{ psi}$$

Since it is proposed to maintain a safety factor of 10 percent for this short time buckling shear stress, the margin of safety is:

$$MS = \frac{39,000}{1.10 \times \left(\frac{1575}{0.125} \right)} - 1 = \frac{39,000}{12,860} - 1 = 1.0$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

INT 6003

3.2.7.3 Collapse of Skin Between Frame Supports Aft of Section (F-F)

Design pressure, with some allowance for asymmetry at angles of attack, is taken as

$$1.15 \times 311 = 358$$

$$\Delta P = 358 - 62 = 296 \text{ psi}$$

$$p' * = 0.807 \times \frac{22.5 \times 10^6 \times 0.125^2}{7.7 \times 11.5} \left[\left(\frac{1}{1 - 0.3^2} \right)^3 \frac{0.125^2}{11.5^2} \right]^{1/4} = 341 \text{ psi}$$

Proposed extra margin between operating load and buckling failure is 10 percent.

$$MS = \frac{341}{1.10 \times 296} - 1 = +0.05$$

3.2.7.4 Innerbody Intermediate Frames

Aft of Section (F-F) of Figure 19, the panel spacing is 7.7 inches. Maximum differential pressure is $358 - 62 = 296$ psi. Loading on one ring per inch of its circumference is $296 \times 7.7 = 2280$ lbs per inch. The elastic stability capacity of the ring is computed following Reference 11, Page 295, Table XV, Case 12. The section properties appear in Figure 10.

$$p' = \frac{3 EI}{R^3} = \frac{3 \times 22.5 \times 10^6 \times 0.0817}{11.28^3} = 3810 \text{ lbs per inch}$$

It is proposed to keep a 25 percent extra factor against such a collapse; therefore,

$$MS = \frac{3810}{1.25 \times 2280} - 1 = +0.34$$

*Reference 5, Page 306, Table XVI, Cond. Q, Support 31

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

3.2.8

Thrust Fitting

The critical loads (Figure 41) on this pin occur at the maximum forward load condition of the innerbody--Paragraph 3.2. 2 (-204,400 lbs) and the lateral load of the yaw condition--Figure 27 (44,4 lbs). There will also be a wedge drag of 2950 lbs.

$$\text{Total load} = -204,400 + 2950 = -201,450 \text{ l}$$

ing and shear.

At Section (A-A) through the pin there will be both bend-

Bending is

$$206,000 \text{ lbs} \times 0.70 \text{ inches} = 144,000 \text{ -lbs}$$

Section (A-A) Moment of Inertia is

$$\frac{\pi D^4}{64} = \frac{\pi \times 2.75^4}{64} = 2.82^4 \text{ inches}$$

$$f_b = \frac{Mc}{I} = \frac{144,000 \times 1.375}{2.82} = 70,200 \text{ p}$$

$$f_{s(av)} = \frac{S}{A} = \frac{206,000 \times 4}{\pi \times 2.75^2} = 34,700 \text{ p}$$

Material: Rene' 41

Short time 0.2 percent yield at 1100°F is

105,000 psi

Short time ultimate stress at 1100°F is

152,000 psi

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

6003

Long time (10 hours) 0.2 percent yield at 1100 ° is

92,000 psi

$$MS \text{ (short time yield)} = \frac{105,000}{1.10 \times 70,200} - 1 = +0.36$$

$$MS \text{ (short time ultimate)} = \frac{152,000}{1.25 \times 70,200} - 1 = +0.74$$

$$MS \text{ (long time yield)} = \frac{92,000}{1.10 \times 70,200 \times \left(\frac{145,800}{206,000}\right)^*} - 1 = + 68$$

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date:

OCT 02 2015

$$*\left(\frac{145,800}{206,000}\right) = \text{Ratio} \frac{\text{Normal Operating Load}}{\text{Maximum Short Time Load}} \quad (\text{See Page })$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

4.0 SUBSONIC DIFFUSER AND REACTOR CONTROL SUPPORT STRUCTURE

4.1 Design

4.1.1 Discussion

For purposes of this report, the subsonic diffuser is defined as that part of the propulsion system between Fuselage Station 660 (Figures 1 and 2) and the front face of the reactor. The reactor control support structure is defined as the components within the subsonic diffuser that make up the mechanical linkage between the reactor control rods and the actuator as well as the structure needed to support this linkage and the rods.

Design of the subsonic diffuser, which (from the point of view of the propulsion system) is a pressure duct, is limited to defining aerodynamic lines. These lines are defined to permit optimum performance of the propulsion system with the smallest possible duct size. Mechanical and structural design of the duct is the responsibility of the airframe manufacturer, inasmuch as the duct is an integral part of the airframe.

The design of the reactor control rod actuators, associated linkage, and support structure is a more complex problem for the engine manufacturer than is the subsonic diffuser. For the former, it is necessary to obtain requirements from the reactor company, geometric and environmental limitations from the airframe company, and inputs from the propulsion system control group, and to integrate all these into a well-designed structural system.

Listed below are the factors that have been considered in this design:

1. Stroke: approximately 40 inches
2. Temperature: 1070°-1200°F
3. Rate of travel: Full stroke in 0.75 seconds
4. Minimum passage blockage
5. Maximum axial g load: 25 g
6. Maximum radial g load: 4.5-0.25g
7. Accessibility
8. Friction load between control rods and reactor tubes
9. Air drag loads in rods
10. Air drag loads in the support mechanism

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

11. Limiting deflections of supports, rods, and duct
12. Vibration of supports and control rods 0.36 inches D.A. Frequency, 0-3000 cps
13. Reliability
14. Number, types, and position of control rods: 12 shim and 2 vernier

In designing this portion of the system several approaches have been studied. The design as shown in Figure 2 has been considered most promising from the point of view that it presents the smallest blockage in the diffuser and provides the best accessibility to the actuator.

In the matter of mechanical and structural functions, the air motor, gear box, and control valve are positioned on the outside periphery of the duct to afford greater accessibility for maintenance and to reduce flow blockage in the duct.

A rack and pinion type drive unit is located at the main support strut. The unit is driven by the motor through a shaft that is located inside the strut. The rack is attached to a spider fitting, which picks up four control rods from the reactor, at the downstream end of the rack. The attachment of the control rod to the spider fitting may be either fixed or a quick-disconnect type with manual or remote handling features. Movement of the rack translates the spider fitting along a guide shaft that is supported between the main support and the aft support. The guide shaft also acts as a housing for the feedback transducer, which is geared down and operates off the main rack. When moving upstream, the rack is housed and guided in a tube that is supported between the main support and the forward support.

Figure 2 does not reflect any method of attachment between the supports and the duct. Because of differential thermal expansion, these points will, of necessity, be a slip-fit type of connection.

A detailed design study and subsequent structural analysis of this area must be accomplished before a final decision can be made as to the best system. However, before this study can be accomplished an overall control system study must be made that will more accurately define the requirements of this structure.

4.1.2 Weights and Centers of Gravity

Weights and centers of gravity for the reactor control rod support structure and actuation mechanism are shown in Table XI

4.2 Preliminary Analysis of Control Rod Support Structure

4.2.1 Loads

Weight of control rod = 5.7 lbs

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Weight of rack (estimated) = 1 in.² x 50 n. long x 0.3
 density = 15 lbs

Weight of spider fitting (estimated) = 1 6 in.² blockage

Estimated average thickness = 3 inches

Weight = 13.6 x 3 x 0.3 = 12.25 lbs

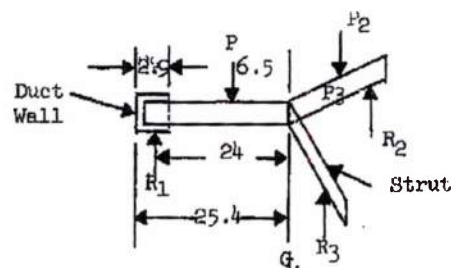
4.2.1.1 Aerodynamic Drag Load Per Strut = 422 lb

Total load per strut (inertia) = (4) (5.5) + 1.5 + 12.25 =
 50 lbs

4.2.1.2 Strut Design Load

Load at the 6g boost condition = (6) (5) 422 = 722 lbs

The twelve coarse control rods are actuated in groups of four and are reacted by three struts as shown in Figure 42. To allow for thermal expansion, the struts are fixed at the center and have a sliding support at the duct wall.



$$P_1 = P_2 = P_3$$

$$R_1 = R_2 = R_3$$

Assume struts fixed at centerline.

Assume only shear load carried thru
 R_1, R_2, R_3 at the duct wall

$$P_1 = R_1 \text{ etc}$$

$$P_1 = 722 \text{ lbs}$$

$$\text{Moment} = 722 (24 - 65) = 12,620 \text{ in-lbs (max)}$$

According to Reference 5 and Figure 43:

$$I_{xx} = ctL^3$$

$$C_1 = f \left(\frac{L_1}{D_1} \right)$$

$$D_1 = 1.50 - 0.05 = 1.45$$

$$\frac{L_1}{D_1} = \frac{5.20}{1.45} = 3.58$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

$$C_1 = 0.20$$

$$Y_1 = 49.09\% L_1$$

$$I = (0.2)(0.05)(5.20)^3 = 1.408 \text{ in.}$$

$$Y_1 = (0.4909)(5.20) = 2.55$$

$$C = 2.55 + 0.025 = 2.58$$

$$f_{bc} = (12,620)(2.58)/1.408 = 23,200 \text{ psi compression}$$

$$f_{bt} = (12,620)(2.67)/1.408 = 24,000 \text{ psi tension}$$

Using 19-9 DX material and assuming that buckling is not critical, then:

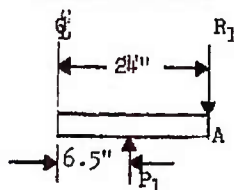
$$F_{TY} = 37,000 \text{ psi at } 1200^\circ\text{F}$$

$$F_{Tu} = 75,000 \text{ psi at } 1200^\circ\text{F}$$

$$E = 22 \times 10^6 \text{ at } 1200^\circ\text{F}$$

$$MS = (37,000/24,000) - 1 = + 0.54 \text{ tensile}$$

4.2.2 Deflection Check of Strut



$$R_1 = P_1 = 722 \text{ lbs}$$

$$l = 24$$

$$a = 6.5$$

$$I = 1.408$$

$$E = 22 \times 10^6$$

4.2.2.1 Deflection at PT A Due to R

$$y_1 = \frac{PL^3}{3EI}$$

4.2.2.2 Deflection at PT A Due to P1

$$y_1 = \frac{P}{6EI} (3a^2l - a^3)$$

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

4.2.2.3 Total Deflection at PT A = $y_1 - y_2$

$$y_1 - y_2 = \frac{P}{6EI} (2l^3 - 3a^2l + a^3)$$

$$= \frac{722}{(6)(22)(10)^6} 1.408 \quad (2)(24)^3 - (3)(6.5)^2 (24) \quad (6.5)^3$$

$$y_1 - y_2 = 0.097 \text{ in.} = \text{Max Deflection}$$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF ID: A6003

5.0 REACTOR SUPPORT SYSTEM

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

5.1 Side Support System Design

During FY 1962 attention has been centered on the minimization of the lateral support annulus dimension in the interest of increasing the aerodynamic performance of the overall Pluto system.

The analytic optimization of the lateral support annulus is based on an assumed dynamic model (see Section 5.2.2) and consists of the optimization of both the radial spring rate and the support spring configuration. The spring rate is optimized to give a minimum required energy storage in the system for the specified design criteria. The spring configuration was selected to achieve a maximum of strain energy storage in the spring for a minimum volume of spring material. A detailed description of this work may be found in Section 5.2.3.

The nuclear reactor core and its reflectors consist of an assembly of approximately 450,000 hexagonal ceramic tubes measuring 0.297 inches across the flats and 2.0 to 4.0 inches in length. The tubes are stacked end to end to form continuous tubes approximately 62.0 inches in length, and these in turn are assembled side to side to form a right circular cylinder 53.25 inches in diameter. The reactor cylinder is aligned axially in the airframe duct.

The core assembly is clamped into the desired cylindrical shape and is supported to the airframe by the lateral support system. The present design concept consists of a series of close fitting, curved pressure pads, which form an expandable shell around the core. They are compressed against the core by radially orientated springs, which in turn bear against a monolithic cylindrical shell that encloses the entire core, pressure pad, and spring assembly.

The pressure pads are supported in radial planes by the springs and in tangential and meridional directions by radially oriented pin-and-socket connections to the pressure shell.

The pressure shell, in turn is supported tangentially by means of a longitudinal tongue and groove rail system, which allows radial growth of the shell with relation to the airframe.

The reactor tube matrix and spring support system are essentially a spring mass system, which is sensitive to vibration loading. The spring system must be so designed that response of the reactor to vibration loads from the airframe will not be excessive.

Weights and centers of gravity are shown in Table XX.

5.1.1 Side Support System Function

The reactor lateral support structure must be capable of (1) insuring an adequate clamping pressure on the reactor matrix, (2) accommodating severe relative thermal expansion between the matrix and the containing

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

1 201 6003

shell, (3) limiting the transverse deflection of the core from inertial and vibration loads, and (4) permitting the reactor to be handled as a complete assembly.

A compressive force must be exerted on the tube matrix at all times to prevent excessive distortion of the tube bundle or gross separation of tubes. The necessity for this compressive force arises from both structural and aerodynamic considerations. Separation induces adverse flow paths in the support system; and tube misalignment, with resulting flow blockage, could be caused by both distortion and separation.

A severe differential thermal expansion exists between the reactor and the surrounding structure; i.e., the reactor, at 2500°F, thermally expands into its supporting shell, which is at approximately 1500°F.

The side support system must furnish adequate support to the reactor when it is subjected to lateral inertia loads. The reactor and the support springs, being essentially a spring-mass system, are sensitive to vibration loads. The support system must prevent the occurrence of excessive reactor response from this type of loads input.

The system should be designed in a way that permits the handling of the reactor as an entity. Such a design will ease ground handling and reactor-vehicle assembly problems.

The above requirements must be met while maintaining the structural integrity of the ceramic tubes. In addition, the structure of the side support system should be contained in as small an annulus as possible, thereby assuring as high a degree of performance as possible for the engine-vehicle system.

5.1.2 Side Support Annulus Thickness to Vehicle Drag Comparison

Since the drag of the vehicle is approximately proportional to the cross sectional area, a 1/8-inch reduction in radius will relieve a 1 percent reduction in drag; i.e., reducing the side support annulus to 1.0 inches will result in a 4 percent reduction in vehicle drag. Considering the low thrust-to-drag margins associated with nuclear ramjets (approximately 10 percent), it is extremely important to minimize the side support annulus dimension.

5.1.3 Reduction in Annulus Width

Considerable effort was expended in 1962 to minimize this annulus width, pointing toward a target of 1.0 inches. The results are illustrated in Figure 44, which depicts the reduction in annulus width versus time for the years 1961 and 1962. The reduction was achieved through optimization of the spring rate and the spring used in the side support system. Section 5.2 contains a detailed description of this work.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5.1.4 Design Side Support System

The resulting side support system is shown in Section (C-C), Figure 45. The system consists of four main components: spring pressure pad, (expansion pad) radial spring, pressure shell, and the support rails.

The spring pressure pad transmits radial loads between the reactor and the spring system. It aids in distributing the concentrated spring loads into the radial reflector. It also carries tangential shear loads due to friction between the pad and reactor, into the support rails.

The spring provides a compressive load during ground handling and flight, contains reactor thermal and inertial movement, and transfers resulting loads to the spring pressure shell.

The spring pressure shell reacts the thermal expansion loads and shears the reactor inertia loads to the support rails. This shell also helps isolate reactor axial movements from those of the airframe. This isolation is accomplished by slicing the shell into short axial sections so that a minimum of the friction load resulting from reactor movement will be transferred to the airframe structure. The support rails transfer reactor inertia loads from the spring pressure shell to the structure of the airframe.

5.1.5 Operating and Installation Requirements

Assumed criteria are outlined for the side support system using either tangentially or radially oriented springs. Figures 46 and 47 show temperatures assumed for structural analysis. Table XXI presents the relative differential thermal expansion that occurs during the system's lifetime. Table XXII summarizes the thermal and inertial conditions used in design analysis. In addition to the above, the following general criteria must also be met.

1. Maximum permitted reactor lateral deflection = 0.100 inches. This requirement arises from considerations of tie rod-to-core interference.
2. No finite separation between the reactor and the support system is permitted for design condition; i.e., the support system must remain in contact with the reflector at all times.

5.1.5.1 Assumptions

1. The structural shell surrounding the reactor remains circular.
2. The tube matrix behaves as a rigid cylinder.
3. The side support structure can be integrated into a single elastic constant or spring rate.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

SI 6003

4. Only the translational mode of the reactor is of interest.
5. Only friction-induced slip damping is present.
6. The dynamic input is a sinusoidal, undamped motion with a frequency of 9.5 cps and a peak load of 7g (see Note 1).
7. Average coefficient of friction on the reflector-support system interface is equal to 0.30.

Note 1. The major vibration loads input to the support system occur as a result of fuselage response, during the low altitude cruise phase of flight to free flight conditions. These conditions are terrain avoidance, atmospheric gusts, and stores ejection. Fuselage response to terrain avoidance is an approximate sinusoidal low frequency motion. That due to stores ejection is sinusoidal but highly damped (5-second duration), while that due to gust input is random. These three fuselage responses combine into a 7g peak load at a discrete frequency. The input used in analysis is conservatively assumed to be a 7g peak, sinusoidal, steady state vibration, occurring at the 9.5-cps fuselage frequency.

5.2 Side Support System Analysis

5.2.1 Nomenclature

- A = Amplification factor (dynamic deflection/static deflection)
- P_o = Magnitude of driving load (lbs/in_a)
- n = Inertia load factor (g)
- W = Reactor weight (185 lbs/in_a)
- V_z = Apparent load on springs (lbs/in_a)
- F = Friction damping load (lbs/in_a)
- μ = Reflector--support system interface friction coefficient
- R = Outer radius of reflector (in.)
- p_o = Reactor preload pressure (psi)
- Δp_T = Change in reactor pressure due to differential thermal expansion (psi)
- Δp_I = Change in reactor maximum pressure due to inertia load (psi)

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

- P_s = Static reactor pressure after differential thermal expansion (psi)
 ΔS_T = Differential thermal expansion (in)
 ΔS_I = Reactor deflection from inertia load (in_r)
 P_m = Reactor dynamic and static pressure (psi)
 f = Driving frequency (cps)
 f_n = Resonant frequency of reactor spring-mass system (cps)
 g = Gravity constant (386.4 in/sec²)
 K_z = Support system integrated spring rate (lbs/in_a/in_r)
 k_r = Support system radial spring rate (psi/in_r)
 E = Spring material elastic moduli (psi)
 T_m = Maximum tensile load in tangential spring systems (lbs/in_r)
 k_c = Circumferential spring rate (lbs/in/in_c)
 V_o = Volume of spring element (in³)
 l = Spring element length (in)
 β = Ratio of distance from end of spring to load point to spring length
 n_p = Number of springs in parallel
 n_s = Number of springs in series
 η = Surface efficiency of the spring
 σ = Allowable stress of the spring material (psi)
 b = Ratio of end thickness to center thickness of the spring
 ψ = Structural efficiency of the spring

Subscripts

- r = Radial
 z = Transverse
 a = Axial

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

- o = initial or preload conditions
- c = circumferential direction

Superscripts

- c = low altitude cruise conditions
- b = boost-transition conditions

5.2.2 Dynamics Model and Optimum Spring Rate

5.2.2.1 Dynamics Model

A dynamic analysis of the reactor and support system, based on an idealized fluid cylinder vibrating in an elastic medium, reference 4, has shown that appreciable excitation of distortional modes of the tube matrix is unlikely in the frequency range of interest (5-30 cps). However, a low frequency resonance could exist corresponding to the rigid-body translation mode of the matrix. This mode occurs at the system fundamental frequency, which is determined by the mass of the reactor and the spring constant of the support structure.

The result of the above dynamic analysis and consideration of the inertia load inputs suggests a simple dynamics model on which the preliminary dynamic analysis of various lateral support structure configurations will be based. The model is a single-degree-of-freedom, slip-damped system.

The radial and tangential configurations are shown schematically in Figure 48. Figure 49 shows the idealized dynamics model with explanatory details.

Load on the reactor mass is

$$V_z = A n W \quad (1)$$

The solution to the amplification factor, A, is well known. Reference 12, page 437 gives

$$A = \frac{\sqrt{1 - \left(\frac{4}{\pi} \frac{r}{P_o}\right)^2}}{1 - f^2/f_n^2} \quad (2)$$

where

$$f_n = \frac{1}{2\pi} \sqrt{\frac{K_z}{M}} \quad (3)$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

The above equation for amplification factor represents an approximate solution of the model. A plot of the exact solution of A vs. f/f_n with F/P_0 as a parameter, may be found on page 438 of Reference 1.

The assumption that the tube matrix reflects as a rigid cylinder permits the solution of the reactor pressure distribution from a static balance of the load. A deflection of the cylindrical matrix an amount ΔS_I (see Figure 50) results in a distribution of deflection given by

$$\Delta S_\theta = \Delta S_I \cos \theta \quad (4)$$

So

$$\Delta P_\theta = \Delta P_I \cos \theta \quad (5)$$

where

$$\Delta P_I = k_r \Delta S_I \quad (6)$$

ΔS_I defined in Figure 50

The differential force dV_z along the axis of displacement is given by

$$dV_z = \Delta P_I R \cos^2 \theta d\theta \quad (7)$$

Summing this force over the periphery gives

$$V_z = \int_0^{2\pi} \Delta P_I R \cos^2 \theta d\theta = \pi R \Delta P_I \quad (8)$$

or

$$\Delta P_I = \frac{V_z}{\pi R} \quad (9)$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPO: 6003

The integrated spring rate, K_z , follows from this

$$K_z = \frac{V_z}{\Delta \delta_r} = \pi R k_r \quad (10)$$

The friction force is derived in a similar manner. The incremental friction force opposing the inertia force is

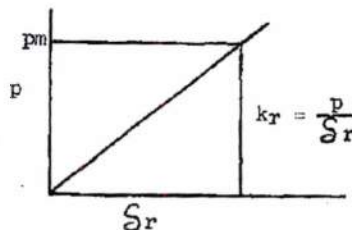
$$dF = \mu p_s \sin^2 \theta d\theta \quad (11)$$

Integration over the periphery gives

$$F = \int_0^{2\pi} \mu p_s R \sin^2 \theta d\theta = 4\mu R p_s \quad (12)$$

5.2.2.2 Optimum Spring Rate

For the spring configurations suitable for the lateral support structure the volume of material required is directly proportional to the maximum value of elastic strain energy that the springs must absorb. For a spring with the following characteristic



the energy stored is

$$W_e^2 = 1/2 p_m \delta_r = 1/2 p_m^2 / k_r \quad (13)$$

Thus, the volume of spring material required is proportional to the parameter p_m^2 / k_r . Since the springs constitute an appreciable portion of the side support structure, and since the volume of the remaining structural material is approximately proportional to the maximum spring load,

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

the least maximum value of p_m^2/k_r will also result in very nearly the minimum size of the entire support structure. The design procedure described below is based on this criterion.

The objective of the design is to limit the value of the reactor inertial deflection ($\Delta \delta_T$) to a given magnitude by selection of the parameters that will result in the least value of p_m^2/k_r . This means that with a given spring rate the static pressure p_s , which is

$$P_B = P_O + \Delta P_T \quad (14)$$

is adjusted so that the friction damping force is of sufficient value to limit reactor deflection to 0.10 inches. By following the above procedure for different spring rates (k_r), a minimum value of p_m^2/k_r is found. The spring rate associated with reactors may be used for design. The results of such a minimization procedure, for the design criteria used, are presented in Figure 51. This indicates that the optimum spring rates are 181 psi/in. (for radial systems) and 140,000 lbs/in._a/in._c* (for tangential systems). These rates correspond to criteria based on the boost-transition design condition. The linear portion of each curve results from the satisfaction of the separation requirement (Paragraph 5.1.5.3) at initial boost condition.

5.2.3 Structural

5.2.3.1 Spring Material Optimization

In the previous section, the spring rate was fixed, resulting in a minimum radial dimension for the side support structure. The spring that furnishes this rate must be as efficient as possible so that additional dimensional losses are minimized.

The strain energy capability of the spring can be expressed by

$$W_e = \frac{1}{2} \int_V \sigma \epsilon \, dV = \frac{\sigma_m^2}{2E} \int_V \left(\frac{\sigma}{\sigma_m} \right)^2 \, dV \quad (15)$$

where

σ = bending stress

σ_m = maximum allowable stress

* This spring rate corresponds to 197 psi/inch in terms of an equivalent radial spring rate.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF 6003

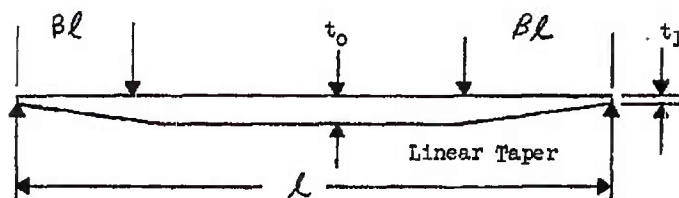
This may also be expressed by

$$W_c = \frac{\psi \sigma_m^2}{2E} \quad (16)$$

where

$$\psi = \frac{1}{V_0} \int_V \left(\frac{\sigma}{\sigma_m} \right)^2 dV \quad (17)$$

in which ψ is called the structural efficiency of the spring. It may be seen that if σ is equal to σ_m throughout the spring material, then 100 percent efficiency is realized. For bending beams, which have good spatial efficiency, the maximum efficiency is realized when the stress distribution over the length of the beam is constant. This stress distribution is approximately realized by a two-point loaded beam with tapered ends:



The moment distribution is



with the resulting stress distribution as



MAC 1072

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

ASD-TDR-63-277 Vol. IV

The thickness variation, which results in a constant stress case, is actually parabolic. However, a parabolic taper is difficult to achieve practically in thin sheet, so a linear taper is used as an approximation. Using Equation 16, the volume of the spring material may be expressed as

$$V_0 = \frac{E \ell}{\eta \psi \sigma_m^2} p_m^2 / k_r \quad (18)$$

since W is proportional to p_m^2 / k_r . The parameter η is the surface efficiency of the spring; i.e., the spring reacts a load of $\ell p / \eta$. The volume of the spring is

$$V_0 = \beta \ell (t_0 + t_1) + t_0 \ell (1 - 2\beta) \quad (19)$$

The structural efficiency is evaluated from the integral in Equation 17 and is

$$\psi = \frac{1}{3} \frac{1 - 2\beta(1 - D)}{1 - \beta(1 - b)} \quad (20)$$

where

$$D = \frac{1}{(1 - b)^3} \left[-\ln b - \frac{1}{2} (1 - b)(3 - b) \right] \quad (21)$$

With Equations 18, 19, and 20, the required total spring thickness is

$$t = \frac{E}{\eta \sigma_m^2} \frac{p_m^2}{k_r} \frac{1}{\psi [1 - \beta(1 - b)]} \quad (22)$$

The spring length, ℓ , may be determined from the spring rate required and the spring beam dimensions, and is

$$\ell = \frac{t}{n_p n_s} \left(\frac{\eta n_p \sigma_m}{3 \beta p_m} \right)^{1/2} \quad (23)$$

Thus, for given values of the parameters the minimum thickness of material required and its associated length can be determined from Equations 22 and 23.

The above design procedure was carried out and resulted in the spring presented in Figure 45. The spring has the following specifications:

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003

Radial free height = 0.857 inches
Radial installed height = 0.807 inches
Spring length = 4.450 inches
Distance to load point = 1.556 inches
Element maximum thickness = 0.092 inches
End thickness (linear taper) = 0.046 inches
Spring rate = 188 psi/in._r (cold)
Spring rate = 181 psi/in._r (380°)
Spring rate = 150 psi/in._r (1400°)
Maximum stress = 109 ksi (boost-transition)
Steady state stress = 37.3 ksi (cruise)
Maximum reactor pressure = 70.7 psi (boost-transition)
Steady state reactor pressure = 25.8 psi (cruise)
Preload reactor pressure = 9.4 psi (cold)
Preload spring deflection = 0.050 inches

5.2.3.2 Component Analysis

5.2.3.2.1 Spring Pressure Shell

Although it is desirable to hold this shell to the minimum thickness possible, the redundancy of the shell geometry makes this difficult to achieve analytically. The following analysis conservatively assumes that each circumferential web and a portion of the shell-in combination act as a ring (see Figure 52). If the web-shell combination is sufficiently stiff, the equivalent ring will be loaded in hoop tension.

Shell loads are critical at boost-transition condition:

Maximum pressure = 70.7 psi
Shell temperature = 400°F
Shell material = Rene' 41
Material allowable = 120 Ksi

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Shell point loads:

$$P = 1/4 \frac{22.5}{57.3} 27.4 (70.7) = 190 \text{ lbs/in}$$

Moment of inertia of Section (A-A):

$$I_{AA} = 0.00610 \text{ in}^4$$

Assume ring loaded at 32 equidistant points and determine deflection from Reference 5 (Case 9, page 158):

$$\delta_{\text{MAX}} = 0.0094 \text{ inches}$$

Deflection is small.

Hoop tension analysis:

$$\text{Diameter} = 55.19 \text{ in.}$$

$$\text{Reflector diameter} = 53.25 \text{ in.}$$

Equivalent ring pressure:

$$P_{EQ} = \frac{53.25}{55.19} 70.7 = 68.2 \text{ psi}$$

$$\text{Hoop Stress} = \frac{68.2 \times 27.50}{1.25} = 15 \text{ Ks}$$

$$MS (\text{Hoop Tension}) = \frac{120}{1.1 \times 15} \times 1 = \frac{11}{11}$$

$$\text{Length between rails} = \frac{\pi 55.19}{16} = 10.6 \text{ in.}$$

$$\text{Rail width} = 1.25 \text{ in.}$$

$$\text{Beam length} = 10.6 \times 1.25 = 9.35 \text{ in.}$$

$$\text{Load/section} = 68.2 \times 10.6 = 724 \text{ lbs/in}_a$$

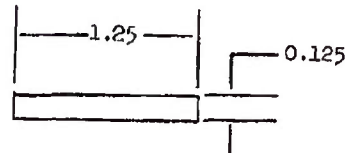
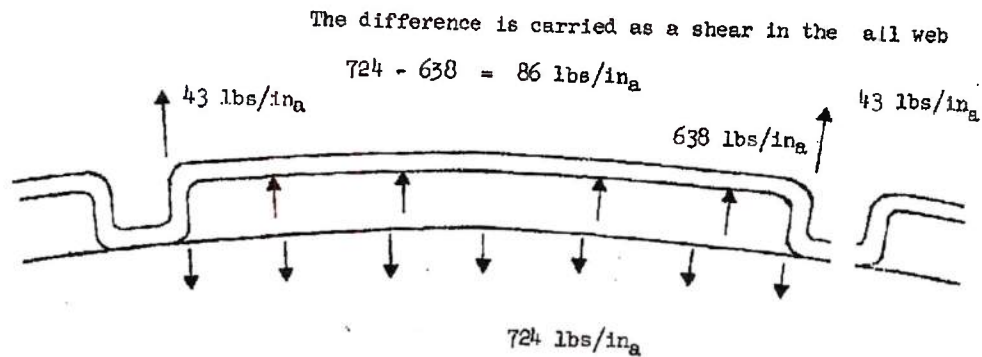
$$\text{Load on web} = 68.2 \times 9.35 = 638 \text{ lbs/in}_a$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003



$$M = \frac{Wl^2}{8} = \frac{68.2 (1.25)^2}{8} = 13.3 \text{ in-lb/ } a$$

Assume a stress concentration factor of 4:

$$f_b = 4 \frac{6M}{t^2} = \frac{24 \times 13.3}{(0.125)^2} = 20.4 \text{ Ksi}$$

$$f_t = \frac{4pr}{t} = \frac{4 \times 68.2 \times 27.59}{0.125} = 60.2 \text{ Ksi}$$

$$f_{\text{total}} = 20.4 + 60.2 = 80.6 \text{ Ksi}$$

$$MS (\text{Bending} + \text{Tension}) = \frac{120}{1.1 \times 80.6} = 1.35$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5.2.3.2.2 Spring Pressure Pad

5.2.3.2.2.1 Pad Lug Bending

beam:

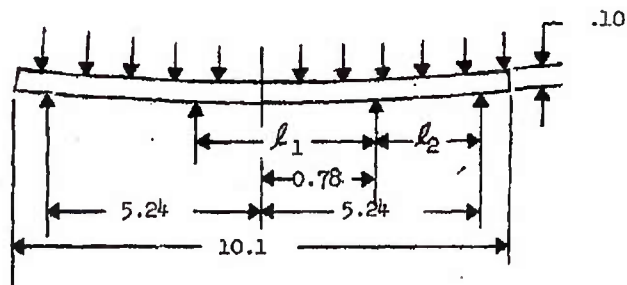
Conservatively analyze the pad as a pressure loaded

Maximum Pressure = 70.7 psi

Pad Temperature = 1400°F

Pad Material = Rene' 41

Allowable = 120 Ksi



$$M_{\text{Max}} = \frac{W}{12} \left(\frac{l_2^3}{l_2 - l_1} - \frac{l_1^3}{l_2 - l_1} \right) = 164 \text{ in-lbs. } n_a$$

$$\sigma = \frac{6M}{t^2} = \frac{6 \times 164}{(0.10)^2} = 98.5 \text{ Ksi}$$

$$MS (\text{Bend}) = \frac{120}{1.1 \times 98.5} - 1 = +0.$$

The pad lugs react the friction load at is produced by reactor movements and is a maximum on the side pads.

Pad lug bending is critical at the static ejection condition.

Pressure = 23.6 psi

Lug Temperature = 1500°F (Assumed)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

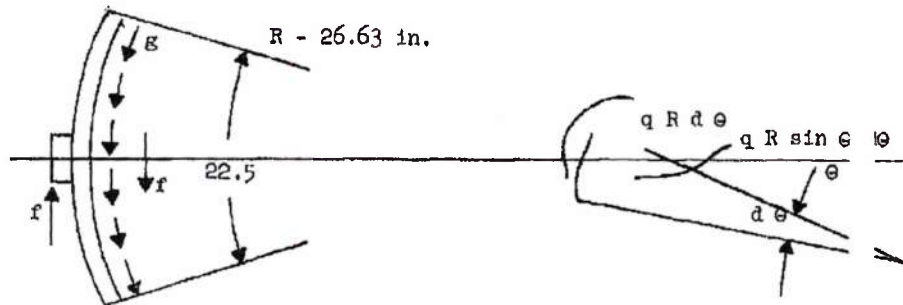
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF 6003

Allowable = 95 Ksi

Lugs are 3.3 inches on center



$$f = \int_{\theta_1}^{\theta_2} q R \sin \theta d\theta$$

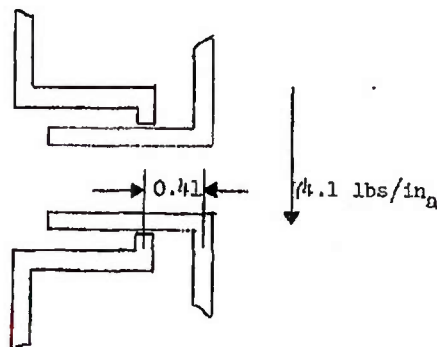
$$q = \mu p \sin \theta$$

So:

$$f = \int_{\theta_1}^{\theta_2} \mu p R \sin^2 \theta d\theta$$

$$= (0.3)(23.6)(26.62) \int_{28.8^\circ}^{101.3^\circ} \sin^2 \theta d\theta$$

$$= 189 \times 0.393 = 74.1 \text{ lbs/in}_n$$



~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-777, Vol. IV

REPORT 6003

$$M = 0.41 \times 74.1 = 30.4 \text{ in-lbs/in}_a$$

$$M = 30.4 \text{ in-lbs/in}_a \times 3.3 \text{ in}_a/\text{lug} = 100.3 \text{ in-lbs/lug}$$

$$z = \frac{77}{64 \times 0.313} \left[(0.625)^4 - (0.375)^4 \right] = 0.0208 \text{ in.}^3$$

$$f_b = \frac{M}{z} = \frac{100}{0.0208} = 4.8 \text{ Ksi}$$

$$MS (\text{Bend}) = \frac{95}{1.1 \times 4.8} - 1 = \text{High}$$

5.3

Front Support Structural Criteria

5.3.1 Discussion of Axial Support System

Axially directed air and inertial forces upon the assembly of hexagonal ceramic tubes comprising the reactor core are reacted by bearing on metal base plates that bear against flanges of tubes extending through the core. These tubes are attached to structural grid located forward of the core and act as tension ties. The grid is connected to the airframe at its periphery.

This axial support system provides for a differential thermal expansion between the ceramic core and the metal structure by means of springs located on the tubes between the core and the grid.

During assembly, tube lengths are adjusted mechanically to clamp the core against the springs and the springs against the grid. The resulting spring deflection preload compresses the core against the airframe face plates and prevents forward movement.

Forward growth of the core due to differential thermal expansion further deflects the springs and increases the spring aft-directed preload. Elastic and plastic stress-induced elongations of the tie tubes partially relieve the spring deflection.

The net force exerted by the springs upon the core must exceed forward-directed core loads if core forward motion is to be prevented.

Forward-directed loading on the reactor core may occur momentarily during several flight phases.

Immediately after rocket booster burnout, an inertia factor of -0.32 g is attained. The net force, however, is less than that of the aft-directed air drag (Refer to Figure 53).

During unstart-restart conditions, the air drag on the reactor core becomes negligible while that on the airframe is unaffected. Inertia of the core will force it forward against the springs.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

6003

5.3.1.2 Discussion of Axial Dynamic Loading Conditions

To obtain insight into the expected dynamic load on a full scale reactor, a one-third scale engine was tested during October 1962. Data from the tests were extrapolated to the full scale system at conditions of Mach 3, an altitude of 1,000 feet, and ANA Hot Day temperature.

Time response was corrected for full scale differences and temperature differences. Since the full scale diffuser is three times longer, the time for a pressure disturbance originating at the inlet to reactor is three times as great and, hence, pressure response data obtained with the one-third scale system should be reduced by a factor of one-third. However, since the test reactor was full length, the time for propagation of pressure disturbances through the reactor remains the same between the one-third scale and neglecting the effects of the nozzle. Both the fore and aft pressure response times were also corrected for sonic velocity differences between the full scale condition T_{T_0} of 1570°R and test data temperature of 900°R. The net result of the scale and temperature corrections was an expanding of the time scale of both front and rear response curves by a factor of:

$$\frac{\text{Full Scale}}{1/3 \text{ Scale}} \times \sqrt{\frac{T_{T_0} - 1/3 \text{ Scale}}{T_{T_0} - \text{Full Scale}}} = 2.3$$

but maintaining the relative lag through the core.

Equivalent full scale pressure levels were obtained by ratioing steady state values recorded for one-third scale system test just prior to inlet unstart to those given in Reference 13 for Mach 3.0, ANA Hot Day, and 1,000 feet. The pressure ratios from one-third scale to full scale were assumed to remain constant throughout the transient.

Typical one-third scale data corrected to full scale conditions are presented in Figure 54. The corrected data are also presented in Figure 55 in terms of ΔP across the full scale core during unstart transient. The effect of such a ΔP change on reactor drag load was evaluated assuming the steady state ΔP was equivalent to the full scale steady state drag (Figure 56) and that drag remained proportional to ΔP throughout the transient.

Based on these assumptions, Figure 57 was prepared and was considered representative of the change in full scale reactor drag load throughout the unstart transient. The curve is presented as a series of straight lines as used to simplify the mathematical analysis. The effects of a hot day pressure response were not considered, and friction in the side support system was neglected. The loads of this curve were then incorporated into the differential equation of motion for the reactor to determine their effects. Figure 5.3.2 presents the derivation of the equations used in this analysis, together with sample calculations. Figures 58 and 59 present the calculated displacements and velocities of the reactor over the time period of the unstart transient.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-777, Vol. IV

REPORT 6003

This analysis indicates that the reactor loads and then abrupt reloads during the unstart transient. F shown in Figure 57 for ground test operation, the forward load sufficient to cause separation from the base block in that el in the tie rods returns the reactor slightly past zero deflect

enables forward the load change are marginally ic energy stored n (Figure 58).

In flight, however, an unstarted inlet 1 drag sharply. Therefore, upon inlet unstart in flight, the re ject not only to forward forces arising from abrupt changes in the propulsion system, but also, since the reactor motion is u forward direction, to the relative deceleration of the vehicle the reactor. Figure 53 presents the estimated reactor drag lo ing an unstart transient that is accompanied by additive 3g ve A deceleration of 3g was selected as representative of a sever condition. In the preparation of Figure 53 it was assumed tha forward load was applied prior to time = 0 and displaced the t from that of Figure 57. The 3g load remained constant over th tion of the unstart transient (to $t = 0.045$) and did not affec dissipated as reload (and restart) occurred. The deceleration assumed to occur linearly over the flat portion at the bottom curve (time = 0.045 to time = 0.057). Offsetting the reload s that of Figure 57) was accomplished to compensate for pressure through the diffuser. The amount of offset and the rate of de decrease were arbitrarily selected to simplify the straight li the drag load curve.

reases vehicle tor becomes sub- ass flow through estricted in the ith respect to variation dur- cle deceleration. inflight drag the 3g additive e = 0 drag level unloading por- the rate, but oed decrease was the 0g drag rt time (from ulse travel time leration load construction of

In relation to the 0g drag load curve (F ure 57), Figure 53 is, therefore, lowered by a factor of 3 times the reactor m t = 0.045 seconds but with the same slope, and shows immediate stay time at minimum load. From time = 0.057 seconds on, Figu identical, since all deceleration loads are assumed to have di vicle reacceleration loads were not considered.

s from $t = 0$ to eload with no s 57 and 53 are ipated and ve-

A mathematical analysis similar to that ground test (0g) condition was conducted for the flight condit: 5.3.2 presents sample calculations of reactor motion under the of Figure 53. Figures 60 and 61 present the calculated results and velocities of the reactor. As shown by these figures, the displacement and velocities increases considerably over that of ground test conditions. A maximum separation from the base bl followed by rapid oscillatory separation at high velocities is though these motion characteristics do not appear to overstres; appears probable that ceramic damage would result from such im;

ducted for the n. Section oad variation t displacements everity of the ained under k of 0.127 inches ndicated. Al- the tie rods, it at load cycling.

It should be noted that the forward reac r springs were ng constant (45 em could proba- ed that such re- design be investigated.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

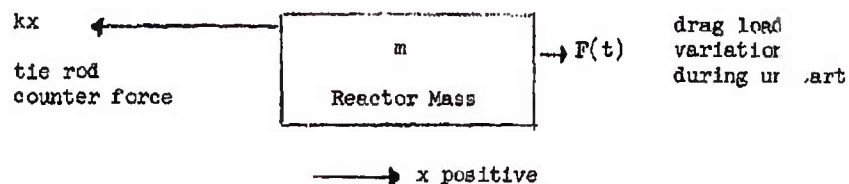
~~SECRET RESTRICTED DATA~~
~~Atomic Energy Act of 1954~~

ASD-TDR-63-277, Vol. IV

RE # 6003

5.3.2 Dynamic Analysis

5.3.2.1 Derivation of Equations for Reactor Dynamic Load Analysis



For the above system

$$m \frac{dx^2}{dt^2} = F(t) - kx \quad (24)$$

or

$$\frac{d^2x}{dt^2} + \frac{k}{m} x = \frac{1}{m} F(t) \quad (25)$$

To solve per Reference 14 let

$$\frac{d^2x}{dt^2} + \frac{k}{m} x = 0 \quad (26)$$

Assume

$$x = e^{Rt} = x_1$$

then

$$\frac{dx}{dt} = R e^{Rt}$$

and

$$\frac{d^2x}{dt^2} = R^2 e^{Rt}$$

ASC 1602

~~SECRET RESTRICTED DATA~~
~~Atomic Energy Act of 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Substituting in Equation (26)

$$R^2 e^{Rt} + \frac{k}{m} e^{Rt} = 0$$

$$\therefore R = \pm \left(-\frac{k}{m}\right)^{1/2}$$

and x_1 , becomes

$$x_1 = c_1 e^{\left(\frac{k}{m}\right)^{1/2} t} + c_2 e^{-\left(\frac{k}{m}\right)^{1/2} t} \quad (27)$$

Assume $1/m F(t)$ can be expressed as $a + bt$

and let

$$x = A + Bt = x_2$$

then

$$\frac{dx}{dt} = B \quad \frac{dx^2}{dt^2} = 0$$

Then Equation (25) can be expressed as

$$\frac{k}{m} A + \frac{k}{m} Bt = a + bt$$

$$\therefore A = a/k/m \quad B = b/k/m$$

and

$$x = x_1 + x_2$$

$$x = c_1 e^{\left(\frac{k}{m}\right)^{1/2} t} + c_2 e^{-\left(\frac{k}{m}\right)^{1/2} t} + \frac{a}{k/m} + \frac{b}{k/m} t \quad (28)$$

which can also be expressed as

$$x = (c_1 + c_2) \cos \left(\frac{k}{m}\right)^{1/2} t + (c_1 - c_2) i \sin \left(\frac{k}{m}\right)^{1/2} t + \frac{a}{k} + \frac{b}{k/m} t \quad (29)$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

Differentiating Equation (28)

$$\frac{dx}{dt} = c_1 i \left(\frac{k}{m}\right)^{1/2} e^{\left(\frac{k}{m}\right)^{1/2} i t} - c_2 i \left(\frac{k}{m}\right)^{1/2} e^{-\left(\frac{k}{m}\right)^{1/2} i t} + \frac{b}{k/m} \quad (30)$$

or

$$\begin{aligned} \frac{dx}{dt} = & c_1 i \left(\frac{k}{m}\right)^{1/2} \left[\cos \left(\frac{k}{m}\right)^{1/2} t + i \sin \left(\frac{k}{m}\right)^{1/2} t \right] \\ & - c_2 i \left(\frac{k}{m}\right)^{1/2} \left[\cos \left(\frac{k}{m}\right)^{1/2} t - i \sin \left(\frac{k}{m}\right)^{1/2} t \right] + \frac{b}{k/m} \quad (31) \end{aligned}$$

Equations (29) and (30) define the displacement and velocity of the reactor during an unstart transient. c_1 and c_2 can be evaluated from known conditions at time = 0; however, x must be positive at $t = 0$. Values of a and b can be determined from test data by defining the unstart transient in terms of successive straight lines (Figure 57).

The value of m was taken as 11,400 lbs from Reference 13, and the steady state load, at Mach 3, 1000-foot altitude, ANA Hot Day, from Figure 56.

The tie rod strain under this load was computed from an effective area and weighted modulus of elasticity for the 81 Rene 41 and 40 R235 tie rods. A tie rod temperature of 1300°F was assumed (Reference 1). Rod length was taken as 80 inches. The initial steady state deflection was therefore:

$$x = \frac{\ell_r}{EA \text{ (no. of rods)}} = \frac{80 (263,000)}{24.5 \times 10^6 (0.095) (121)} = 0.075 \text{ in.}$$

The tie rod spring constant, k , was therefore

$$k = \frac{F}{x} = \frac{263,000}{0.075} = 3.5 \times 10^6 \text{ lbs/in.}$$

5.3.2.2 Sample Calculations of Reactor Displacement and Velocity Changes During Unstart-Ground Test

For conditions at $t = 0$ (Figure 57)

$$x = 0.075 \quad \frac{dx}{dt} = 0$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

the first straight line segment can be expressed as

$$\frac{a + bt}{m} = \frac{1}{m} F(t)$$

therefore,

$$a = 8,900 \quad b = -190,000$$

also,

$$\frac{k}{m} = 118,500$$

$$\left(\frac{k}{m}\right)^{1/2} = 344$$

at $t = 0$, Equation (28) and (30) become

$$x = c_1 + c_2 + \frac{a}{k/m}$$

$$\frac{dx}{dt} = c_1 + \left(\frac{k}{m}\right)^{1/2} c_2 + \frac{b}{k/m}$$

Solving for c_1 and c_2 and substituting

$$c_1 = \frac{1.61}{6.881} \quad c_2 = \frac{1.61}{6.881}$$

Since all constants are now known, x and dx/dt can now be determined for times up to 0.045 seconds:

$$c_1 + c_2 = 0 \quad (c_1 - c_2) = 0.0047$$

$$x = 0.0047 \sin 344 t + 0.075 - 1.61 t$$

$$\frac{dx}{dt} = \frac{1.61}{6.881} (344) e^{344 t} - \frac{1.61}{6.881} (344) e^{-344 t} - 1.61$$

MAC 4472

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Solving at $t = 0.045$,

$$x = 0.003 \text{ inches}$$

and

$$\frac{dx}{dt} = -3.20 \text{ inches/sec}$$

Thus, the tie rods have relaxed from 0.075 to 0.003 and the reactor is moving forward at 3.2 inches/sec. Each straight line segment of Figure 57 is treated similarly, and a , b , c_1 , and c_2 are reevaluated each time. For nose line segments where the velocity reverses direction, the time of zero velocity is determined and the maximum (or minimum) displacement is determined.

5.3.2.3 Sample Calculations for Reactor Displacement and Velocity Changes During Unstart-Flight Operation

It was assumed that the application of the 5g load prior to time = 0 (Figure 53) did not change the reactor motion characteristics at time = 0 from the characteristics under ground test conditions (i.e., $x = 0.075$ and $dx/dt = 0$). This assumption was made to simplify the analysis and was justified because the change in initial drag load was relatively small, and its effect on initial displacement and velocity would also be relatively small.

Therefore, at $t = 0$,

$$x = 0.075$$

and

$$\frac{dx}{dt} = 0$$

Solving for the constants a , b , c_1 , and c_2 as in Paragraph 5.3.2.1,

$$a = 7770$$

$$b = -190,000$$

$$c_1 = \frac{1.61 + 3.26}{688}$$

$$c_2 = \frac{-1.61 + 3.26}{688}$$

Solving Equations (29) and (31) with the known constants at $t = 0.045$,

$$x = -0.016 \text{ inches}$$

and

$$\frac{dx}{dt} = -3.76 \text{ inches/sec}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Thus, the reactor has separated from the base plate and has forward momentum. To evaluate reactor motion over the next straight line segment new equations must be derived since the constants can not be evaluated for negative. Under a negative x , $k = 0$ and the equations are invalid. For $x =$ negative at $t = 0$, Equation (25) becomes

$$\frac{d^2x}{dt^2} = \frac{1}{m} F(t)$$

Expressing $1/m F(t)$ as $a + bt$,

$$\frac{d^2x}{dt^2} = a + bt$$

Integrating for dx/dt and x ,

$$\frac{dx}{dt} = at + \frac{b}{2} t^2 + c_1 \quad (32)$$

$$x = \frac{a}{2} t^2 + \frac{b}{6} t^3 + c_1 t + c_2 \quad (33)$$

At $t = 0$,

$$\frac{dx}{dt} = c_1 = -3.76$$

$$x = c_2 = -0.016$$

From Figure 52,

$$a = -834$$

$$c = 96,000$$

Equation (33) is solved for the time at which x returns to zero. Equations (32) and (33) are only valid up to this time after which Equations (1) through (31) must be used, since k is again the factor.

Setting Equation (33) equal to 0 and solving for t ,

$$0 = -417 t^2 + 16,000 t^3 - 3.76 t - 0.016$$

$$t = 0.033$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

Therefore, after time $0.045 + 0.033 = 0.078$ of Equations (32) and (33) no longer apply for the straight line segment from $t = 0.045$ to $t = 0.090$. The segment from $t = 0.078$ to $t = 0.090$ is evaluated as before using Equations (28) through (31).

The remaining line segments of Figure 53 are evaluated similarly. Equations (32) and (33) are applied whenever x is negative at the start of a line segment. For all line segments times of maximum and minimum velocity are determined to define completely the behavior over the unstart transient.

5.3.3 Structural Summary

According to Reference 13 (page III-29), data for the standard tie rod springs are as follows:

<u>Material Inconel X</u>	<u>Inconel X</u>
Number of springs	103
Outside Diameter, inches	1.75
Inside Diameter, inches	1.32
Spring constant, lbs/in. at 1100°F	45
Free length, inches	4.4
Bottom-out length, inches	2.9
Assembly pre-load, lbs	55.97

There are 121 tie rods, and the total reactor weight supported is approximately 11,500 lbs. Assuming the tubes are equally loaded, $\frac{11,500}{121} = 95$ lbs/tube dead weight loading (1.0g).

Thus, a 1.0g forward inertia load would exceed the 55.97-lb preload by 39.03 lbs and would deflect the spring 0.87 inches. Such movement cannot be tolerated.

If loaded after the core and support structure have reached stabilized temperature levels and all differential expansion has occurred, the additional preload due to the core expansion spring deflection will be available. Total spring travel can be only $4.4 - 2.9 = 1.5$ inches and spring load is then $1.5 \times 45 = 67.5$ lbs. Since this value is less than the 95 lbs/tube load, forward movement would still occur.

The present spring system appears unsatisfactory for the unstart condition. Since the present load data are of a preliminary nature, spring redesign should be delayed until final load studies are available.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5.3.4 Conclusions and Recommendations

1. Reactor damage due to forward loading may occur during ground testing of a full scale Pluto engine if inlet unstarts occurs. Full scale engine dynamic evaluations should be conducted with a dummy reactor prior to any hot core testing.
2. Definite forward loadings are predicted to accompany inlet unstart in flight due to the additional forward loads resulting from thrust loss and drag increase. Modifications to the existing Tory II front support system are indicated.
3. Additional dynamic load evaluations would be accomplished with the finalized one-third scale inlet configuration to ascertain the effects of inlet "hard start" (restart with bypass doors closed) and to determine possible change in the load characteristics with increasing inlet contraction. These tests should be performed at design and off-design Mach numbers.

5.4 Side Support System Test

5.4.1 Discussion

An extensive spring evaluation test program for the Pluto reactor side support system was continued during 1962. Seven ring configurations (Reference 15) were to be investigated to support preliminary design studies:

1. Split cylindrical tube
2. Solid cylindrical tube
3. Modified Belleville
4. Buggy (elliptical)
5. Torsion bar
6. Corrugated
7. Plate

Test programs for evaluating the performance of Belleville corrugated and flat ribbed plate springs (all fabricated from R-235 alloy material) were conducted in the past (Reference 16). Because in early 1962 the Belleville spring exhibited the most desirable high temperature characteristics, al 1962 spring tests were directed toward the evaluation of Belleville spring incorporating increase load capacity. Belleville springs of R-235 alloy material were on hand lending emphasis to this work. Subsequently, concerted analytical analysis of the other spring forms listed above indicated their potential was inferior to the Belleville spring regarding functional requirements.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ONT 6003

Concurrent with the above work, analysis was begun on a new type of spring, embodying the aspects of a tapered curved beam specifically designed to minimize the lateral support annulus dimension. A detailed test program was formulated to evaluate the mechanical performance of this type of spring under expected operating conditions.

5.4.2 High Temperature Springs

5.4.2.1 Belleville Spring

A theoretical analysis of the stress profile of a Belleville spring (Reference 17) shows that very high stress concentrations occur at the inner diameter edges, the higher stress being at the convex edge. Consequently, the actual stresses occurring at both inner diameter edges exceed the allowable yield stress of the spring material and are responsible for the permanent set (local edge material plastic yield) during initial load-deflection operation.

It was assumed that, if the rather sharp inner diameter edges were modified, high stresses would be decreased resulting in reduced permanent set and a more uniform stress distribution across the spring. Particular attention was given to the alleviation of stress concentrations by employing full edge radii.

The test items were conventional form Belleville springs of R-235 material with an optimum solution heat treatment. Nominal dimensions were 0.100-inch material, 2.000-inch O.D., 0.875-inch I.D., and 0.05 inch coned height. These springs were designed with a linear load rate of approximately 85 percent of the spring deflection capability, and a load limit (flat position) of 1800 pounds (Reference 18). Several springs were modified by rounding the inside edges. Full edge radii of 1/64, 1/32, and 1/16 inch were used on either or both edges. Figure 62 shows some typical springs that were tested. A spring containing 1/16-inch full radii on both inner diameter edges is presented in Figure 63.

The spring research effort entailed compressive static load-deflection tests, which were conducted in a Baldwin Universal Test Machine at ambient and elevated temperatures. Appropriate spring holding fixtures, CHAL thermocouples, and Brown temperature recording equipment were used.

Single spring specimens of each of the conventional and modified configurations were tested with and without strain gages at ambient temperature. A nonstrain gaged single spring setup is shown in Figure 64. The loading method shown in the figure is referred to as flat plate loading. The nonstrain gaged springs were first preset by deflection to the flat position, and then subjected to a load-deflection test to 85% of the maximum deflection for three cycles to establish the spring rate.

In order to determine the spring stress profile, one modified spring and one conventional spring (springs identical in physical dimension except inner diameter edge modification) were instrumented with strain gages and tested. SR-4 type A-19 gages were installed on both outside and in-

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

side spring surfaces at mirror view locations equally spaced (angle) in spiral fashion as shown in Figure 65. The spiral method of gage placement permitted average radial-wise strain coverage and also facilitated attachment of electrical leads. The actual test setup for these tests is shown in Figure 66, and details of the "house-of-cards" type of test arrangement are shown in Figure 67. A cone pointed ram was used to prevent damage to gages (as opposed to a flat plate) and yet produce effective flat plate loading. In general, the test load-deflection procedure for gaged springs duplicated the above procedure for non-gaged springs.

In addition to the single spring tests, compressive tests with 10-spring series stacks of conventional springs were conducted. Spring stack load-deflection rate was determined at ambient temperature and constant deflection-load relaxation characteristics at 85 percent of the stack deflection capability were evaluated at 1400°F. Setups of spring stacks for the ambient and elevated temperature tests are shown in Figures 68 and 69, respectively.

The average permanent set of a single conventional spring at initial deflection to flat position was approximately 32 percent of the original coned height (nominal, 0.030 inches). No differences in permanent set were observed between the conventional and any modified springs as shown in Figure 70.

The spring rate for the conventional for spring is linear up to 85 percent of deflection capability, and the load limit is approximately 1845 pounds as noted in Figure 71. These data verify the spring design (Reference 18).

For comparative purposes, one conventional R-235 alloy spring was tested at the same conditions as had been applied to a conventional Rene' 41 alloy spring of near identical physical dimensions (tested in 1961). Figure 72 indicates that the Rene' 41 spring underwent a permanent set of only 42 percent of the R-235 spring set at the comparable deflection points. In Figure 73 the Rene' 41 spring exhibits approximately 12 percent greater load carrying capacity at 85 percent of the spring deflection capability.

An increase in load carrying capability is exhibited by springs tested with a radius on the outer I.D. edge (Figure 74). The spring that had the greatest increase was the one with a 1/16 inch radius. In the figure, a spring with the above outer edge conditions shows an increase of 12 percent in load capacity over that of the conventional spring at the 85 percent level of deflection capability. The increase in load capacity was predicted from the formulae in Reference 17 and is the result of permitting a shorter couple moment arm around the spring center as depicted schematically in Figure 75. Corresponding load-deflection data verified this mechanical phenomenon. Also it will be noted in Figure 74 that no significant performance effects resulted from springs with a radius on only the inner I.D. edge as compared with the conventional spring. In addition to increasing spring load capacity, rounded I.D. edges should also increase spring performance and integrity under a high temperature vibrational load environment.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-777, Vol. IV

PORT 6003

Strain gages were installed on two nearly identical springs, on one conventional spring (No. 29), and on one modified spring (No. 25) having 1/16-inch full radius on both I.D. edges (Figure 65). All gages were positioned to record strains in the tangential direction. Identical loading point for both springs was accomplished by a cone pointed ram (Figure 67). Strain results were obtained for presetting and preset spring conditions during deflection to the flat position. These strain results were converted to stress values and are plotted in Figure 76.

In general, the rather consistent strain data indicate that high local compressive stress at the conventional spring outer I.D. edge was considerably reduced with a full rounded edge. The penalty for stress improvement was manifested in an increase of the remaining spring stress profile as compared to that of the conventional spring. Trend of the modified spring stress data here suggests that a "tear drop" form of edge rounding may reduce the developed stresses in the outer portion of the profile. Experimental stresses, although slightly lower, are in good agreement with the stresses calculated from the formulae in Reference 17.

The load-deflection rate for a preset 10-spring series stack of conventional form springs was found to be in close agreement with the rate of a single spring when tested at both ambient and elevated temperature; i.e., the deflection capability is ten times that of a single spring for the same load. At 1400°F the stack load is a maximum of 83 percent of the corresponding load at ambient temperature as shown in Figure 77. The rate change was predicted and is attributable to the reduction of material modulus of elasticity at elevated temperature.

Load loss of a preset 10-spring series stack due to constant deflection at 85 percent of the stack deflection capability at 1400°F is presented in Figure 78. The test data show an approximate linear load loss rate of 3 percent per hour.

5.4.2.1.1 Conclusions

1. Of the modified Belleville springs tested, the one with a 1/16-inch radius on the outer I.D. edge was found to have the greatest increase in load-carrying capability.
2. Creep test results indicate that a constant relaxation rate of approximately 3 percent per hour was obtained. Extrapolating this to 10 hours will give a total lifetime reduction of 30 percent, which is excessive. The results indicate that redesign of the Belleville spring is required to reduce the lifetime relaxation rate.
3. No future testing is contemplated for the Belleville spring at Marquardt. The curved plate spring appears more attractive from the standpoint of reducing the lateral support annulus.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5.4.2.2 Tapered Curved Plate Springs

The springs presently under development lateral side support of the Pluto reactor are linear rate, tapered, curved plate spring stacks consisting of two sets of parallel spring leaves stacked in series. Single 1.0 inch wide spring leaves and an assembly stack are shown in Figure 79. The spring material is Rene' 41 with an optimum solution heat treatment.

Marquardt for ed, curved aves stacked stack are shown olution heat

The spring leaf is normally 4.450 inches widths of 1 inch, 2 inches, and 4 inches. The nominal height stack of any width is 0.750 inches. Each spring stack had a nominal deflection capability, with a rated load of 366 pounds per inch of spring width.

n length with a spring dinal 0.375- inch of spring

Spring tests at ambient temperature were Universal Tester as shown in Figure 80. A dial gage was used spring deflection, and the readouts of multiple SR4 type A-18 a spring leaf were monitored by an SR-4 strain indicator. Representative strain gage locations are shown schematically in Figure 81.

run on a Baldwin r recording rain gages on sentative

The setup used for high temperature test Temperature Test Machine is shown in Figure 82. This setup in spring stack holding apparatus, 12-inch Hevi-Duty split tube furnace, thermocouples, Brown temperature recorder, Baldwin load cell, meter, and automatic load programming equipment.

in an Elevated lved a special nce, CH-AL ring deflectom-

Spring stack load rate was established for width stacks under conditions of static compressive load-deflection ambient and 1400°F. The magnitudes and effects of transverse spring deflection, and the readouts of multiple SR4 type A-18 a spring leaf were monitored by an SR-4 strain indicator. Representative strain gage locations are shown schematically in Figure 81.

the various ion at both rains were de- mance was in- light trajec- the cyclic

5.4.2.3 High Temperature Spring Test Results

Significant results of the spring tests are as follows:

1. Ambient temperature load-deflection tests for spring stacks 1 inch and 2 inches in width indicate an average data series lead of 10 percent as shown in Figure 84. These spring stacks, in addition a 4-inch stack, exhibited an average of 24 percent greater spring rate than the predicted rate based on nominal spring dimensions. Stack permanent strain averaged 4 percent of stack initial deflection capability. Strain gage data indicated that permanent sets were caused by stress concentrations near a mid-span load points. The test spring rate of a 1-inch stack was 3.4 percent greater than the calculated spring rate of the subject stack based on actual dimensions.

tes for spring ead of 10 per- a 4-inch an the predict- averaged 4 a indicated a mid-span cent greater tual dimen-

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

SPORT 6003

2. Longitudinal material strains of single ring leaves verified the existence of stress concentrations around the 1-span load points as shown in Figures 85 and 86. Comparison of transverse strain data presented in Figures 87 and 88 shows that spring edge area stress increases with increased spring width (tendency of spring edge to turn inward spring side containing longitudinal tension strain).

3. Evaluation of spring rate for 1-inch and 1.5-inch spring stacks at 1400°F (Figure 84) produced data consistent with decreased material modulus of elasticity for the test temperature.

4. Spring stacks, 2 inches in width tested at 1400°F for 10 hours under cyclic load-deflection conditions (Figure 83) exhibited the following uniformly increasing material creep:

	<u>Sector</u>			Total
	A	B	C	Creep
	(in.)	(in.)		(in.)
Stack S/N 2	0.022	0.006	0	0.028
Stack S/N 4	0.024	0.011	0	0.035

These material deformations are attributed primarily to stress concentrations.

5. Ambient temperature recalibrations of spring stacks tested at elevated temperatures produced rates nearly identical to the initial ambient rates for respective stacks.

Conclusions from the significant test results listed above are as follows:

1. Tests indicate that springs can be designed with a 22 percent confidence in the predicted spring rate, based on nominal spring dimensions. This percentage should improve as spring tolerances are reduced.

2. Maximum deviation in the predicted vs. experimental strains was 25 percent. This deviation is probably due to the stress concentration caused by the spring loading bosses.

3. Cyclic, 10-hour creep testing indicated a total relaxation of 0.035 inches in spring-free height. This means a relaxation of 4.1 percent based on a nominal spring-free height of 0.857 inches. This value although based on only two spring tests, should be well within Pluto lateral support design requirements.

4. Future spring tests should be pointed toward increasing confidence in the total relaxation that the spring will undergo when subjected to the Pluto flight loads environment.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5.4.3 Engine-Airframe Lateral Attachment Test -Phases I and II

5.4.3.1 Discussion

During the second quarter of 1962, experimental vibratory tests were conducted to evaluate the characteristics of a proposed engine-airframe lateral support system. The history and results of these tests are contained in Reference 19.

The objective of the tests was to evaluate the assembly integrity, the response modes, and the spring system characteristics of a 360° cross section of the engine-side support system in a vibration loads and elevated temperature environment.

The full scale side support specimen tested consisted of a tangential (to core) corrugated spring array coupled to a track-rail system suspending a simulated core within an outer ring (see Figure 1).

Results of the above test were compared with the predicted dynamic responses as derived from a model analysis compiled in Reference 4. Inconclusive correlation was evident due to (1) the inflexibility of the core-side support assembly and (2) only one possible core preload magnitude. In addition, many difficulties occurred with the major instrumentation at elevated temperature. Analysis indicated the need for evaluation of a generalized test specimen at ambient temperature.

A second full scale engine-side support configuration, somewhat similar to the basic geometry of the first item, was assigned and tested during the last quarter of 1962. The primary test objectives here were to bracket the dynamic test conditions used in the first test and to evaluate the basic dynamic response of the core matrix with variable spring preloads.

The results of the second test are reported in Reference 20.

5.4.3.2 Phase I Test

The test item consisted of a full scale 360° cross section of the side support system linking a simulated reactor core and outer ring. The core outside diameter was 53.25 inches, the outer ring inner diameter was 61.50 inches, and the thickness of the assembled unit was 10 inches.

The principal components of the side support system were expansion shells and tracks, curved linear rate corrugated springs, spring retainers, and rails (as shown in Figure 90). All support components were fabricated from Rene 41 alloy. On assembly, the nominal spring compressive load was approximately 900 pounds per spring, resulting in an average core pressure of 16 psi.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

5. Core resonant conditions in the range of 20 to 26 cps for two horizontal in-line accelerometers are indicated by the data plots in Figures 98 and 99, which are typical of all the data listed in Table XXIII. Apparently, these resonant data are in good agreement with the 21-cps resonant frequency determined from calculations (Reference 4) considering a rigid body made for the core. Corresponding sine wave acceleration data for Figures 98 and 99) indicating core and outer ring dynamic responses are listed in Tables XXIV and XXV.

6. No change in amplification factor was indicated between ambient and 1300°F testing.

7. Phase angle and g level relationships between horizontal in-line accelerometers (Figure 98) indicated a core distortion mode (as opposed to rigid body mode) in the order of 0.02 inches for the frequency band 10 to 40 cps.

8. Flat random excitation data parallel to the sine wave data in that discrete resonant frequencies were not eminent at either ambient or high temperatures.

9. Post-test load-deflection calibration of springs showed no change from the pre-test calibrations.

10. Structural integrity of the side support system was maintained throughout the test.

Conclusions that may be drawn from the results are summarized as follows:

1. Apparently, binding of the rail-traction assemblies was primarily responsible for erratic dynamic response of the core relative to the ring (static tests provided an indication of irregular core peripheral friction distributions).

2. Results of the static g level tests indicate that, as predicted, a sinusoidal core pressure distribution was exhibited by the core matrix when the inertial loading exceeded the core friction.

3. Vertical response of the bottom of the core (Figure 98) compared with horizontal dynamic input indicates distortion "breathing" of the core.

5.4.3.3 Phase II Test

5.4.3.3.1 Test Hardware Design and Description

Test hardware was designed and fabricated for the reaction airframe lateral support dynamic response test (slice test). The major components of this hardware are the reactor core matrix, support ring, and spring support assemblies. The configuration as shown in Figure 100 is 10 inches wide and simulates a section through the reactor airframe structure. The

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

reactor core matrix consists of approximately 78,000 hexagonal steatite tubes (0.3005 inches across flats), 36 peripheral shims, 107 tie rod tubes, and 14 control rod tubes. The outside diameter of the assembled core matrix is 53.250 inches. The core matrix is compressed radially by a series of 18 pad segments that form a cylindrical shell around the periphery. There is no physical connection between these pad segments; however, the compressive force is produced in the core matrix, through the pad segments, by a series of preload spring assemblies. Radially outward, the spring loads are reacted by a rigid support ring that simulates the airframe structure. A track and rail type connection is provided between each pad segment and the support ring to react the shear loads. Seventy-two spring assemblies are used for the entire test fixture, furnishing a spring rate of 150 psi/in. Four spring assemblies are mounted between each pad segment and the support ring. The spring assembly consists of a 3.00-inch O.D. primary coil spring, a 1.00-inch O.D. coil spring deflector, and a spring guide. Provisions are made for adjustment of the spring loads during various test setups. All metal parts are fabricated from mild steel except the ceramic hexagonal tubes and the springs, which are chrome-vanadium.

The assembled test item is presented in Figure 101. Total weight of this test item was approximately 3400 pounds.

5.4.3.3.2 Dynamic Tests

Ambient temperature dynamic tests were conducted on a 28,000 force-pound MB Electrodynamics Shaker Model C210 at a facility outside Marquardt. Desired vertical vibration (Figure 102) was obtained in addition to imposed g loads on the test item at the expected flight magnitude (approximately 8g). The C210 control system is similar to that of the C100.

A typical instrumentation setup is shown in a schematic form in Figure 103. SR-4 Type A-7 strain gages on spring coils were utilized as prime instrumentation regarding core dynamic displacement. Accelerometers and linear motion transducers served as backup instrumentation to the strain gages.

All tests were performed at ambient temperature. Dynamic conditions for nine scheduled runs entailed sine wave sweeps of controlled discrete frequencies through the range of 5 to 300 to 5 cps at 0.5 to 8g. Three core preload pressures of 5, 15, and 30 psi were utilized with g inputs as follows:

Preload (psi)	g Input
5	0.5, 2, 3
15	2, 4, 6
30	3, 6, 8

Results of the static and dynamic tests are summarized as follows:

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

1. Information on dynamic responses of a simulated core matrix to vibration loads was obtained during the subject test program.
2. Displacement data indicated that the core matrix responded dynamically as a right-circular cylinder at the first resonant frequency. Examples of core displacements for several runs are shown in Figure 104.
3. At the second resonant frequency, the core responded dynamically in elliptical form. Core modes at the first and second resonant frequencies for Run 4 are presented in Figure 105.
4. These core mode shapes are in good agreement with theory as presented in Reference 4.
5. The frequencies at which the first and second modes occurred were higher than predicted by a factor of 2. This is thought to be due to an increase in the theoretical integrated spring rate caused by mechanical binding at the reactor periphery.
6. Maximum core displacements relative to the outer ring frame occurred at the 6g input force level and 15-psi core preload. The displacements were approximately 0.100 inches around the bottom vertical centerline and approximately 0.030 inches around each end of the horizontal centerline for the first and second mode conditions, respectively.
7. Apparent core separations observed during the 5 psi core preload and 3g input run were not reflected in the core displacement data.
8. The first resonant frequency of the core increases and displacement decreases with an increase in core preload pressure as shown in Figure 106.
9. Steatite hexagonal tube damage was slight and insignificant. No other apparent structural damage to the test item occurred during the test program.

Conclusions that may be drawn from the results are summarized as follows:

1. The analysis for the theoretical dynamic modes of core deformation as presented in Reference 4 were substantiated to a high degree by test results for first and second resonant frequencies.
2. At input frequencies above approximately 10 cps, core responses were negligible for all conditions tested.
3. Core separation can be prevented with proper values of core pressures during high g load inputs.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003

4. For any core preload, core displacement creases approximately in proportion to increases in input force levels.

5. Steatite tube damage was not detrimental o the operation aspects of the test item.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

6.0 EXIT NOZZLE

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

6.1 Design

6.1.1 Discussion

The exit nozzle is a convergent-divergent ejector type that extends from Engine Station 565.470 to Engine Station 668.070, as shown in Figure 107. The nozzle is cantilevered from and attached to the airframe near the aft face of the reactor. The nozzle-to-vehicle attach joint is a quick-disconnect type, which consists of the vehicle ring, exit nozzle ring, locking ring, and split ring retainer. The exit nozzle attach ring and locking ring are full-threaded, American Standard stub 29° Acme screw thread, with 24 inch lead and 1-inch pitch. The split ring retainer is assembled to the locking ring with a series of 3/8-inch diameter bolts. When fully assembled, a labyrinth type seal is made between the airframe ring and the exit nozzle attach ring. This joint design has the advantages of uniform distributed mass around the circumference to minimize thermal stress, thin sections to reduce gamma heat generation, uniform circumferential load distribution that increases joint structural efficiency, and a single joining member (lock ring) that has optimum quick-disconnect potential.

The nozzle is designed to provide for radial and axial thermal expansion between the nozzle outer shell, nozzle liner, and reactor expansion pads. The aft face of the aft series of reactor circumferential expansion pads is flanged to nest into a radial slip joint in a support ring, thereby allowing radial thermal expansion of the reactor. The nozzle liner forms a slip joint with the inside diameter of the support ring that allows the liner to expand axially. The liner is attached to the nozzle outer shell at the throat area by eight pylons, equally spaced about its circumference.

The exit area between the nozzle outer shell and the exit nozzle shroud is designed to provide a constant annular area in order that boattail drag may be minimized. The exit nozzle is designed to incorporate Rene' 41 material throughout.

6.1.2 Weights and Centers of Gravity

Weights and centers of gravity for the ejector exhaust nozzle are shown in Table XXVI.

6.1.3 Design Data

6.1.3.1 Operational Data

Critical operational phases for the exit nozzle system occur during high altitude cruise and low altitude cruise. Critical pressure loading and temperature environments occur during the following flight operations:

High Altitude Cruise: Mach 3.9 at 35,000 feet
on an ICAO Standard Day

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Low Altitude Cruise: Mach 3.22 at 1000 feet on
an ICAO Standard Day

Pressure and temperature profiles for these operational
regimes are presented in Figures 108 to 113.

6.1.3.2 Configuration

Radial and axial coordinates defining the configuration
of the various components comprising the exit nozzle system are presented in
Figures 114 and 115.

6.2 Structural Analysis

6.2.1 Nozzle Forward Cylinder and Convergent Cone (Primary Shell)

6.2.1.1 Discussion

The ring assembly by which the nozzle is attached to the
airframe extends aft to Nozzle Station 11.7 (E.S. 577.170) and is essentially
a right circular cylinder. The cylindrical shell portion of the exit extends
to Nozzle Station 13.7 (E.S. 579.170) where a 3 1/2-inch radius transition
knuckle joins the cylinder to a convergent circular cone. This is in
turn connected to the small end of a diverging cone by a double curvature
transition section which forms the nozzle throat.

From the nozzle forward station to approximately 10.0
inches aft of the minimum diameter section of the throat the net pressure dif-
ferential between the nozzle inner and outer surfaces is internal, acting out-
ward radially. This bursting pressure decreases with distance from the for-
ward end.

Because hoop tension varies directly with both pressure
and cylinder radius, the requirement for wall thickness also decreases with
distance from the forward end.

Further, because joints are required at intersections
of the cylinders, cones, and transition knuckles, a saving in structural weight
may be achieved by progressively reducing gages at these points. Internal
stresses are small compared to hoop stresses.

6.2.1.2 Operating Condition

Critical operation occurs during low altitude cruise at
Mach 3.22 and 1000 feet on an ICAO Standard Day. Pressure and temperature pro-
files for this phase of operation are presented in Figures 108 and 112.

The shell temperature is nearly constant, varying from a
high of 1225°F to a low of 1215°F.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

6.2.1.3 Material Properties

The nozzle material is Rene' 41 sheet, solution heat-treated at 1975°F for 30 minutes, water quenched, aged at 1650°F for 4 hours, and air cooled.

At 1225°F:

$$F_{t_y} \approx 106,000 \text{ psi (Yield Stress)}$$

$$F_{t_u} \approx 160,000 \text{ psi (Ultimate Stress)}$$

Stress Rupture: 1 hr > 100,000; 10 hrs > 100,000;
100 hrs = 100,000 psi

6.2.1.4 Analytical Factors

Design Factor = 1.15

Factor of Safety = 1.25

Weld Efficiency = 85 percent

6.2.1.5 Analysis for Hoop Tension Loading

$$f = \frac{p R}{1.0 \times t} \quad \text{and} \quad t = \frac{p R}{f}$$

$$t_{\text{required}} = \frac{p R}{F_t}$$

Since the operating temperature is relatively low for this material creep and stress, rupture for the life involved is not critical. The design will be based upon yield stress which will be modified by a 1.15 design factor and an 85 percent weld efficiency factor.

$$F_{t_y} = 106,000 \text{ psi}$$

$$\text{Design Stress} = \frac{106,000 \times 0.85}{1.15} = 78,500$$

$$t_{\text{req.}} = \frac{p R}{78,500}$$

Required material thicknesses are shown in Tab. XXVII.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

SPOT 6003

Minimum Margin of Safety (see Figure 116):

At Station 24, $t_{reqd} = 0.0765$ and $t_{specified} = 0.08$

$$MS = \frac{0.08}{0.0765} - 1 = +0.05$$

6.2.1.6 Analysis for Transverse and Axial Loading

6.2.1.6.1 Discussion

Axial stresses resulting from axial drag and from transverse bending are rarely critical for an exit nozzle structure. However, tension or compression loading producing circumferential and radial stresses govern the gage requirements and are not additive to axial stresses.

6.2.1.6.2 Axial Drag Force

Critical loading occurs at Mach 3.22 and 100 feet on an ICAO Standard Day. $P = 367,438$ -lb limit (Reference: Figure 111)

6.2.1.6.3 Inertia Factors

Referring to design criteria inertia factor specified in Table IV for weapons ejection in low altitude cruise,

$$\begin{aligned} g_z &= +8.50 \\ &= -1.17 \end{aligned}$$

and

$$\begin{aligned} g_x &= +0.3 \\ &= 0 \end{aligned}$$

6.2.1.6.4 Weight and Center of Gravity

The total weight of the nozzle and shroud assembly is 1054 lb and the center of gravity is at Nozzle Station 28.75 (E.S. 4.22)

6.2.1.6.5 Axial and Shear Stresses

Critical loadings are:

$$\begin{aligned} P_{axial} &= 367,438 + 1100 \times 0.3 = 367,768 \\ M_y &= +8.5 \times 1100 \times 30 = 280,500 \\ V_z &= 8.5 \times 1100 = 9350 \text{ lb} \end{aligned}$$

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

Assume these loads act on the 0.093 in. age shell.

$$R_1 = 28.06 \text{ in.}$$

$$R_m = 28.06 + \frac{0.093}{2} = 28.1065 \text{ in.}$$

$$f_b = \frac{M}{\pi r^2 t} \quad f_t = \frac{P_{\text{axial}}}{A}$$

$$f_s = \frac{V}{\pi r t}$$

$$f_b = \frac{280,500}{\pi \times 28.1065^2 \times 0.093} = 1220 \text{ psia}$$

$$f_s = \frac{9350}{\pi \times 28.1065 \times 0.093} = 1142 \text{ psi}$$

$$f_t = \frac{367,768}{\pi \times 2 \times 28.1065 \times 0.093} = 22,250 \text{ psi}$$

$$\text{Combined axial stress} = 22,250 + 220 = 23,470 \text{ psi}$$

$$\text{Shear} = 1142 \text{ psi}$$

6.2.1.6.6 Analysis Factors

$$\text{Design} = 1.15$$

$$\text{Factor of safety} = 1.25$$

$$\text{Weld factor} = 0.85$$

MS (Combined Bending and Axial)

$$MS = \frac{106,000 \times 0.85}{23,470 \times 1.15} = + 2.33$$

MS (Ultimate Shear)

$$F_{su} = F_{tu} \times 0.6 = 0.6 \times 160,000 = 96,000 \text{ psi}$$

$$MS = \frac{96,000 \times 0.85}{1142 \times 1.25} - 1 = \text{High}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.2 Cone-Cylinder and Cone-Cone Intersections

6.2.2.1 General Discussion

The change of slope in the shell membranes at a cone-cylinder intersection produces large localized stresses.

The required thickness of the transition sections may be determined from Formula 3 of Para. UA-4(d), Page 111 of Reference 21:

$$t = \frac{PIM}{2SE - 0.2P}$$

or

$$P = \frac{2SEt}{IM + 0.2t}$$

where

$$M = 1/4 (3 + L/r)$$

P = Internal pressure, psi (use 100 psi as conservative average)

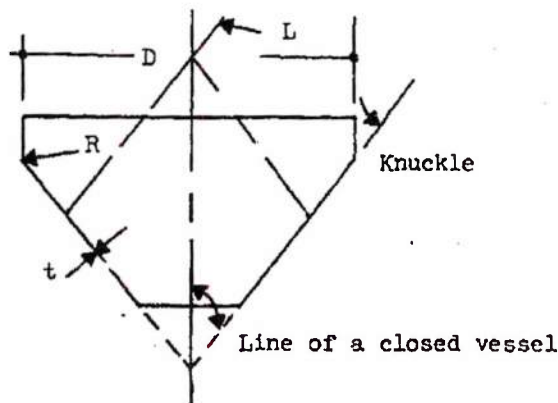
t = Material thickness

S = Maximum allowable working stress, psi

E = Lowest efficiency of joint: 80% (Refer to Para. UW-12, Reference 21)

L = Inside spherical or crown radius, inches (16.485 in.)

r = Inside knuckle radius, inches (21.48 in.)



(Reference: Figure UA-4 (3), Page 110 of Reference 21)

~~SECRET RESTRICTED DATA~~

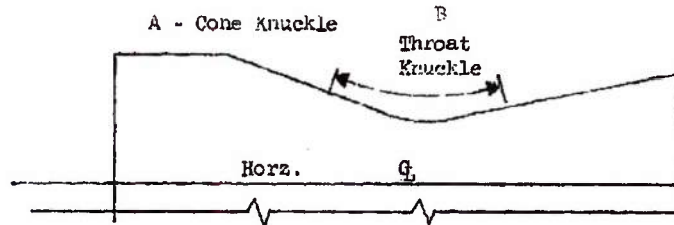
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

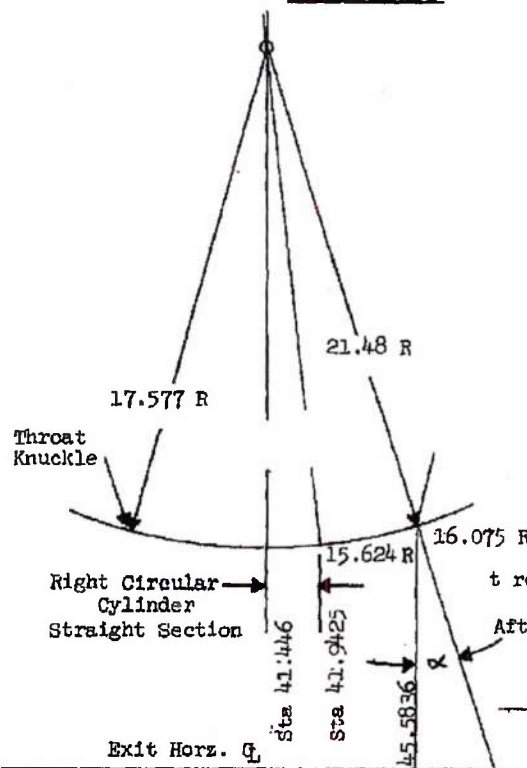
REPORT 6003

The actual exit shape is as follows:



Section A conforms to code shape, but Section B is reversed in that the section representing the head is smaller rather than larger than the cone. The theory still applies, however, since the cone angle is small and the transition fairly flat.

6.2.2.2 Nozzle Throat



$$M = 1/4 (3 + \sqrt{\frac{L}{R}})$$

$$= 1/4 (3 + \sqrt{\frac{16.4}{21.4}})$$

$$= 1/4 (3 + 0.875) = 0.97$$

$$t = \frac{PLM}{2SE + P(M - 2)}$$

$$= \frac{100 \times 16.4 \times 0.97}{(28 \times 0.80) + (0.97 - 0.2)}$$

$$= \frac{1598}{1.68 + 77}$$

S = 106,000 psi and design factor of 1.15

$$S = \frac{106,000}{1.15} = 92,000$$

$$t_{req.} = \frac{1598}{1.6 \times 92,000 + 77}$$

$$= \frac{1598}{147,377} = 0.010 \text{ in.}$$

The gage chosen for the convergent cone and the throat is 0.063 inches. The divergent aft cone is 0.063 inches.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF ID: A6003

By proportion, $\frac{0.0108}{0.063} \times 147,372 = 25,200$ psi. At the throat where $R_1 = 16.1$ inches, the bending moment is approximately $225 \text{ lb} \times 8.5 \text{ gs} \times 32 \text{ in.} = 1,200$ in/lb (assume acting at joint with aft cone)

$$f_b = \frac{M}{\pi r^2 t} = \frac{61,200}{\pi \times (16.1)^2 \times 0.063} = 1,195 \text{ psi}$$

The axial drag is 33,650 lb.

$$\sum f_t = \frac{33,650}{2 \times 16.1 \times 0.063} = 16,600 \text{ psi}$$

$$\sum f_b + f_t = 17,800 \text{ psi}$$

$$\text{Transition stress} + f_b + f_t = 25,200 + 17,800 = 43,000 \text{ psi}$$

$$F_{t_y} = 106,000 \text{ psi and } \frac{106,000 \times 0.8}{1.15} = 73,700 \text{ psi}$$

$$MS = \frac{73,700}{43,000} - 1 = +0.71$$

Notes: 1. This analysis is conservative; the uniform pressure of 100 psi is greater than the actual average pressure, and the axial load effect was included in the original solution for t reqd.

2. For an analysis of bursting hoop stress refer to Para. 6.2.1.

6.2.2.3 Convergent Cone

The required thickness of the conical portion is determined by Formula 4 in Section UG-32-g of Reference 21:

$$t = \frac{PD_1}{2 \cos \alpha (SE - 0.6 P)}$$

$$\alpha = 30^\circ, \cos \alpha = 0.866 \text{ and } 2 \cos \alpha = 1.732$$

$$E = 0.80$$

$$P = 200 \text{ psi (assumed)}$$

$$D_1 = 18.0 \text{ inches}$$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

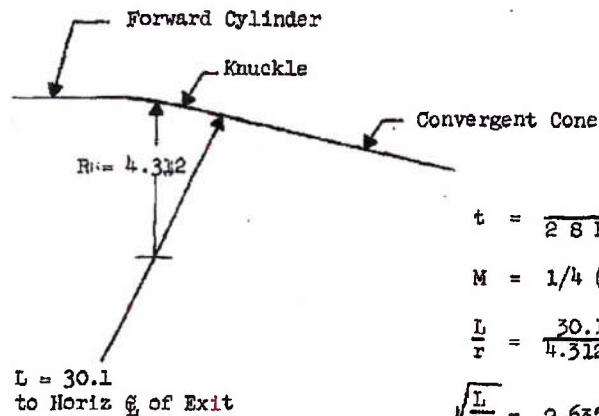
$$S = 106,000 / 1.15 = 92,000$$

$$t_{\text{req.}} = \frac{200 \times 18}{1.732 (92,000 \times 0.8 - 0.6 \times 200)} = \frac{3600}{1.732 \times 73,480} \\ = 0.0284$$

$$MS = \frac{0.08}{0.0284} - 1 = +1.82$$

6.2.2.4 Forward Cylinder - Convergent Cone Intersection

Refer to Reference 21, Section UA-4-d, Equation 3.



$$t = \frac{PLM}{28E - 0. \bar{P}}$$

$$M = 1/4 (3 + \sqrt{4r})$$

$$\frac{L}{r} = \frac{30.1}{4.3125} = 6.975$$

$$\sqrt{\frac{L}{r}} = 2.635$$

$$M = 1/4 (3 + 2.635)$$

$$= 1.409$$

$$P = 200 \text{ psi}$$

$$E = 0.8$$

$$S = 106,000 / 1.15 = 92,000$$

$$t = \frac{200 \times 30.1 \times 1.409}{(2 \times 92,000 \times 0.8) - (0.2 \times 200)} = \frac{8500}{147,000 - 40} = \frac{850}{146,960} = 0.00578$$

$$MS = \frac{0.08}{0.00578} - 1 = +0.38$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.3 Divergent Exit Cone

6.2.3.1 General Discussion

The exit internal pressure decreases rapidly aft of the forward cylinder, and approximately 10 inches aft of the nozzle throat becomes less than that of the annulus. Most of the divergent cone section is subjected to collapsing pressure.

6.2.3.2 Design of Cone

The weight of a cylinder subjected to an external collapsing pressure may be minimized by stiffening a thin shell with rings that hold the shell essentially round. The flexural rigidity of the combination must equal that of the minimum monolithic shell that will support the required pressure.

As the shell thickness decreases, the required weight of the rings increases. The optimum combination for minimum weight must be determined by trial. Preliminary calculations indicate that a 0.063 gage approaches the optimum requirement.

In the following analyses a 0.063 gage of Rene' 41 will be analyzed without stiffeners to determine the strength of the shell alone.

The material to be used is Rene' 41 sheet, solution heat-treated at the mill and solution heat-treated by Marquardt at 1975°F (30 minutes), water-quenched, aged at 1650°F (4 hours), and then air cooled.

6.2.3.3 Pressure and Temperature Data

Operation in low altitude cruise phase at Mach 3.22 and 1000 feet on an ICAO Standard Day produces critical pressure temperature loading (see Figures 108 and 112). Pressure and temperature data follow:

	613.412	623.416	633.416	643.416	653.416	662	668.070
Annulus							
Pressure (psia)	60.22	60.22	60.22	60.22	60.22	59	14.00
Nozzle Internal							
Pressure (psia)	109.00	60.22	37.75	28.50	23.90	20	19.75
Nozzle Δp							
(psi)	+48.75	0	-22.49	-31.72	-36.72	-38	+5.75
Nozzle Internal							
Radius (in.)	15.656	17.80	20.41	22.75	24.46	26	27.259
Nozzle							
Temperature (°F)	1220	1225	1230	1245	1280	135X	1450

The shell axial length is 39.05 inches.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.3.4 Analysis of Ring-Stiffened Cone for External Pressure

The methods employed in Reference 22 for determining the collapsing pressure of thin-walled cylinders with ring stiffeners will be employed to determine the size and location of rings required.

The shell is assumed to be divided into a series of short shells whose lengths are the distance between rings. This assumption applies well and is conservative for a conical shell.

6.2.3.4.1 Ring at Station 660.320

The height of the ring to be used must be limited or the ring will interfere with the cooling annulus airflow.

Shell inner radius = 26.18 inches

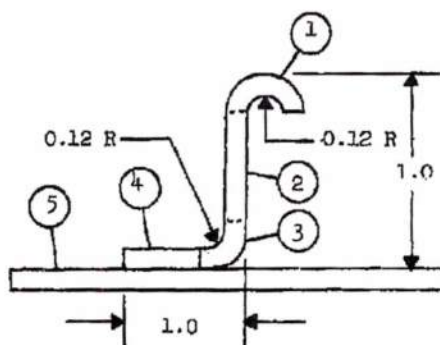
Differential pressure for Mach 3.22 at 1000 feet on ICAO Standard Day = -39.22 psi

Shell temperature = 1325°F

E of Rene' 41 $\approx 24.3 \times 10^6$

The height of a stiffening ring is limited to 1.0 inches by the annulus airflow requirements.

6.2.3.4.1.1 Ring Section Properties



~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPO 6003

30 t = 1.89 inches. Assume effective shell width acting with ring as

Item	Area	y_1	AY_1	AY_1^2	I_{ox}
1	0.0572	0.9426	0.0538	0.0507	0.0003
2	0.0422	0.532	0.02145	0.01142	0.000982
3	0.02106	0.10653	0.00224	0.0002385	0.00006
4	0.0636	0.103	0.00656	0.000676	0.000034
5	0.1190	0.0315	0.00376	0.0001182	0.0000396
	<u>0.30306</u>		<u>0.08781</u>	<u>0.0631527</u>	<u>0.0014156</u>

$$\bar{y} = 0.08781/0.30306 = 0.29 \text{ in.}$$

$$I = 0.0631527 + 0.0014156 - 0.29 \times 0.08781$$

$$0.0392 \text{ in}^4$$

6.2.3.4.1.2 Required Flexural Rigidity

For the case of a ring-stiffened cylinder, refer to Section IV, Page 36, of Reference 23. The shell is assumed divided into a series of short shells whose lengths are the distances between rings. The flexural rigidity, EI_s , of the combined ring stiffener and shell required to prevent collapsing from an external pressure is:

$$EI_s = \frac{w_s D^3 L_s}{24}$$

where

I_s = Required moment of inertia of stiffener and effective width of shell

E = Modulus of elasticity of material = 24.3×10^6

D = Cylinder outer diameter = 52.486 in.

L_s = Ring spacing

w_s = Maximum allowable unit pressure = 39.22 psi

For Station 660.320, the ring is placed close to the thickened throat section. The adjoining ring forward on the shell is spaced 6.0 inches away.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

$$T_3 = 1350^\circ\text{F and } E = 24.3 \times 10^6$$

$$D^3 = 144,000$$

$$I_B = \frac{39.22 \times 144,000 L_B}{24 \times 24.3 \times 10^6} = 0.0392$$

$$L_{B\text{req.}} = 4.05 \text{ in.}$$

Since the shell width between the nozzle and the ring is small, the stiffening effect on the shell is greater than if an equal spacing were used on both sides. A ring spacing of six inches is therefore utilized to the next forward ring.

$$\begin{aligned} \text{Spacing to thickened aft section} &= (660.320 - 624.47) \\ &= 35.85 \end{aligned}$$

$$\text{Spacing to forward ring} = 6.00$$

$$\text{Average spacing} = 4.05 \text{ in.}$$

Since a high margin of safety was obtained on the thickened aft section, the shell is fully effective over the distance to the ring at Station 660.320, and the section is sufficient.

6.2.3.4.2 Ring at Shell Station 651.320

$$\text{Shell I.R.} = 25.20 \text{ in.}$$

$$\text{O.R.} = 25.263 \text{ in.}$$

$$\text{O.D.} = 50.526 \text{ in.}$$

$$\Delta P = 60.22 - 23.5 = 36.72 \text{ psi}$$

$$T = 1285^\circ\text{F (Use } 1300^\circ\text{F)}$$

$$E = 25 \times 10^6$$

$$L_B = 6.0$$

$$I_{B\text{req.}} = \frac{36.72 \times 50.526^3 \times 6.0}{24 \times 25 \times 10^6} = 0.0475 \text{ in.}^4$$

A ring section similar to that used at Station 660.320 will be utilized with an over-all height of 1.5 inches and material gage of 0.063. An effective shell skin width of 2.5 inches is assumed.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003

Item	Area	y_1	Ay_1	Ay_2	I_{ox}
1	0.0433	1.5469	0.0670	0.1033	0.0002
2	0.0668	0.7505	0.05025	0.0377	0.0062
3	0.01579	0.14967	0.00236	0.000354	0.00001
4	0.0512	0.0945	0.00483	0.000456	0.00001
5	0.1575	0.0315	0.00496	0.0001562	0.00001
	<u>0.33459</u>		<u>0.12940</u>	<u>0.1419666</u>	<u>0.00661915</u>

$$\bar{y} = 0.1294 / 0.33459 = 0.3875 \text{ in.}$$

$$I_x = 0.1419666 + 0.00661915 - (0.3875 \times 0.1419666) = 0.11849 \text{ in.}^4$$

$I_x = 0.1185 > 0.0475$ required for 6.0 in. spacing;
 therefore, space 3rd ring at 8.0 inches from
 this (second) ring.

$$MS = \frac{0.1185}{0.0475 \times 1.25} - 1 = + 0.99$$

6.2.3.4.3 Ring at Shell Station 646.320

$$T = 1252^\circ\text{F}$$

$$\text{Inner radius} = 23.69; \text{ outer} = 23.75; \text{ O.D.} = 47.50$$

$$\text{Exit pressure} = + 27 \text{ psia}$$

$$\text{Annulus pressure} = - 60.22$$

$$\Delta p = - 33.22 \text{ psi}$$

$$E \text{ of Rene' 41} = 25.5 \times 10^6 \text{ at } 1250^\circ\text{F}$$

$$I = 0.1185 \text{ in.}^4$$

$$L_{s \text{ req}} = \frac{0.1185 \times 24 \times 25.5 \times 10^6}{33.22 \times 47.5^3} = 20.4$$

$$\text{For } L_s = 8.0 \text{ in.}$$

$$I_{s \text{ req}} = \frac{33.22 \times 47.5^3 \times 8}{24 \times 25.5 \times 10^6} = 0.0463 < 0.1184$$

$$MS = \frac{0.1184}{0.0463 \times 1.25} - 1 = + 1.0$$

Space next ring at 10 inches.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.3.4.4 Ring at Shell Station 636.320

Exit pressure = + 34 psia

Annulus Pressure = - 60.22 psia

$\Delta p = - 26.22$

Inner radius = 21.32; outer = 21.383; D = 42.766

T = 1235°F (Use 1250)

E = 25.5 x 10⁶

$$I_{req} = \frac{26.22 \times 42.77^3 \times 10}{24 \times 25.5 \times 10^6} = 0.0335 \text{ in.}^4$$

Section properties of 1 1/8 inch ring 0.063 gage:

I = 0.0453 in.⁴ with 2.35 in. effect of shell

I = 0.0453 > 0.0335 req.

$$MS = \frac{0.0453}{0.0335 \times 1.25} - 1 = + 0.085$$

6.2.3.4.5 Shell Stations 623.416 to 636.320

At Station 623.416 10 inches aft of the throat, the annulus and exit pressures balance and the cylinder is subject to zero radial pressure.

At Station 633.416, 10 inches further a Δp of 23.22 psi exists. A ring is located at Station 636.320. The average is $\frac{0 + 23.33}{2} = 12.61$ psi. The average radius is $\frac{16.096 + 20.6}{2} = 18.35$ in. The length is 10 inches.

By the method of Reference 22,

$$\frac{D}{t} = \frac{2 (18.35 + 0.063)}{0.063} = \frac{36.826}{0.063} = 585$$

$$\frac{L}{R} = \frac{10}{18.143} = 0.55$$

K > 200

$$W_c = 25.5 \times 10^6 \times 200 \times \left(\frac{0.063}{36.823} \right)^3 = 1.5 \text{ psi} > 12.61 \text{ psi}$$

No rings are required in this area.

$$MS = \frac{12.61 \times 1.25}{25.5} - 1 = + 0.61$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

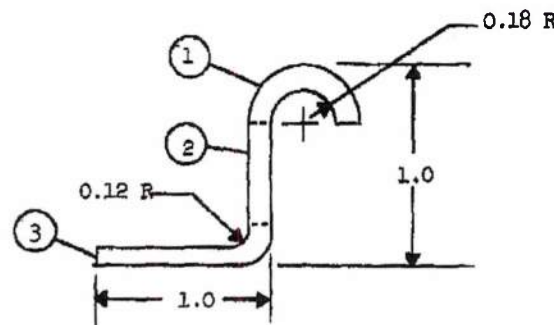
~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

on 6003

6.2.3.5 Stiffening Angles--Stiffening Crippling Stress:

Analysis by the methods given in Reference 23. Refer to Figure 11, page 228 of Reference 23 for equivalent sections.



Material is Rene' 41 sheet of 0.08 gage; temperature is 1400°F; $E = 24.8 \times 10^6$.

Material treatment is solution heat treat at 1575°F for 30 minutes, water quench, age at 1650°F for 4 hours AC.

$$F_{cy} = 118,000 \text{ psi}$$

$$F_{ce} = \frac{\sqrt{F_{cy} E}}{(b'/t)^{0.75}} \quad \text{for individual angle sections}$$

Equation 15, page 229 of Reference 23.

where

F_{cy} = Compressive yield stress

E = Young's modulus in compression

b'/t = Equivalent $b/t = \frac{a+b}{2t}$

C_e = Edge support coefficient

$C_e = 0.316$ for 2 edges free

$= 0.342$ for 1 edge free

$= 0.366$ for no edge free

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

The weighted crippling stress for stiffer is consisting of several equivalent angle sections is:

$$F_{cc} = \frac{\sum (\text{crippling loads of angles})}{\sum (\text{areas of angles})} \quad \text{Reference 23, page 229}$$

$$\begin{aligned} \sqrt{F_{cy} E} &= \sqrt{118,000 \times 24.8 \times 10^6} \\ &= \sqrt{2,820,000,000} = 530,000 \end{aligned}$$

$$\text{Item 1 } F_{cc} = \frac{0.316 \times 530,000}{(2.83) 0.75} = \frac{167,000}{2.18} = 76,700 \text{ psi}$$

$$\text{Item 2 } F_{cc} = \frac{0.366 \times 530,000}{(4.55) 0.75} = \frac{194,000}{3.6} = 54,000 \text{ psi}$$

$$\text{Item 3 } F_{cc} = \frac{194,000}{6.9 \times 0.75} = \frac{194,000}{4.5} = 43,000 \text{ psi}$$

$$\begin{aligned} F_{cc} \text{ combined} &= \frac{(76,700 \times 0.454) + (54,000 \times 0.727) + (43,000 \times 1.105)}{(0.454 + 0.727 + 1.105)} \\ &= \frac{34,800 + 39,300 + 47,500}{2.286} = \frac{121,600}{2.286} \\ &= 53,000 \text{ psi} \end{aligned}$$

Area of ring and skin at Station 660.320 0.303 in.²

$$p = -39.22 \text{ psi}$$

Ring spacing = 6.0 in.

$$p = p_{RS} = 39.22 \times 26.18 \times 6 = 6130 \text{ l}$$

$$P/A = \frac{6130}{0.303} = 20,200 \text{ psi}$$

$$MS = \frac{53,000}{20,200 \times 1.25} - 1 = +1.09$$

Rings at other stations are of smaller diameter and are less heavily loaded; they are, therefore, satisfactory by comparison.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

FORM 6003

6.2.3.6 Exit Nozzle Divergent Cone--Transverse and Axial Loading

The cone is loaded transversely and axially by inertia forces and axially by pressure forces. Since the cone acts as a variable section cantilever beam with the reduced area forward, the critical section occurs at the forward end station where the shell gage changes from 0.08 to 0.063 (Station 623.416).

6.2.3.6.1 Axial Air Load

Maximum axial air load occurs in low altitude cruise at Mach 3.22 and 1000 feet on an ICAO Standard Day. The total aft axial load is 33,650 pounds.

6.2.3.6.2 Dead Weight and Center of Gravity

The weight of the divergent cone aft of the nozzle throat minimum diameter area at Engine Station 613.417 is ≈ 223.6 lbs (total assembly weight). The assembly center of gravity is located 32.6 inches aft of the throat. The weight of the 0.08-inch section is ≈ 38.1 lbs, and its CG is 5.8 inches aft of the throat.

6.2.3.6.3 Weight and CG of 0.063-in. Section Aft of Station 623.416

$$W = 223.6 - 38.1 = 185.5 \text{ lbs}$$

$$CG = \frac{(223.6 \times 32.6) - (38.1 \times 5.8)}{185.5} = \frac{7680 - 221}{185.5} = 40.2 \text{ inches}$$

$$\text{Arm to Station 520.412} = 40.2 - 10 = 30.2 \text{ inches}$$

$$M = 185.5 \times 30.2 = 5600 \text{ in.-lbs (dead weight)}$$

$$V = 185.5 \text{ lbs}$$

6.2.3.6.4 Inertia Load Factors

The critical combined y-z load factor acting at the missile center of gravity is ≈ 5.0 g. A conservative value of 10 acting on the exit nozzle is assumed for this analysis.

6.2.3.6.5 Transverse Shear and Bending at Station 623.416

The bending stress at a section in a right circular cylinder is:

$$f_b = \frac{M}{\pi r^2 t}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

For a truncated cone, $f_{b1} = f_b \sec \beta$ where β is the angle between the axial center line and meridional lines in the shell. Assuming a straight surface between throat and exit,

$$\beta = 10^\circ 55'$$

$$\sec \beta = \frac{1}{\cos \beta} = \frac{1}{0.98190} = 1.02$$

$$R \text{ at Station } 623.416 \approx 17.5 \text{ inches}$$

$$f_b = \frac{M}{77 \times 35^2 \times 0.063} = \frac{M}{242}$$

$$f_{b1} = 0.00422 M$$

$$\text{For a 1.0 g inertia } f_{b1} = 0.00422 \times 50 = 23.6 \text{ psi}$$

$$10 \text{ g} = 236 \text{ psi}$$

The transverse shear stress at Station 623.416 is:

$$f_{s1} = \frac{V_1}{77 r t}$$

where

$$V_1 = V - \frac{M \tan \beta}{r}$$

for 1 g

$$V_1 = 185.5 - \frac{5600 \times 0.19287}{17.5} = 185.5 - 61.7 = 123.8 \text{ lbs}$$

$$f_s = \frac{123.8}{77 \times 17.5 \times 0.063} = 35.7 \text{ psi for 1.0 g}$$

$$= 357 \text{ psi for 10 g}$$

6.2.3.6.6 Axial Air Load Stress

Total air load at throat - 33,650 lbs Since the section is obviously over strength, assume the same load at Station 623.416.

$$\text{Section area} = (2 \times 17.563) 77 \times 0.063 = 7.05 \text{ in.}^2$$

$$P/A = \frac{33,650}{7.05} = 4780 \text{ psi}$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

6.2.3.6.7 Combined Axial Stress

Conservatively utilizing an inertia load factor of 10.0 g, the full cone drag load, and a design factor of 1.15, the design limit stress is:

$$f_t = (236 + 4780) (1.15) = 5770 \text{ psi}$$

6.2.3.6.8 Material Properties

The temperature at Shell Station 623.416 is 121 °F, and the F_{ty} of Rene' 41 sheet is 107,500 psi.

Assuming a weld joint efficiency of 85 percent

$$F_{tu} = 150,000 \text{ psi}$$

Assuming

$$F_{su} = 0.5 F_{tu},$$

$$F_{su} = 75,000 \text{ psi}$$

6.2.3.6.9 Margins of Safety

$$\text{MS Combined Tension Yield} = \frac{107,500 \times 0.85}{5770} - 1 \quad \text{High}$$

$$\text{MS Combined Tension Ultimate} = \frac{150,000 \times 0.85}{5016 \times 1.25} - 1 \quad \text{High}$$

$$\text{MS Ultimate Shear} = \frac{75,000 \times 0.85}{357 \times 1.25} - 1 \quad \text{High}$$

Note: Loads utilized in this analysis of combined axial-transverse, while not the true values, were obviously conservative, and in view of the high margins obtained are deemed sufficient.

6.2.3.7 Exit End Doubler Ring

6.2.3.7.1 Discussion

From Exit Station 662.470 to the aft end at Station 668.070, a 1/4-inch thick plate ring is utilized to stiffen the unsupported open end of the 0.063 gage conical shell (Figure 117). A similar ring is provided on the airframe shell that encloses the nozzle.

The annulus formed between these shells is utilized as a cooling air duct. The shells converge toward the open end, and by the end portions of the stiffening rings a convergent-divergent cooling air exhaust nozzle is formed.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.3.7.2 Pressure Loading

The divergent cone is subjected to a net external collapsing pressure differential. The critical pressure-temperature combination occurs during low altitude cruise on an ICAO Standard Day at Mach 3.22 and 1000 feet. Pressure profiles for the operation are presented in Figure 110. The following are data from these curves:

	Exit Station			
	662.470	665.470	666.97	668.070
Annulus pressure, psia	-59.0	-54.7	-48.5	-14.00
Nozzle pressure, psia	+20.2	+20.0	+20.0	+19.75
Δp on nozzle	-38.8	-34.7	-28.5	+ 5.75

The axial forces acting on this small section are negligible.

6.2.3.7.3 Temperature

The temperature through the shell thickness is assumed uniform. The temperature profile for Mach 3.22 at 1000 feet on an ICAO Standard Day is presented as Figure 112. Values for the ring are as follows:

	Exit Station			
	662.470	665.470	666.97	668.070
Temperature, °F	1350	1392	1425	1450

6.2.3.7.4 Material

The material is Rene' 41 plate, mill annealed and process-treated as follows:

Solution heat-treat at 1975°F (30 minutes); water quench; age at 1650°F (4 hours); air cool.

6.2.3.7.5 Analysis by Method of Reference 24

In this report Donnell's equation for the equilibrium of stresses under other loading conditions for cylinders with simply supported edges.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REP 6003

6.2.3.7.6 Critical Buckling Stress for Uniform External Pressure

Pressure applied to entire outer surface radius r ; ends open; edges simply supported.

Solve for nondimensional buckling parameter,

$$z = \frac{r^2}{rt} \sqrt{1 - \mu^2}$$

Obtain buckling constant,

$$k_y = f_y \left(\frac{t L^2}{D \pi^2} \right)$$

Solve for f_y = critical compressive hoop stress

Values apply where $n = 1 < 2$

$$D = \text{plate flexural stiffness/unit length} = \frac{t^3}{12 - 2)}$$

Assume

$$\mu = 0.3$$

$$t = 0.25 \text{ in.}$$

$$L = 6.0 \text{ in.}$$

$$r = 26.9 \text{ in. (average)}$$

$$E = \text{at } 1375^\circ\text{F (average)} = 24.3 \times 10^6$$

$$D = \frac{24.3 \times 10^6 \times (0.25)^3}{12 (1 - 0.3^2)} = 34,700$$

$$z = \frac{(6^2)}{(26.9)(0.25)} \sqrt{(1 - 0.3^2)} = 5.09$$

$$r/t \sqrt{1 - \mu^2} = \frac{26.9}{0.25} \times 0.951 = 102.2$$

From Figure 1 of Reference 24:

$$k_y = 4.7$$

$$f_{y\text{cr}} = \left[\frac{k_y}{\left(\frac{t L^2}{D \pi^2} \right)} \right] = \frac{4.7}{\left(\frac{0.25 \times 6^2}{34,700 \times \pi^2} \right)} = \frac{4.7 \times 34,700 \times \pi^2}{0.25 \times 6}$$

$$= 178,000 \text{ psi}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

For f_c (hoop compression), assume average pressure of 35 psi over ring width.

$$f_c = \frac{35 \times 26.9}{0.25} = 3770 \text{ psi}$$

$$\text{Design factor} = 1.15$$

$$\text{Factor of safety} = 1.25$$

$$MS(\text{Ultimate}) = \frac{178,000}{3770 \times 1.25} - 1 = \text{Hig}$$

6.2.3.7.7 Analysis by Method of Reference 5

Refer to Reference 5, page 270, Case 1 Table XIII. This theory applies to a more concentrated loading than the actual case involved. A conservative assumption will be used in which the entire pressure acting on the ring is concentrated at the end.

Then:

$$\text{Max } s_2 = \frac{-2 V_0}{t} \lambda R$$

$$V_0 = \text{transverse shear normal to wall lbs/in.}$$

$$\lambda = \sqrt[4]{\frac{3(1-\mu^2)}{R_2^2 t^2}}$$

$$R_2 = R = \text{mean radius of shell}$$

$$\mu = \text{Poisson's ratio} = 0.3$$

$$t = 0.25 \text{ in.}$$

$$s_2 = \text{hoop stress}$$

$$\lambda = \sqrt[4]{\frac{3(1-0.3^2)}{26.9^2 \times 0.25^2}}$$

$$= \left(\frac{2.71}{4.52}\right)^{1/4} = (0.6)^{1/4}$$

$$= 0.88$$

$$V_0 = p (\text{assume } 35 \text{ psi average for } 1.0 \text{ inch width})$$

$$s_2 = \frac{-2 \times 35}{0.25} \times 0.88 \times 26.9 = 6620 \text{ psi}$$

$$\text{Station } 662.470 \text{ to } 668.070 = 5.6 \text{ inches}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

at the end: If it is assumed that the entire load is concentrated

$$V_0 = 5.6 \times 35 \times 82 = 5.6 \times 6620 = 37,000 \quad 1$$

$$F_{ty} \text{ of Rene' 41 at } 1450^\circ\text{F} = 127,000 \text{ psi (r stock)}$$

$$\text{Design factor} = 1.15$$

$$MS (\text{Yield}) = \frac{127,000}{37,000 \times 1.15} - 1 = + 1.98$$

6.2.4 Ejector Shroud (Nozzle Inner Liner)

6.2.4.1 Discussion

The inner liner acts as a second nozzle and extends aft to the exit nozzle throat. It serves as a heat shield between the exhaust flow and the primary nozzle. Airflow from the lateral support annulus is ducted between these exit nozzle shells and used for nozzle cooling.

Exit nozzle exhaust flow pressure is less than that in the cooling annulus. The shroud is subjected to a collapsing pressure.

Radial components of pressure loadings are reacted within the shroud. Axial components are transferred to the exit nozzle throat by means of radially oriented pin and socket supports. Axial forces are reacted similarly. The shroud acts as a simply supported beam to transfer radial inertia loads to the exit throat and the airframe at the reactor aft end.

6.2.4.1.1 Method of Analysis

Analysis of the cylindrical and conical shells is based on the methods of Reference 22 previously utilized in Section 6.2 for the divergent cone aft section of the exit nozzle.

6.2.4.1.2 Operational Regimes, Pressure and Temperature

Critical pressure and temperature regimes occur during low altitude cruise operation at Mach 3.22 and 1,000 feet on an ISO Standard Day.

Data related to this operational phase of the regime are presented in Figure 108 and in Table XXVII.

6.2.4.1.3 Materials

The material used is Rene' 41 sheet stock processed solution heat treated at 1975°F for 30 minutes, water quenched, and air cooled. The following strength values apply at 1650°F

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

Temperature (°F)	$F_{ty} \times 10^{-3}$	$E \times 10^{-6}$
1350	104.8	25
1400	102	24.3
1450	99	23
1500	96	21.5
1550	93	19.5

Assume

$$F_{cy} = F_{ty}$$

$$F_{su} = 0.6 F_{tu}$$

6.2.4.2 Analysis of Forward Cylinder

The forward 22.25 inches of the liner is a right circular cylinder with an I.D. of 53.5 inches. The open end is stiffened by a heavy doubler ring and is simply supported in the radial direction by the flange of the support ring attached to the airframe. The liner ring slides inside this ring and is unrestrained in the axial direction.

Critical operation occurs in low altitude cruise at Mach 3.22 and 1,000 feet on an ICAO Standard Day. The net differential pressure between the inner and outer shell surfaces is compressive and varies from 66 psi at the forward end to 26 psi at the aft end.

Utilizing the methods of Reference 22:

$$L = 22.25 \text{ in.}$$

$$R = 26.75 \text{ in.}$$

$$L/R = \frac{22.25}{26.75} = 0.833$$

$$D = 53.50 \text{ in.}$$

The average shell temperature is 1350°F and E is 25×10^6 psi.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003

t	D/t	K	t/D	$(t/D)^3$	W_c (psi)
0.063	850	220	0.001178	1.63^{-9}	8.95
0.080	667	190	0.001493	3.34^{-9}	15.85
0.093	576	175	0.001735	5.23^{-9}	22.95
0.109	490	160	0.00204	9.45^{-9}	37.80
0.125	428	140	0.00234	1.285^{-8}	45.10
0.156	343	135	0.00292	2.5^{-8}	84.2
0.1875	285	130	0.00351	4.32^{-8}	140.5

The forward edge of the shell aft to Station 568.47 is 0.1875 gage material. Ring grooves are machined in this area to provide a pressure seal, but the majority of the section is full thickness.

Maximum $p = 66.0$ psi at the forward end

$$MS (t = 0.1875) = \frac{140.5}{66 \times 1.25} - 1 = 0.70$$

The shell is spin-formed and is assumed taper in thickness from 0.1875 at Station 568.47 to 0.125 at Station 580.47, here $p = 31$ psi and $W_c = 45.10$ psi

$$MS (t = 0.125) = \frac{45.10}{31 \times 1.25} - 1 = +0.16$$

6.2.4.3 Analysis of Aft Cone (Convergent)

Analysis methods are those utilized in the study of the exit shell (Refer to Paragraphs 6.2.1 and 6.2.3).

$L = 18.0$ inches

R fwd = 27.5 inches; r aft = 18.25 inches; average $r = 23.0$ inches

Average temperature = 1450°F

E of Rene' 41 = 23×10^6 psi

$$L/R = \frac{18}{23} = 0.782$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WWS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

t	D/t	K	t/D	(t/D) ³	W _c (psi)
0.063	730	225	0.00137	2.58 ⁻⁹	13.4
0.080	575	200	0.00173	5.17 ⁻⁹	23.8
0.093	495	185	0.00202	8.22 ⁻⁹	35.0
0.109	422	175	0.00237	1.33 ⁻⁸	53.5
0.125	378	160	0.00272	2.01 ⁻⁸	74

At the forward end, p = 27 psi and R = .75 inches.

At the aft end, p = 44.3 and R = 18.27 inches.

For a 23-inch average R, W_c = 74 psi for a 1/8-inch gage.

W_c for R = 27.75 on the forward cylinder = 45.10 psi.

$$MS \text{ (forward end)} = \frac{45.10}{27 \times 1.25} - 1 = + .333$$

At the aft end the radius of 18.25 will provide a higher W_c than the 74.0 psi for the average R = 23.0 inches. The W_c value for R = 16.0 inches in the throat area is greater than 100 psi. The 0.125 gage, therefore, is deemed satisfactory.

$$MS = \frac{74}{44.3 \times 1.25} - 1 = + 0.33$$

6.2.4.4 Analysis of Throat (Aft End)

The axial length from the point of tangency with the convergent conical shell to the open aft end is approximately 10 inches. The variation in shell radius is small, and the average radius is 16.4 inches. The maximum collapsing pressure is 140 psi at the throat mid-point and decreases to zero at the open end (see Figure 118).

Utilizing the method of Reference 22,

$$L/R = \frac{10}{16} = 0.625$$

Assume t = 0.109 in.,

$$D/t = \frac{32}{1109} = 293, K \approx 225$$

t = 1750°F maximum and E = 19.5 x 10⁶ psi minimum

$$\begin{aligned} \left(\frac{t}{D}\right)^3 &= \left(\frac{0.109}{32}\right)^3 = 3.97 \times 10^{-8} \\ W_c &= 225 \times 18.5 \times 10^6 \times 3.97 \times 10^{-8} = 31.5 \text{ psi} > 140 \text{ psi} \end{aligned}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

1 OCT 6003

Using $t = 0.125$,

$$D/t = \frac{32.0}{0.125} = 256, K \approx 210$$

$$t/D^3 = 5.93 \times 10^{-8}$$

$$W_c = 210 \times 19.5 \times 10^6 \times 5.93 \times 10^{-8} = 243 \text{ p}$$

$$MS = \frac{243}{140 \times 1.25} - 1 = +0.39$$

6.2.5 Analysis of Exit Nozzle Attach Joint

6.2.5.1 General Discussion

The exit nozzle is cantilevered from the airframe at the aft face of the reactor where it is attached by a quick disconnecting joint consisting of an airframe ring, an exit nozzle ring, a locking ring, and a split ring retainer (see Figure 107).

The exit nozzle and locking ring are threaded. The retainer ring is fastened to the locking ring with 3/8-inch diameter steel bolts.

The attach ring is subjected to (1) the axial or load and inertia forces acting on the exit nozzle assembly, (2) a differential air pressure between its circumferential surfaces, and (3) the transverse shear and bending moments of the inertial loads on the nozzle.

6.2.5.1.1 Design Loads

Critical pressures and temperatures occur during low altitude cruise operation at Mach 3.22 and 1000 feet on an ICAO Standard Day.

The net axial drag force acting on the nozzle assembly, nozzle and liner, at the attach joint is 367,438 pounds (limit).

The nozzle assembly weight is 1054 pounds, and the inertial load factors are $g_x = 0$, $g_z = 8.5$, $g_y = 1.0$.

The airframe annulus pressure is 60.22 psia, and the exit-liner annulus pressure is 320 psia at Station 565.47 and 305 psia at the joint of center line station. For analysis, a uniform differential of 250 psi and net axial load of 375,000 pounds are conservatively assumed.

6.2.5.1.2 Temperatures

A uniform temperature of 1400°F is assumed for the entire attach structure with no variation throughout the length or radial thickness.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASO-TDR-63-277, Vol. IV

REPORT 6003

6.2.5.1.3 Material

The material is Rene' 41 fabrication, heat-treated at 1975°F for 1/2 hour, water quenched, aged at 1650°F for 4 hours and air cooled. The rings are machined from forged rings.

For 1400°F, the material properties are as follows:

$$F_{t_u} = 130,000 \text{ psi}$$

$$F_{t_y} = 100,000 \text{ psi}$$

$$E = 24 \times 10^6$$

$$0.2\% \text{ plastic creep in 10 hrs} = 68,000 \text{ psi}$$

$$1\% \text{ creep, 10 hrs} = 85,000 \text{ psi}$$

6.2.5.2 Analysis of Vehicle Coupling

The open end cylinder with its integral flange is subjected to a uniformly distributed axial load, applied in bearing to the flange, and to a uniform internal (bursting) pressure. The assumed coupling geometry is illustrated in Figure 119. A method for analyzing such a structure is presented in Reference 5, Case 16, Table XIII, page 263, and is applied in the calculations which follow:

$$V_o = \text{transverse shear normal to wall, lb linear in.}$$

$$M_o = \text{bending moment, uniform along circumference, in.-lb/linear in.}$$

$$p = \text{unit pressure, psi (250 psi)}$$

$$t = \text{wall thickness, in. (0.375 in.)}$$

$$a = \text{median diameter of vessel, in. (58.25 in.)}$$

$$b = \text{median diameter of flange, in. (59.75 in.)}$$

$$h = \text{flange thickness, in. (0.75 in.)}$$

$$P = \text{tensile force, lb. (375,000 lb.)}$$

$$f = \sqrt{at}, (4.688 \text{ in.})$$

$$d = \text{flange outer diameter, in. (59.50 in.)}$$

$$s_1 = \text{meridional membrane stress}$$

$$s_2 = \text{meridional bending stress}$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

~~ATOMIC ENERGY ACT OF 1954~~

6003

s_2 = hoop membrane stress

s_2^1 = hoop bending stress

$$T_1 = \frac{t^3 (3a^2 + 5d^2)}{h^3 (d^2 - a^2)} = 33.864 \text{ lb/in.}$$

$$T_2 = \frac{3.58 t^3}{h^3 (d^2 - a^2)} \frac{d}{3} \log e \frac{b}{a} + 0.1 (b^2 - a^2) = 0.1033 \text{ /in.}$$

$$V_o = \frac{(f^2 - \frac{h^3}{2t} t_1) (t + 0.2325 \text{ ft}) p - 2T_2 (h + 0.5377 f) P}{1.86 \text{ ft} + T, L^2 (2 + 0.116 \frac{f}{t} t_1) + 1.6103 fh + 0.866 f^2} = -12.31 \text{ lb/in.}$$

$$M_o = \frac{(h^2 T_1 + 1.86 ft) V_o + ht_2 P - 0.5 tp (f^2 - \frac{h^3}{2t} t_1)}{1.5 T h - 3.464 t} = 710.93 \text{ --lb/in.}$$

The maximum axial stress in the cylinder is:

$$S (\text{cylinder}) = \frac{6M_o}{t^2} + \frac{P}{\pi at} = 84,500 \text{ psi}$$

$$MS (\text{cylinder}) = \frac{100,000}{1.15 \times 84,500} - 1 = +0.03$$

The maximum radial stress in the flange is:

$$S_R (\text{flange}) = \frac{6}{h^2} (M_o - 1/2 V_{oh}) + \frac{V_o}{h} + p = 816 \text{ psi}$$

The maximum tangential stress in the flange is

$$S_t (\text{flange}) = \frac{6}{h^2} (M_o - \frac{1}{2} V_{oh}) + \frac{0.8}{h^2 (d^2 - a^2)} d^2 (-15 M + 7.5 h V_o + 1. \dots \geq P \log e$$

$$\frac{b}{a} + 0.447 tp (b^2 - a^2) + \frac{h^2}{4t} T_1 (V_o + hp) = 11,840 \text{ psi}$$

$$MS (\text{flange}) = \frac{100,000}{1.15 \times 11,840} - 1 = \text{High}$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 4571

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.5.3 Retaining Ring and Bolts

The retaining ring is segmented and is bolted to the lock ring by 140 3/8-inch stud bolts. Retaining ring geometry is shown in Figure 120.

Total nozzle assembly axial load = 375,000 lbs

Load/bolt = $375,000/140 = 2680$ lbs/lmt.

6.2.5.3.1 Bolt Shear

$$A = 0.7854 (0.375)^2 = 0.1102 \text{ in.}^2$$

$$f_s = 2680/0.1102 = 24,300 \text{ psi}$$

$$F_{su} = 0.6 \times 130,000 = 78,000 \text{ psi}$$

$$MS (\text{shear}) = \frac{7800}{24,300 \times 1.25} - 1 = + 1.56$$

6.2.5.3.2 Bolt Bearing on Retainer

$$A = 0.15 \times 0.375 = 0.0563 \text{ in.}^2$$

$$f_{br} = 2680/0.0563 = 47,600 \text{ psi}$$

$$F_{bru} = F_{tu} (\text{conservative}) = 130,000 \text{ psi}$$

$$MS = \frac{130,000}{47,600 \times 1.25} - 1 = + 1.18$$

6.2.5.3.3 Bolt Coupling Loading

The axial load transfers from the lock bolt, and thence through the retainer ring to the vehicle fitting shell (Figure 121). The moment occasioned by load transfer in the retainer ring is assumed to be reacted by a couple acting between the bolt center line and the bearing of the retainer on the lock ring.

Moment Arm = 0.25 inches

Load/bolt = 2680 lbs; moment = $2680 \times 0.25 = 670$ lbs;

$$\text{Couple} = \frac{670}{0.50} = 1340 \text{ in.-lbs}$$

Bolt inner dia. = 0.3179; $A = 0.079 \text{ in.}^2$

$$F_t = 1340/0.079 = 17,000 \text{ psi}$$

$$MS = \frac{100,000}{17,000 \times 1.15} - 1 = \text{High}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.5.3.4 Bending of Retainer Ring

O.D. of lock ring = 60.9 in.; circumference = 193.5 in.;
140 bolts are specified.

Perimeter of retainer ring at mid height = 36 in.;

$$\frac{186}{140} = 1.33 \text{ in./bolt.}$$

Assuming a cross sectional area 1.01 inches wide to act
as a beam in simple bending,

$$f = \frac{6M}{bd^2} = \frac{6 \times 2680 \times 0.251}{1.0 \times (0.4)^2} = 25,300 \text{ psi}$$

$$MS = \frac{100,000}{25,300 \times 1.15} - 1 = + 2.44$$

6.2.5.3.5 Ring Shear-Out

$$\text{Area} = 0.31 \times 0.4/2 = 0.124 \text{ in.}^2$$

$$P/A = 2680/0.124 = 21,620 \text{ psi}$$

$$F_{su} = 78,000 \text{ psi}$$

$$MS = \frac{78,000}{21,620 \times 1.25} - 1 = + 1.9$$

6.2.5.4 Analysis of Lock Ring (Figure 122)

Assume each thread reacts one half of the 37,000-pound
limit axial load. The load per inch of thread at the pitch diameter is:

$$P = \frac{37,000}{77 \times 59.6475 \times 2} = 1000 \text{ lb/in.}$$

At Section A-A, assuming a 1.0-inch circumferential width
of shell and thread acting as a beam,

$$M = 1000 \times 0.4375 = 437.5 \text{ in.-lb}$$

$$I = \frac{1 \times (0.415)^3}{12} = 0.00595 \text{ in.}^4$$

$$f_b = \frac{437.5 \times 0.2075}{0.00595} = 15,250 \text{ psi}$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

$$\text{Axial stress} = \frac{2 \times 1000}{1 \times 0.415} = 4820 \text{ psi}$$

$$\text{Total stress} = 15,250 + 4820 = 20,070 \text{ psi}$$

$$\text{MS (yield)} = \frac{100,000}{20,070 \times 1.15} - 1 = + 3.33$$

$$\text{MS (10-hr creep of 1\%)} = \frac{85,000}{20,070 \times 1.15} - 1 = 2.67$$

6.2.5.4.1 Thread Shear at Pitch Diameter

$$\text{Area} = 0.50 \text{ in.}^2$$

$$F_{Bu} = 0.6 F_{tu} = 78,000 \text{ psi}$$

$$P = 1000 \text{ lbs; } F_s = \frac{1000}{0.5} = 2000 \text{ psi}$$

$$\text{MS} = \frac{78,000}{1.25 \times 2000} - 1 = \text{High}$$

6.2.5.4.2 Analysis of Lock Ring as a Cylinder

Due to the high margins obtained in a similar analysis of the airframe portion of the attach assembly and the margins obtained in the preceding analyses of the lock ring, no further investigation is deemed necessary.

6.2.5.4.3 Bending on Lock Ring--Section B-B

Assuming 1/140 x circumference-wide strip as a beam,

$$M = 0.54 \times 2680 = 1450 \text{ in.-lbs}$$

$$I = \frac{0.970 \times (0.55)^2}{12} = 0.0135 \text{ in.}^4$$

$$f = \frac{1450 \times 0.275}{0.0135} = 29,600 \text{ psi}$$

$$\text{Axial stress} = \frac{2680}{0.97 \times 0.55} = 5020 \text{ psi}$$

$$f_b + f_t = 29,600 + 5020 = 34,620 \text{ psi}$$

$$\text{MS (Yield)} = \frac{100,000}{34,620 \times 1.15} - 1 = + 1.5$$

$$\text{MS (10-hr. creep of 1.0\%)} = \frac{85,000}{34,620 \times 1.15} - 1 = + 1.14$$

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

6.2.5.4.4 Analysis--Exit Nozzle Attach Ring

A section of this ring is shown in Figure 3.

6.2.5.4.5 Stress at Section A-A

Load per inch on each thread is:

$$P = \frac{375,000}{(3.14) (59.647) (2)} = 995 \text{ lbs/in.}$$

$$M = (0.458) (995) = 450 \text{ in.-lbs/in.}$$

$$I = \frac{1.0 (0.437)^3}{12} = 0.00697 \text{ in.}^4 \text{ (assuming 1.0-in. wide section)}$$

$$f(\text{total}) = \frac{(450) (0.2185)}{0.00697} + \frac{375,000}{(3.14) (58.77) 0.437} = 1,720 \text{ psi}$$

$$MS = \frac{100,000}{1.15 \times 18,720} - 1 = \text{High}$$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

7.0 MODELS

7.1 Direct-Connect Aerodynamic Coupling

7.1.1 Design

The direct-connect aerodynamic coupling hardware for this contract period is a modification of the test hardware used in the 1961 aerodynamic coupling tests (see Figures 124-128).

To evaluate simulated reactor airflow distribution, a control rod and actuator assembly and front support assembly were designed to simulate geometrically the flight engine hardware and to create as nearly as possible the blockage and airflow distribution anticipated in a flight engine. A transition plate and exit plate were designed with approximately 50 total and static pressure pickups for use at the aft reactor face during testing. Test results are reported in Volume II.

7.2 One-Third Scale Free Jet Model

7.2.1 Design

The 1/3 scale free jet model shown in Figure 129 utilizes the basic hardware from the direct-connect aerodynamic coupling test hardware; i.e., the reactor section, exit nozzle, exit nozzle spacers, and instrumentation sections.

Figures 130 to 142 show engine components in various stages of fabrication. The new hardware that was designed and fabricated included a free jet inlet with translating centerbody spike and by-pass doors, subsonic diffuser section and translating exit nozzle plug. The geometric lines of the inlet nozzle and centerbody are reduced from the flight engine by a scaling factor of 0.3625, which produces an inlet cowl diameter of 4 inches. The inlet spike is translated by means of a hydraulic actuator that is mounted inside the centerbody structure. This actuator operates on 1000 psi supply pressure and is capable of translating the spike 2.5 inches at the rate of 0.25 inches per second working against a load of 3000 pounds.

Two by-pass doors are located on the external wall of the inlet structure near the aft end of the centerbody, one door on each side of the horizontal centerline. Each door has an open area of 18.8 square inches. A hydraulic actuator for each door is mounted externally and is capable of positioning the doors from full open to full closed within two seconds.

The exit nozzle plug is positioned at the center of the exit nozzle throat and is contoured for a linear area variation during translation. The plug is translated by means of a hydraulic actuator, which is capable of translating the plug 18 inches against a 2500-pound load at the rate of one inch per second.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

The reactor section was modified to allow for thermal expansion of the reactor core when tested with 450°F inlet air. All thermal expansion was contained by use of a spring-loaded ring mounted at the forward face of the reactor. Radial thermal expansion was contained by use of spring-loaded straps around the periphery of the core matrix. All the basic components for this engine were fabricated from mild steel and 4130 steel. Test results are reported in Volume II.

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

8.0 REFERENCES

1. "Letter Contract between Department of the Air Force and The Marquardt Corporation", Contract AF 33(657)-8123, 16 April 1962, ~~SECRET RESTRICTED DATA~~, Reg. 10,201.
2. "Performance Bulletin No. 5 for the Marquardt Nuclear Ramjet Engine", The Marquardt Corporation, 1 June 1962, Marquardt 5893, ~~SECRET RESTRICTED DATA~~, Reg. 22,593.
3. "Aerothermodynamics for Pluto", Part II, Volume 5, "Structural Design and Materials", Ling-Temco-Vought, April 1962, ASD-TDR-62-395, ~~SECRET RESTRICTED DATA~~, Reg. 22,718.
4. "Preliminary Analysis of Lateral Vibrations of the Pluto 1 motor", The Marquardt Corporation, 15 December 1961, Marquardt Report 373, ~~SECRET RESTRICTED DATA~~, Reg. 22,350.
5. "Formulas for Stress and Strain", R. J. Roark, 1954, McGraw-Hill, New York.
6. "Analysis and Design of Aircraft Structures", E. F. Bruhn, 1949, Tri-State Offset Co., Cincinnati.
7. "A Second-Order Shock-Expansion Method Applicable to Bodies of Revolution Near Zero Lift", C. A. Syvertson and T. H. Dennis, National Advisory Committee for Aeronautics, 1957, NACA Technical Report 1328.
8. "Computation Curves for Compressible Fluid Problems", C. I. Dailey and F. C. Wood, 1949, John Wiley & Sons, New York.
9. "Statics of Circular-Ring Stiffeners for Monocoque Fuselages", W. Stieda, National Advisory Committee for Aeronautics, February 1942, NACA Technical Memorandum 1004.
10. "Loads Imposed upon Intermediate Frames of Stiffened Shells", Paul Kuhn, National Aeronautics and Space Administration, NASA Technical Note 687.
11. "Strength of Metal Aircraft Elements", Air Force, Navy, Commerce Departments, March 1955, ANC-5 Bulletin.
12. "Mechanical Vibrations", Den Hartog, 1947, McGraw-Hill, New York.
13. "Tory IIC Data Book", 16 October 1962, University of California, Lawrence Radiation Laboratory, ~~SECRET RESTRICTED DATA~~, Reg. 22,107.
14. "Elements of the Differential and Integral Calculus", Granville, Smith, and Longley, Ginn & Co.
15. "Nuclear Ramjet Propulsion System Research and Development for 1962", The Marquardt Corporation, 5 January 1962, Marquardt Proposal 3067A (Vol. I), ~~SECRET RESTRICTED DATA~~, Reg. 22,400.

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

16. "Annual Report for 1961, Nuclear Ramjet Propulsion System, Final Report," The Marquardt Corporation, 30 January 1962, Marquardt Report 876, ~~SECRET RESTRICTED DATA~~, Reg. 22,395.
17. "The Uniform-Section Disk Spring", American Society of Mechanical Engineers 1936. ASME, New York, Vol. 58, No. 6.
18. "Handbook of Mechanical Spring Design", Associated Spring Corporation, Bristol, Conn., 1951.
19. "Test Report, Reactor Side Support System", The Marquardt Corporation, 1 June 1962, Marquardt Report S-276, ~~SECRET RESTRICTED DATA~~, Reg. 22,625.
20. "Engine-Airframe Lateral Attachment Test (Phase II)", The Marquardt Corporation, 31 January 1962, Marquardt Report 5960, ~~CONFIDENTIAL RESTRICTED DATA~~, Reg. 23,066.
21. "Rules for Construction of Unfired Pressure Vessels", American Society of Mechanical Engineers, 1959, ASME Boiler and Pressure Vessel Code Section VIII.
22. "A Study of the Collapsing Pressures of Thin-Walled Cylinder", R. Sturm, Illinois Institute of Technology, Bulletin 329.
23. "The Ultimate Strength of Aluminum-Alloy Formed Structural Shapes in Compression", Robert A. Needham, Journal of Aeronautical Sciences, 4-54, Vol. 21, No. 4, p. 217.
24. "A Simplified Method of Elastic-Stability Analysis for Thin Cylindrical Shells", S. B. Batdorf, National Advisory Committee for Aeronautics, 1947, NACA Technical Report 874.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

TABLE I

TABLE OF WEIGHTS AND CENTERS OF GRAVITY FOR MA50-XCB PROPULSION SYSTEM

Item	Weight (lb)	Eng: e Station C.G.
MA50-XCB Propulsion System	16,456	477
Inlet	2,717	170
Diffuser Duct*	919	405
Reactor Control Rod Support Mechanism	297	441
Reactor and Support System	11,468	333
Exhaust Nozzle	1,054	394

* To be furnished by manufacturer

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

FORM 6003

TABLE II
 MARGINS OF SAFETY

Item	Type of Stress	Margin
<u>Cowl</u>		
Inside skin	Tension	1.75
Outside skin	Tension	4.06
Inside skin	Compression	3.82
Leading edge	Compression	1.67
Skin (near leading edge)	Bending	0.56
<u>Diffuser Skin</u>		
Long time "limit" load	Hoop tension	0.03
Short time "ultimate" load	Hoop tension	0.26
<u>Innerbody Support Ring</u>		
Long time "limit" load	Compression	ample
Long time "ultimate" load	Compression	ample
Short time "limit" load	Compression	0.85
Short time "ultimate" load	Compression	0.62
<u>Innerbody Skin</u>		
Short time "limit" load	Compression	0.30
Short time "limit" load	Shear	ample
Long time "ultimate" load	Compression	0.05
<u>Thrust Fitting (Pin)</u>		
Short time "limit" load	Bending	0.36
Long time "limit" load	Bending	0.68
Short time "ultimate" load	Bending	0.74
<u>Reactor Support</u>		
Pressure shell	Hoop tension	1.5gh
Shell web	Bending	0.35
Pressure pad	Bending	0.11
Pressure pad lug	Bending	1.5gh

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASO-TDR-63-277, Vol. IV

REPORT 6003

TABLE II (Continued)

Item	Type of Stress	Margin
<u>Exit Nozzle</u>		
Forward cylinder	Hoop tension	Minimum = + 0.05
Forward cylinder	Combined bending and axial	+ 2.33
Forward cylinder	Shear	High
Throat	Combined bending and axial	+ 1.32
Forward cone	Combined bending and axial	+ 1.82
Cone-Cylinder Intersection	Combined bending and axial	+ 0.38
<u>Aft Cone</u>		
Station 636.32	Collapsing	+ 0.085
Station 623.41 to 636.32	Collapsing	+ 0.61
	Bending and Axial	High
	Shear	High
Aft Doubler	Collapsing	High
<u>Shroud</u>		
Forward cylinder	Collapsing	+ 0.70
Station 15	Collapsing	+ 0.16
Cone	Collapsing	+ 0.33
Throat	Collapsing	+ 0.39

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASP-TDR-63-277, Vol. IV

OUT 6003

TABLE III
INERTIAL LOAD FACTORS

Parameter	Launch Conditions		Low Level Maximum Cruise		War Head Ejection	High Altitude Cruise
	At Burnout	At Separation				
Altitude	12,500	21,500	1000	1000	10,000	30,000
Mach Number	2.25	2.95	2.90	3.0	2.9	3.5
Day Condition	Hot	Hot	Hot	Cold	Hot	Cold
g_x	+ 6.0 - 0	+ 0 - 0.32	+ 0 - 0.3	+ 0 - 0.3	+ 0 - 0.3	+ 0.10 -
g_y	± 1.0	± 1.0	± 1.0	± 1.0	± 0.25	± 1.0
g_z	0	0	+ 4.25 - 2.25	+ 4.25 - 2.25	+ 2.8 - 0.8	+ 1.5 - 0
w_y	± 0.12	0	± 0.50	± 0.50	± 0.50	± 0.50
w_z	± 0.25	± 0.25	± 0.50	± 0.25	0	± 0.50

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE IV

INERTIA LOAD FACTORS FLIGHT MANEUVER CONDITIONS

(Translational Accelerations (g -1/2) at Centers of Gravity of Propulsion System Items)

Flight Phase	Inlet			Duct			Reactor Controls			Reactor			Exit		
	ϵ_x	ϵ_y	ϵ_z	ϵ_x	ϵ_y	ϵ_z	ϵ_x	ϵ_y	ϵ_z	ϵ_x	ϵ_y	ϵ_z	ϵ_x	ϵ_y	ϵ_z
Boost at	+6.015	+1.144	+0	+6.0	+1.0	+0.005	+6.0	+1.038	+0.031	+6.0	+1.09	+0.042	+6.0	+1.06	+0.103
Burnout	-0	-0.856	-0.0672	-0	-1.0	-0.005	-0	-1.038	-0.031	-0	-1.09	-0	-0	-1.06	-0.103
Rocket	+0	+1.091	+0	+0	+1.06	+0	+0	+1.09	+0	+0	+1.14	+0	+0	+1.18	+0
Rejection	-0.32	-1.091	-0	-0.32	-1.06	-0	-0.32	-1.09	-0	-0.32	-1.14	-0	-0.32	-1.18	-0
High Altitude Cruise	+0.164	+1.182	+1.690	+0.10	+1.12	+1.62	+0.10	+1.17	+1.67	+0.10	+1.28	+1.78	+0.10	+1.37	+1.87
Low Altitude Cruise	-0.164	-1.182	-0.190	-0.10	-1.12	-0.12	-0.10	-1.17	-0.17	-0.10	-1.28	-0.28	-0.10	-1.37	-0.37
Weapons	+0.364	+1.182	+4.44	+0.3	+1.12	+4.37	+0.3	+1.17	+4.44	+0.3	+1.5	+4.53	+0.3	+1.37	+4.62
Ejection	-0.064	-1.182	-2.44	-0	-1.12	-2.37	-0	-1.17	-2.44	-0	-1.5	-2.53	-0	-1.37	-2.62
	+0.364	+0.25	+5.50	+0.3	+0.25	+4.50	+0.3	+0.25	+5.0	+0.3	+1.5	*	+0.3	+0.25	+8.50
	-0.064	-0.25	-0.99	-0.0	-0.25	-0.92	-0.0	-0.25	-0.97	-0	-1.5	-1.08	-0	-0.25	-1.17

* +5.77 g at reactor forward end, Fuselage Station 882.4

+6.68 g at reactor aft end, Fuselage Station 938.5

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

#CMT 6003

TABLE V
VERTICAL LOAD FACTORS

Parameters	Low Level Maximum Maneuver	Low Level Warhead Ejection
Altitude	1000	1000
Mach Number	2.9	2.9
Day Conditions	Hot	Hot
g_x	+ 0.3	+ 0.3
g_y	± 1.5	± 1.5
g_z	+ 4.5 - 2.5	*

* Values of g_z for weapons ejection at the reactor ends are as follows:

<u>Forward End</u>	<u>Aft End</u>
F.S. 882.4	F.S. 938.5
$g_z = + 5.77$	$g_z = + 6.68$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE VI

GROUND HANDLING LOAD FACTORS

Parameter	Hoist	Transport	Erection
E_x	± 0.25	± 5.0	± 1.0
E_y	± 0.25	± 0.05	± 0
E_z	$+ 2.67$ $- 0$	$+ 2.0$ $- 1.0$	$+ 2.0$ $- 1.0$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

Form 6003

TABLE VII
ANALYTICAL DESIGN FACTORS AND FACTORS OF SAFETY

A. For temperature regimes in which time and material creep are negligible factors			
Type of Limit Stress	Material Mechanical Strength Property	Multipl Design	ig Factor Safety
Single (uniaxial) (tension, compression, etc.)	Yield or proportional limit	1.10	--
Combined stresses biaxial, or triaxial	Yield or proportional limit	1.15	--
Single or combined	Short time ultimate	--	1.25
B. For temperature regimes in which time is a major factor			
Single	Uniaxial steady state creep	1.10	--
Combined	Uniaxial steady state creep for type of stress involved	1.15	--
Single or combined	Creep rupture for type of stress involved	--	1.25
Single or combined	Fatigue for type of stress and load-temperature pattern involved	--	1.25

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE VIII
WEIGHTS AND CENTERS OF GRAVITY FOR AXISYMMETRIC LET

	Weight (lbs)	Center of Engine St	avity ion
Inlet Assembly	2717.1	169.9	
Nose Cone (Movable parts)	145.9	88.7	
Innerbody - Rings	132.8	130.6	
Innerbody - Skins	219.4	137.0	
Outer Structure - Rings	333.7	192.1	
Outer Structure - Skins	920.0	194.8	
Longerons	251.1	146.4	
Attach Struts	369.2	221.0	
Actuating Mechanism	345.0	111.6	

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF

6003

TABLE IX
SECTION PROPERTIES AND LOCATIONS FOR CALCULATION OF STRESS

Item	Thickness h (av)	w	Area h · w	d from * ref line	Ad	$\bar{d}$	$\bar{A}d$	$\bar{A}d^2$	I_o
a	0.228	3.28	0.74900	0.700	0.52400	+0.147	+0.10980	0.016120	0.000582
b	0.400	1.762	0.70500	0.480	0.33800	-0.073	-0.05150	0.003760	0.009400
c	0.600	1.920	1.15300	0.370	0.42700	-0.183	-0.21100	0.038600	0.020000 approx.
d	0.062	0.90	0.05570	0.800	0.04450	+0.247	+0.01375	0.003400	--
e	0.062	0.90	0.05570	0.031	0.00173	-0.522	-0.02905	0.015190	--
f	0.062	2.30	0.14250	1.250	0.17800	+0.697	+0.09940	0.069400	--
g	0.062	2.30	0.14250	0.031	0.00441	-0.522	-0.07450	0.038900	--
h	0.090	1.26	0.11320	1.120	0.12720	+0.567	+0.06430	0.036500	--
j	0.090	1.26	0.11320	0.107	0.01212	-0.446	-0.05050	0.022520	--
k	1.000	0.09	0.09000	0.600	0.05400	+0.047	+0.00423	0.000199	0.007500
l	0.062	2.30	0.14280	1.620	0.23150	+1.067	+0.15220	0.162500	--
m	0.090	1.26	0.11320	1.520	0.17200	+0.967	+0.10940	0.105800	--
n	0.062	2.30	0.14280	0.031	0.00442	-0.522	-0.07460	0.039000	--
o	0.090	1.26	0.11320	0.107	0.01212	-0.446	-0.05050	0.022550	--
p	1.200	0.09	0.10800	0.800	0.08650	+0.247	+0.02670	0.006600	0.012930
			3.93980	0.553	2.21750		+0.57958 -0.54165	0.581039	0.050412
							+0.03793	0.631451 in. ⁴	

* Reference line is at 20.188 inch radius.
n to N.A. = 20.100-0.223 = 19.877 inch

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE X
 MOMENTS FROM AIR LOAD

1	2	3	4	5	6
x Location (degrees)	Moment from Loading I	Moment from Loading II	Combined Moment of I and II	sin	+20 sin x-M
Centerline					
0	+ 13250	- 26500	- 13250	0	0
10	+ 12650	- 25400	- 12750	0.17	- 44500
20	+ 10500	- 21000	- 10500	0.34	- 72000
30	+ 7350	- 14650	- 7300	0.50	- 73000
40	+ 2820	- 5610	- 2790	0.64	- 35900
50	- 1910	+ 3810	+ 1900	0.76	+ 29100
60	- 6650	+ 13320	+ 6580	0.86	+114000
70	- 10850	+ 21600	+ 10750	0.94	+202000
80	- 13640	+ 27200	+ 13560	0.98	+276000
Side					
90	- 14500	+ 29000	+ 14500	1.00	+290000
100	- 13640	+ 27200	+ 13560	0.98	+276000
110	- 10850	+ 21600	+ 10750	0.94	+202000
120	- 6650	+ 13320	+ 6580	0.86	+114000
130	- 1910	+ 3810	+ 1900	0.76	+ 29100
140	- 2820	- 5610	- 2790	0.64	- 35900
150	+ 7350	- 14650	- 7300	0.50	- 73000
160	+ 10500	- 21000	- 10500	0.34	- 72000
170	+ 12650	- 25400	- 12750	0.17	- 44500
Centerline					
180	+ 13250	- 26500	- 13250	0	0
					Σ 1,900,000 Units

Notes:

Columns 2 and 3 derived from Reference 5 (Case 27 data with = 77/2).

Column 2 based on + 492 lbs/in. maximum (inward)

Column 3 based on - 984 lbs/in. maximum (outward)

(+) moments place compression on outside of ring.

Columns 2 and 3 integrate to transverse loads of 30,900 lbs, 5,450 lbs or a total transverse load of 46,350. Radial deflection, Δ , of point at x = 0 relative to a line joining points at x = 90° and x = 270° is:

$$\Delta = \frac{1,900,000}{EI} = \frac{1,900,000}{22.7 \times 10^6 \times 0.631} = 0.132 \text{ in.}$$

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

NY 6003

TABLE XI

TORQUE LOADS AT VARIOUS SECTIONS

Section	Distance from Centerline to fitting y	y^2	Ratio $y^2/1534$	Torque (in.-lbs)	Σ Torq	
A-A	22.5	506	0.330	370,000	370	00
B-B	21.76	472	0.308	345,000	715	00
C-C	18.24	333	0.217	245,000	958	00
D-D	12.40	154	0.100	112,000	1,070	00
E-E	8.30	69	0.045	50,000	1,120	00
		1534	1.000	1,120,000	See N	e

Note: These values utilized for plot (Figure 27) of torques
be sustained by inlet itself.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XII

SHEARS, MOMENTS, AND TORQUES FROM Z DIRECTION COMPONENTS

1	2	3	4		
Location	x to Section A-A	x' from Centroid to Section	(x') ²	Load at	Joint
A-A	0	+ 77.30	5,980	22,	0
B-B	- 26.90	+ 50.40	2,540	14,	0
C-C	- 53.50	+ 23.80	566	6,	0
D-D	- 95.50	- 18.20	331	- 5,	0
E-E	-127.86	- 50.56	2,550	- 14,	0
F-F	-160.46	- 83.16	6,900	- 23,	0
	-464.22	(+) is aft centroid	18,867	(+) t	s.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XIII

PRESSURE CALCULATIONS

Segment Station Location	Large Area (in. ²)	Small Area (in. ²)	Average ΔP (psi)	Forward Load (lb)	Aftward Load (lb)	Description
-37.7 to -15	124	0	35-62 = -27	3,350		pike ithout kirt
-15 to 0	453	124	42-62 = -20	6,580		
0 to 5	610	453	110-62 = +48		7,540	
5 to 8.4	660	610	110-62 = +48		2,400	
8.4 to 18.7 Outer	730	639	110-62 = +48	4,370		pike kirt
8.4 to 18.7 Inner	730	639	62-62 = 0		0	
18.7 to 22.4	639	566	110-62 = +48	3,500		ixed Part of nner Body
22.4 to 39.06	566	453	311-62 = 249	28,100		
39.06 to 71.66	453	226	311-62 = 249	56,500		
71.66 to 89.96	226	0	311-62 = 249	56,300		
0 to 5.0	1210	1150	110-62 = 48	2,880		nner Skin of ouble Wall owling
5 to 13.2	1210	1112	110-62 = 48		4,700	
13.2 to 22.4	1112	1010	110-62 = 48		4,900	
22.4 to 39.06	1010	966	311-62 = 249		10,940	
39.06 to 71.66	966	880	311-62 = 249		21,400	
71.66 to 89.96	880	855	311-62 = 249		6,220	
0 to 5.0	1278	1150	31-62 = -31	3,970		uter Surface of ouble Wall

NOTE: These symmetrical loads are base loads on which the loads 25 to 29 and Tables XI and XII are superposed to obtain an rical load pattern.

Figures
nasymet-

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 0 2 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 1672

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XIV
JOINT LOADS

1	2	3	4	5
Location	Section A-A	x' from Centroid to Section	(x') ²	P at Joint
A-A	0	+77.30	5,980	15,000
B-B	-26.90	+50.40	2,540	9,790
C-C	-53.50	+23.80	566	4,610
D-D	-95.50	-18.20	331	3,530
E-E	-127.86	-50.56	2,550	9,800
F-F	-160.46	-83.16	6,900	16,120
	-464.22	(+) is aft of centroid	18,867	(- tension

$$\frac{\sum x}{6} = \frac{-464.22}{6} = -77.3 \text{ inches}$$

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XV
 ATTACHMENT REACTIONS FROM AIRLOADS FOR THE UNSYMMETRICAL CONDITION

Fitting Point	Description and References	X	Y	Z
a Center	Figure 29 $Z = -31,153$ Figure 31 $Z = -16,120$	0	0	- 47,273
b Left	Figure 29 $Z = 0.5 (-21,853) = - 10,926$ Figure 28 $Z = 50,000 \div 16.6 = + 3010$ Figure 31 $Z = 0.05 (-9800) = - 4900$	0 0	0 0	- 12,816
c Right	Figure 29 $Z = 0.5 (-21,853) = - 10,926$ Figure 28 $Z = -(50,000 \div 16.6) = - 3010$ Figure 31 $Z = 0.5 (-9800) = - 4900$	0	0	- 18,836
d Left	Figure 29 $Z = 0.5 (-12,643) = - 6321$ Figure 28 $Z = 112,000 \div 24.8 = + 4520$ Figure 31 $Z = 0.5 (-3530) = - 1765$	0	0	- 3,566
e Right	Figure 29 $Z = 0.5 (-12,643) = - 6321$ Figure 28 $Z = -(112,000 \div 24.8) = - 4520$ Figure 31 $Z = 0.5 (-3530) = - 1765$	0	0	- 16,606
f Left	Figure 29 $Z = 0.5 (-673) = - 336$ Figure 28 $Z = 243,000 \div 36.48 = + 6670$ Figure 31 $Z = 0.5 (4610) = + 2305$	0	0	+ 8,639
g Right	Figure 29 $Z = 0.5 (-673) = - 336$ Figure 28 $Z = -(243,000 \div 36.48) = - 6670$ Figure 31 $Z = 0.5 (4610) = + 2305$	0	0	- 4,701
h Left	Figure 29 $Z = 0.5 (+6,927) = + 3464$ Figure 28 $Z = 345,000 \div 43.52 = + 7920$ Figure 31 $Z = 0.5 (+9790) = + 4895$	0	0	+ 16,279

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XV (Continued)

Fitting Point	Description and References	X	Y	Z
j Right	Figure 29 Z = $0.5 (+6,927) = +3464$ Figure 28 Z = $-(345,000 \div 43.52) = -7920$ Figure 31 Z = $0.5 (+9790) = +4895$		+	439
k Left	Figure 29 Y = $0.5 (+17,640) = 8820$ (Aft of Section A-A Joint) Figure 29 Z = $0.5 (+14,547) = +7,278$ Figure 31 Z = $0.5 (15,000) = 75,000$ Figure 28 Z = $370,000 \div 45 = +8,220$	0	* + 8,820	+ 22,998
l Right	Figure 27 Y = $0.5 (+17,640) = 8820$ (Aft of Section A-A Joint) Figure 29 Z = $0.5 (+14,547) = +7,278$ Figure 31 Z = $0.5 (15,000) = +7,500$ Figure 28 Z = $370,000 \div 45 = -8,220$	0	* + 8,820	+ 6,558
t Center	Figure 27 Y = -44,400 Figure 31 X = +145,800	** +145,800	-44,400	0

** For a short time, this load could rise to 204,400 lbs (See Paragraph 3.2.5)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XVI
 SYMMETRICAL LOAD PORTION OF STEADY STATE FLIGHT CONDITION RING MOMENTS

θ	Symmetrical Load Portion				MF C1 (in-lbs)
	C1 P at 0°	C1 P at 120°	C1 P at 240°	Total	
0	- 0.24	- 0.025	- 0.025	- 0.190	- 17 00
15	- 0.12	0.065	- 0.015	- 0.070	- 6 10
30	- 0.02	0.090	- 0.050	+ 0.020	
45	0.05	0.100	- 0.072	+ 0.078	
60	0.088	0.088	- 0.075	+ 0.101	+ 9 50
75	0.10	0.050	- 0.072	+ 0.078	
80	0.090	- 0.020	- 0.050	+ 0.020	
105	0.005	- 0.120	- 0.015	- 0.070	
120	0.025	- 0.240	+ 0.025	- 0.190	- 17 00
125	- 0.015	- 0.120	+ 0.065	- 0.070	
160	- 0.050	- 0.020	+ 0.090	+ 0.020	
165	- 0.072	0.050	+ 0.100	+ 0.078	
180	- 0.075	0.088	+ 0.088	+ 0.101	+ 9 50
195	- 0.072	0.100	+ 0.050	+ 0.078	
210	- 0.050	0.090	- 0.020	+ 0.020	
225	- 0.015	0.065	- 0.120	- 0.070	
240	0.025	0.025	- 0.24	- 0.190	- 17 00
255	0.065	- 0.015	- 0.12	- 0.070	
270	0.090	- 0.050	- 0.02	+ 0.020	
285	0.100	- 0.072	0.05	+ 0.078	
300	0.088	- 0.075	0.088	+ 0.101	+ 9 50
315	0.050	- 0.072	0.100	+ 0.078	
330	- 0.020	- 0.050	0.090	+ 0.020	
345	- 0.120	- 0.015	0.065	- 0.070	- 6 10
360	- 0.240	0.025	0.025	- 0.190	- 17 00

Notes:

Coefficient C1 is from Reference 9; (+) moment is compression on outside; PR = $8500 \times 10.79 = 91,600$ (from Table XI)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XVII

STEADY STATE FLIGHT CONDITION RING MOMENTS

ϕ	Unsymmetrical		Symmetrical	mbined
	C_1 P at 0°	$M_p =$ $C_1 PR$ (in-lbs)	M_p (in-lbs)	M_p n-lbs)
0	- 0.24	- 9,710	- 17,400	27,110
15	- 0.12	- 4,860	- 6,410	11,270
30	- 0.02			
45	0.05			
60	0.088	+ 3,560	+ 9,250	12,810
75	0.100			
90	0.090			
105	0.065			
120	0.025	+ 1,012	- 17,400	16,388
135	- 0.015			
150	- 0.050			
165	- 0.072			
180	- 0.075	- 3,040	+ 9,250	6,210
195	- 0.072			
210	- 0.050			
225	- 0.015			
240	0.025	+ 1,012	- 17,400	16,388
255	0.065			
270	0.090			
285	0.100			
300	0.088	+ 3,560	+ 9,250	12,810
315	0.050			
330	- 0.020			
345	- 0.120	- 4,860	- 6,410	11,270
360	- 0.240	- 9,710	- 17,400	27,110

Notes: Coefficient C_1 is from Reference 9; (+) moment is compression on outside. For unsymmetrical portion of moments, PR is $3,760 \times 10.79 = 40,500$ (see Figure 37). Moments for symmetrical portion of moments are carried over from Table XVI.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XVIII

RING MOMENTS FROM SHORT TIME MANEUVER LOADS

ϕ	Symmetrical		Unsymmetrical		Combined
	Total C1	Moment (in-lbs)	Total C1	Moment (in-lbs)	Moment (in-lbs)
0	- 0.190	- 23,960	- 0.24	- 68,100	- 92, 00
15	- 0.070	- 8,810	- 0.12	- 34,050	- 42, 00
30	+ 0.020		- 0.02		
45	+ 0.078		0.05		
60	+ 0.101	+ 12,700	0.088	+ 25,000	+ 37, 00
75	+ 0.078		0.100		
80	+ 0.020		0.090		
105	- 0.070		0.065		
120	- 0.190	- 23,960	0.025	+ 7,100	- 16, 00
125	- 0.070		- 0.015		
160	+ 0.020		- 0.050		
165	+ 0.078		- 0.072		
180	+ 0.101	+ 12,700	- 0.075	- 21,300	- 8, 00
195	+ 0.078		- 0.072		
210	+ 0.020		- 0.050		
225	- 0.070		- 0.015		
240	- 0.190	- 23,960	0.025	+ 7,100	- 16, 00
255	- 0.070		0.065		
270	+ 0.020		0.090		
285	+ 0.078		0.100		
300	+ 0.101	+ 12,700	0.088	+ 25,000	+ 37, 00
315	+ 0.078		0.050		
330	+ 0.020		- 0.020		
345	- 0.070	+ 8,810	- 0.120	- 34,050	- 42, 00
360	- 0.190	- 23,960	- 0.240	- 68,100	- 92, 00

Notes: Data are from Table XVI; (+) moment is compression outside for symmetrical portion PR = 11,698 x 10. 126,000; for unsymmetrical portion PR = 26,320 x 10. 284,000.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

RI DT 6003

TABLE XIX

WEIGHT AND CENTERS OF GRAVITY FOR CONTROL ROD SUPPORT STRUCTURE
AND ACTUATION MECHANISM

Item	Weight	Engine Station C	
Assembly Total	297.7	422.4 Hot* 441.4 Cold	
(a) Front Support	9.7	393.4	
(b) Main Support Assembly	28.0	419.8	
(c) Vernier Rod Strut Assemblies	9.4	419.8	
(d) Motors and Servo Valves	90.0	419.8	
(e) Linear Transducers + Housing	10.0	419.8	
(f) Torque Drive Shaft + Gears	3.2	419.8	
(g) Rack Guide Tubes	24.0	422.6	
(h) Aft Support Structure	11.5	466.6	
(j) Drive Racks	75.0	396.6 Hot 436.6 old	
(k) Spider Fittings	36.9	423.6 Hot 463.6 old	

* "Hot" refers to an operating power reactor condition; "Cold" refers to a zero power reactor condition.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XX

WEIGHTS AND CENTERS OF GRAVITY FOR REACTOR AND SUPPORT SYSTEM

Item	Weight (lbs)	Engine Station C
Reactor and Support System	11,468	533.940
Core and Reflector	7,569	537.046
Tie Rods	402	531.203
Retainer Assembly	228	509.563
Base Blocks	520	565.408
Grid	900	501.186
Lateral Support Structure	1,593	536.870
Axial Support Structure	256	501.186

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XXI

THERMAL EXPANSION OF TANGENTIALLY AND RADIALY ORIENTED RINGS

Expansion (radial)
 R (Core) = 23.625 in.
 R (Reflector) = 26.625 in.
 R (Pressure Shell) = 28.125 in

Reactor Component	Material	End of Boost (60 sec)			Steady Stat		Cruise (300 sec)	
		Temp. °F	Coef/Exp.	ΔR	Temp. °F	Co	/Exp	ΔR
Core	Fueled BeO	2570	5.65×10^{-6}	0.3345	2570	5.	$\times 10^{-6}$	0.3345
Zone 1	Unfueled BeO	*		0.0046	1890	5.	$\times 10^{-6}$	0.0046
Zone 2	Unfueled BeO			0.0066	1490	4.	$\times 10^{-6}$	0.0066
Zone 3	Unfueled BeO			0.0091	1390	4.	$\times 10^{-6}$	0.0091
TOTAL				0.3548				0.3548
Tangential System								
Exp. Shell	Rene' 41	360	6.8×10^{-6}	0.0525	1500	8.	$\times 10^{-6}$	0.322
Radial Diff.	Rene' 41	360	6.8×10^{-6}	0.302	1500	8.	$\times 10^{-6}$	0.033
Diam. Diff	Rene' 41	360	6.8×10^{-6}	0.604	1500	8.	$\times 10^{-6}$	0.066
Circ. Diff.	Rene' 41	360	6.8×10^{-6}	1.895	1500	8.	$\times 10^{-6}$	0.207
Radial System								
Pressure Shell	Rene' 41	300	6.8×10^{-6}	0.0439	1200	7.	$\times 10^{-6}$	0.248
Radial Diff.				0.311				0.107

* Reflector expansion assumed the same as cruise.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF ID: A6003

TABLE XXII

SUMMARY OF CRITERIA

(Inertia Load Factors Taken from Section 2.0)

Condition	Spring Temperature (°F)	Relative Thermal Expansion		Inertia Load Factor (g)	Type of Load
		Radial System (in.)	Tangential System (in.)		
Ground Handling	70	0	0	3.75	Static *
Boost	70	0	0	3.00 (RMS)	Random vibration
Boost Transition	380	0.311	0.302	3.00 (RMS) 2.25	Random vibration Static*
High Altitude Cruise	1400	0.107	0.033	2.25 (RMS) 4.16	Random vibration Static*
Low Altitude Cruise	1400	0.107	0.033	2.25 (RMS) 5.95 7.0 (Peak)	Random vibration Static* Sinusoidal vibration at 9 cps

* Conservatively assumed static.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 487

~~SECRET RESTRICTED DATA~~~~ATOMIC ENERGY ACT OF 1954~~

1 out 6003

ASD-TDR-63-277, Vol. IV

TABLE XXIII

TABLE OF DYNAMIC TEST RUNS "NOMINAL CONDITIONS"

Run No.	Temperature (°F)	Type of Test	Frequency Range	"g" Load	Duration of Run (min)	Comments
1	Ambient	Sine wave sweep	5-500	0.5 RMS	10	Check out } A-C noise interference
2	Ambient	Sine wave sweep	5-500	0.75 RMS	10	Check out }
3	Ambient	Sine wave sweep	5-2000	1 RMS	15	Low response
4	Ambient	Sine wave sweep	5-500	2 RMS	11	
5	Ambient	Sine wave sweep	5-500	3 RMS	11	
6	Ambient	Sine wave sweep	500-13	4 RMS	9	Broke shaker head support brace - repaired
7	Ambient	Flat random	20-500	3 RMS overall	10	
8	400-1130	Sine wave sweep	500-5	3 RMS	15	Lost 3 of 8 high temperature accelerometers
9	400-1130	Flat random	20-500	3 RMS overall	10	
10	1100-1380	Sine wave sweep	500-5	3 RMS	10	
11	1100-1380	Flat random	20-500	3 RMS overall	10	

~~SECRET RESTRICTED DATA~~~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

TABLE XXIV

ACCELEROMETER RESPONSE
(Run No. 4)

Input Frequency (cps)	Input gRMS	Response gRMS							
		A-10	A-3	A-4	A-5	A-6	A-7	A-8	A-11
12	0.55*	1.70*	0*	--	1.5*	--	0.3	0.8	0.8
15	0.65*	2.1	0.3*	0.26*	1.85*	--	0.9	1.1	1.8
20	0.75*	2.3	0.85*	0.60*	2.0*	--	1.6	1.1	1.3
25	0.85*	2.30*	0.85*	0.44*	2.0*	--	0.88	1.2	1.3
30	1.0*	2.05*	0.65*	0.52*	1.9*	--	1.0	1.6	1.2
35	1.20*	1.75*	0.40*	--	1.70*	--	1.0	1.2	2.0
40	1.25*	1.65*	0.30*	--	1.70*	--	0.75	1.0	1.2
80	1.5	2.0	1.2	1.5	2.0	1.6	1.0	2.0	2.0
140	1.7	3.0	0	1.8	2.5	2.4	2.0	2.0	2.5
200	2.5	3.2	4.0	2.0	3.5	3.0	--	3.8	2.5
250	2.3	4.0	3.6	2.0	3.0	3.5	--	3.5	--
320	2.2	3.8	1.2	2.0	2.5	2.5	--	2.4	1.0
400	2.2	2.5	0.80	1.0	1.0	0.80	2.5	1.0	1.50
440	1.7	3.0	0.80	0.70	1.2	0.50	1.0	1.0	1.60
500	1.5	2.6	1.8	0.60	1.0	0.70	0.50	2.0	)

* Frequency analyzed.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XXV

ACCELEROMETER RESPONSE
(Run No. 10)

Input Frequency (cps)	Input g _{RMS}	Response g _{RMS}				
		A-10	A-3	A-4	A-5	A-8
12	0.95*	2.2*	--	--	--	
15	1.0*	2.45*	0.5	0.2	0.8	6
20	1.0*	2.65*	--	--	--	
25	1.0*	2.75*	2.0	0.8	0.5	4
30	1.1*	2.70*	2.2	2.0	1.0	8
35	1.2*	2.65*	--	--	--	
40	1.5*	2.50*	2.5	1.0	1.0	0
80	3.0	--	3.0	1.0	1.5	0
140	3.0	--	3.0	2.5	2.5	0
200	3.1	5.5	3.0	2.5	--	7
250	2.9	--	2.5	1.7	--	0
320	2.7	--	3.0	1.0	0.5	5
400	2.0	--	3.0	0.8	1.0	5
440	1.7	--	1.0	1.0	1.0	5
500	2.0	4.5	1.5	1.5	1.0	0

* Frequency analyzed.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XXVI

WEIGHTS AND CENTERS OF GRAVITY FOR EJECTOR EXHAUST NOZZ

Item	Weight (lb)	Engine Stat n
Exhaust Nozzle Assembly	1054	594.2
Primary Shell	370.9	618.9
Secondary Shell	239.5	587.7
Lock Ring	178.4	568.1
Attach Ring	162.1	571.0
Ring Assembly - Expansion	41.0	565.8
Stiffener Ring - Aft	9.1	660.3
Stiffener Ring - Forward	8.4	654.3
Stiffener Ring - Intermediate	7.9	646.3
Stiffener Ring - Intermediate	6.1	636.3
Drag Brace (12 required)	8.7	573.9
Pylon (8 required)	9.0	613.3
Support Doubler (8 required)	11.8	613.9
Fitting - Inner (8 required)	1.1	663.4

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET-RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE XXVII

REQUIRED MATERIAL THICKNESS

Exit Station	Internal Radius (in.)	Nozzle Interior* (psia)	Δp (psi)	t_{reqd} $\frac{(\Delta p)}{78,000}$ (in.)
0	28.06	320	260	0.093
5	28.06	300	240	0.086
10	28.06	292	232	0.083
15	28.06	288	228	0.081
20	28.06	286	226	0.080
22	27.80	284	224	0.079
24	26.70	283	223	0.076
26	25.50	282	222	0.075
28	24.40	280	220	0.068
30	23.20	279	219	0.064
32	22.10	277	217	0.061
34	21.00	275	215	0.057
36	19.90	273	213	0.054
38	18.70	270	210	0.050
40	17.70	266	206	0.046
42	16.90	262	202	0.043
44	16.40	258	198	0.041
46	16.20	254	194	0.040
48	16.10	248	188	0.036
52	16.40	77	17	0.002

* Airframe cooling annulus pressure profile assumed uniform at 60 psia (Figure 110).

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

TABLE XXVIII

SHROUD PRESSURE AND TEMPERATURE DATA

(Low Altitude Cruise at Mach 3.22 and 1,000 ft, ICAO Standard D)

Engine Station	Inner Surface	Outer Surface	Pressure (psia)	Temperature (°F)
0	320	254	66	1350
570.470	300	254	46	1350
575.470	296	254	38	1350
585.470	286	258	28	1355
587.24	284	258	26	1360
590.470	283	256	27	1380
595.470	279	252	27	1420
600.470	274	241	33	1465
605.037	267	216	51	1520
611.289	254	126	128	1550
613.413	249	109	140	1550
615.87	160	90	70	1490
616.969	81	81	0	--

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

1



~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

23

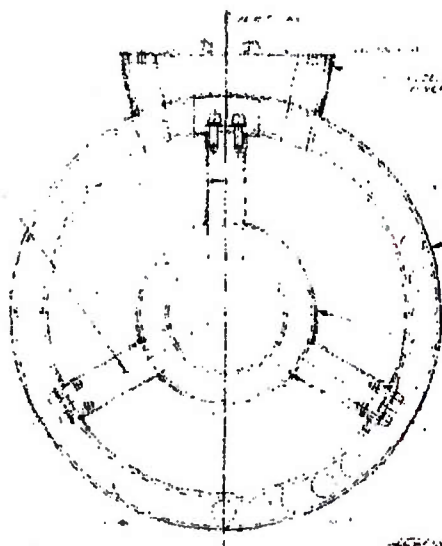
12-12

22

21

2

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

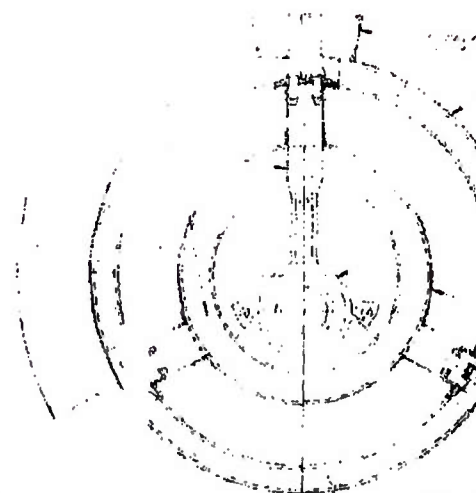


SECTION C-C

X81420

20

11



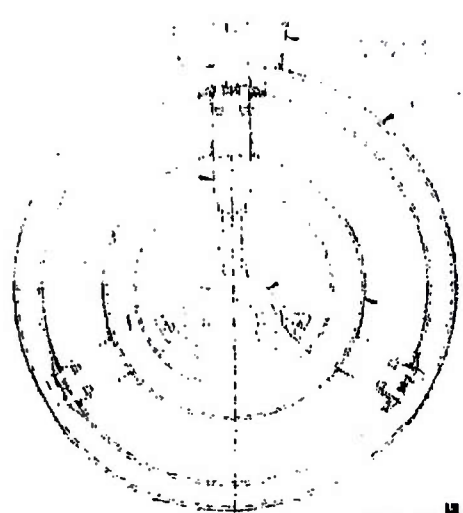
19

3

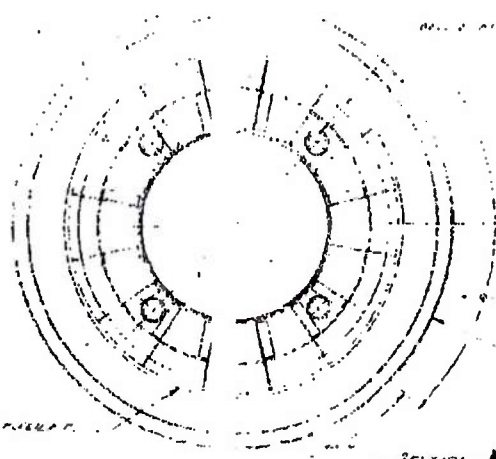
DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WNS
Date: OCT 02 2015



1-12



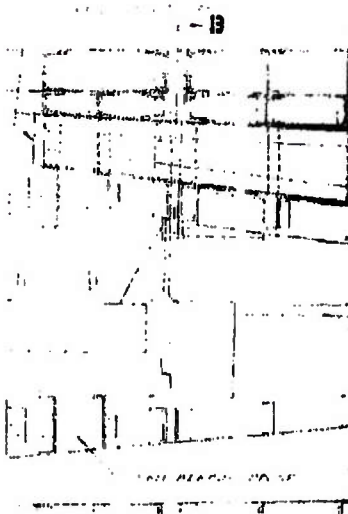
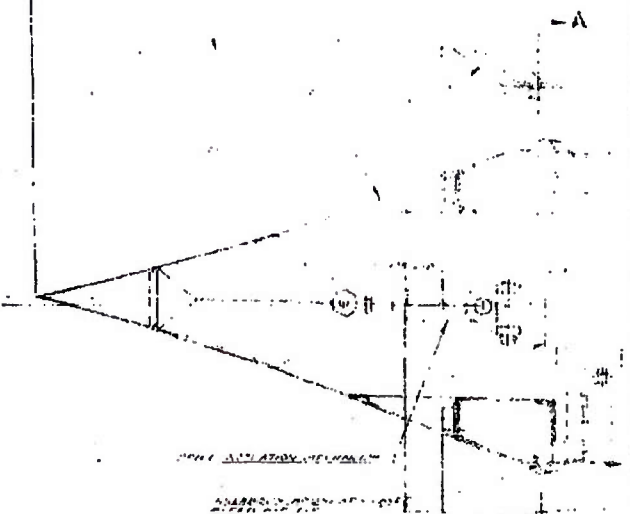
B-B



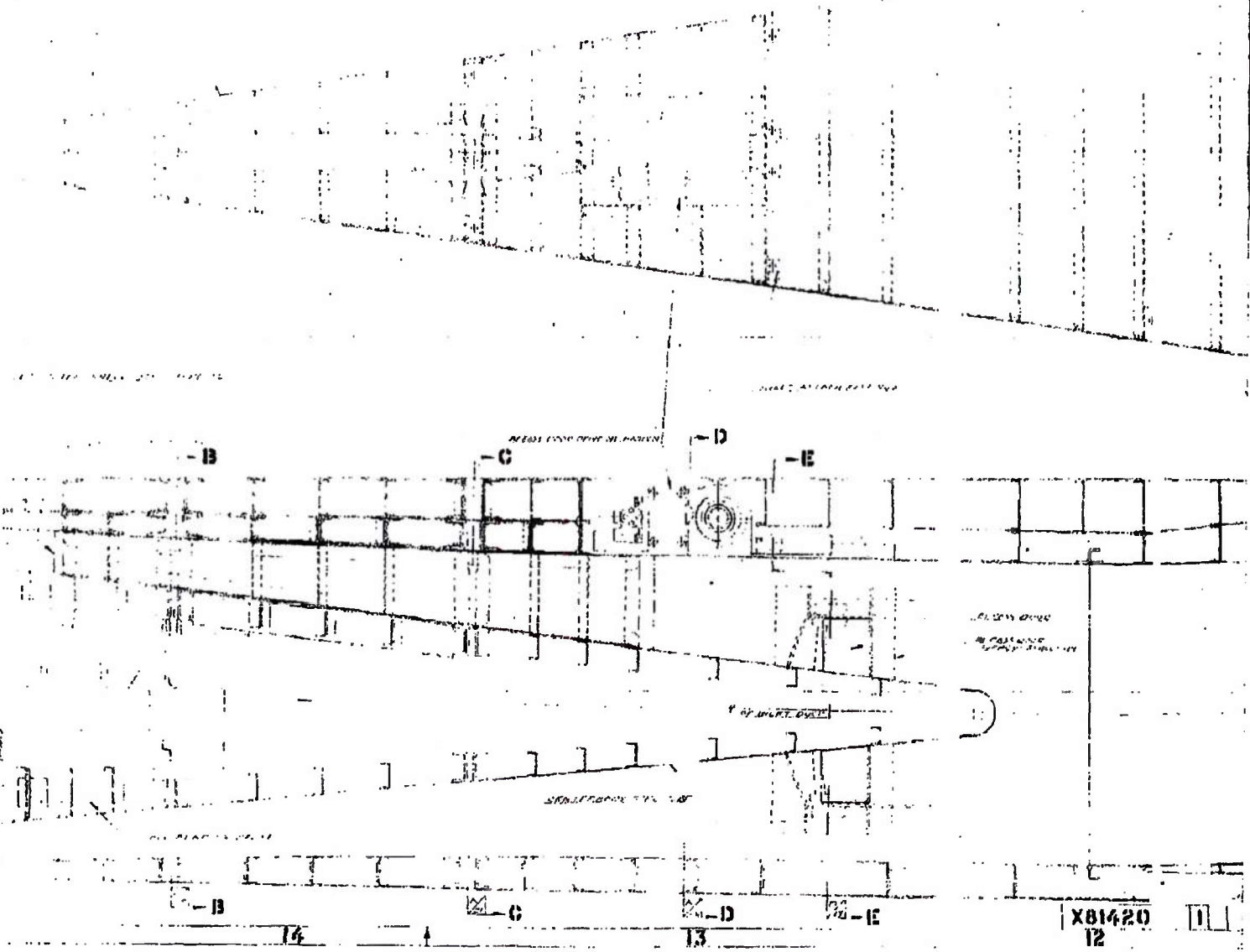
A-A

4

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

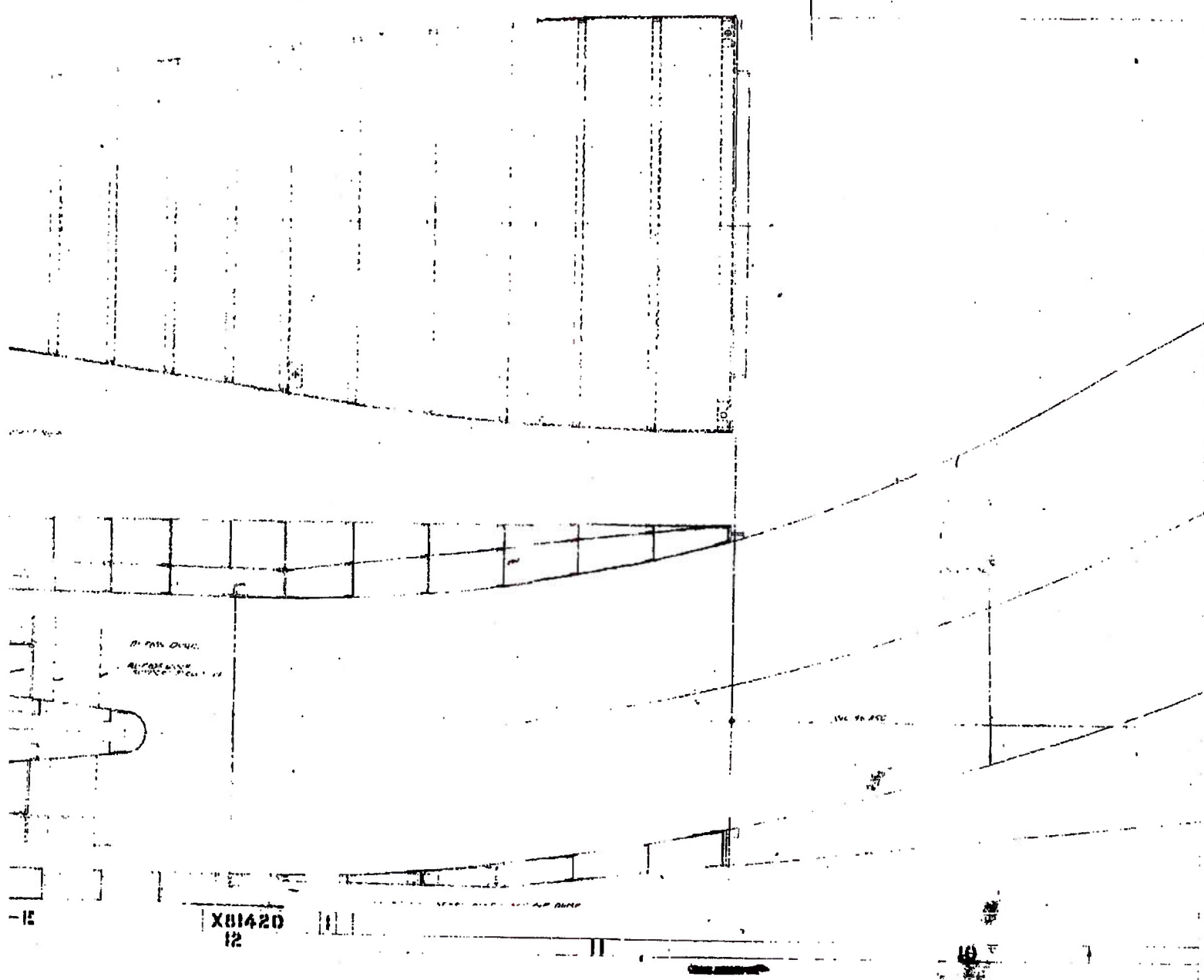


5



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

6



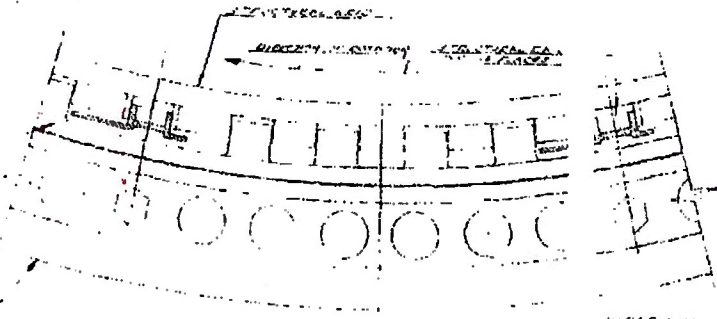
DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

7

P. 0002

SHEAR STRESS
WASHER IN G.M.
POSITION

0.0000000000000000



P-F

X01420

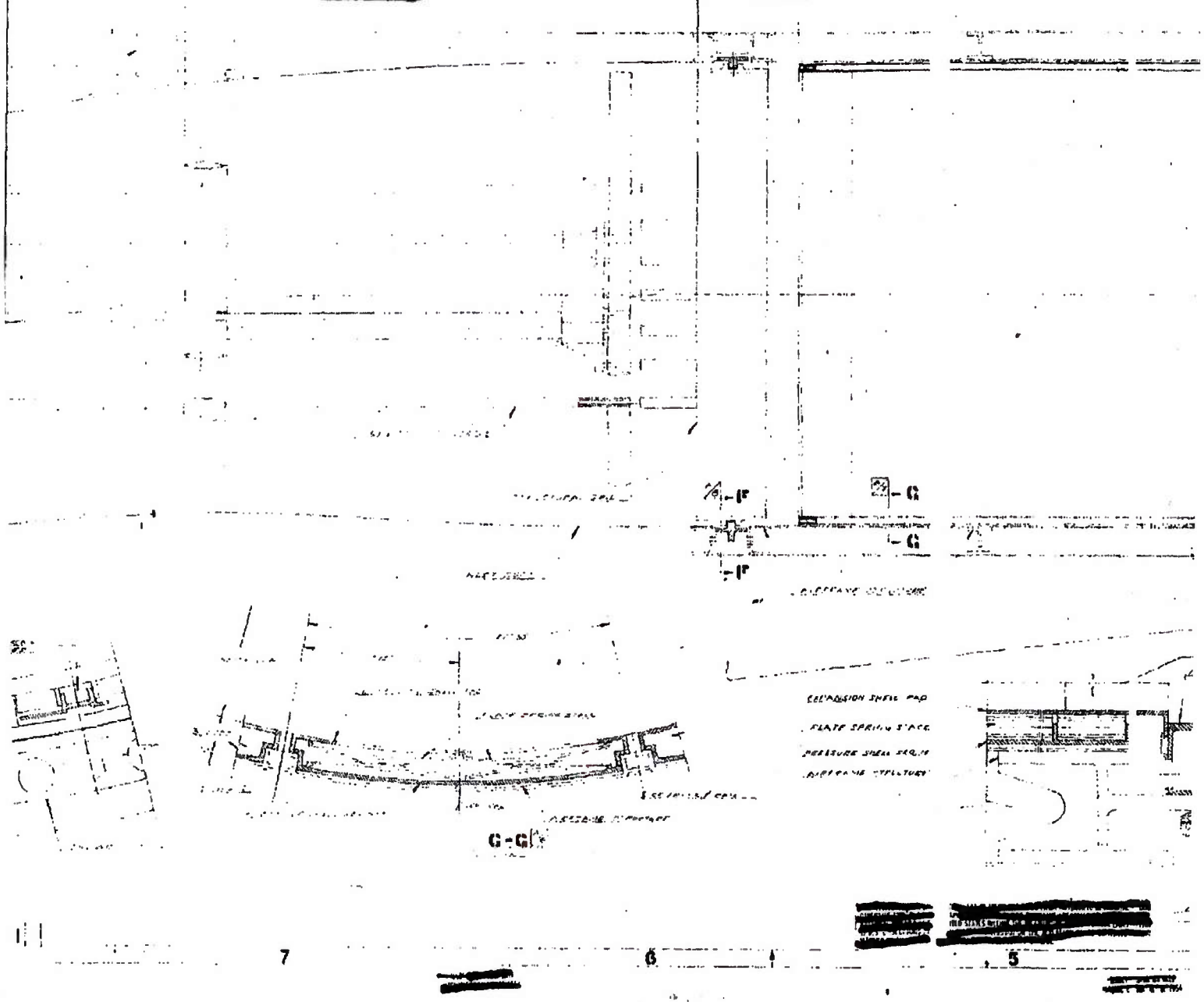
0

1

7

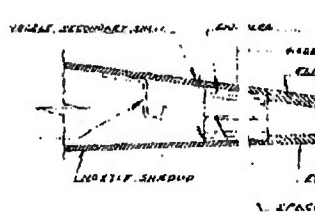
DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

8



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

9



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

NR 6003

10

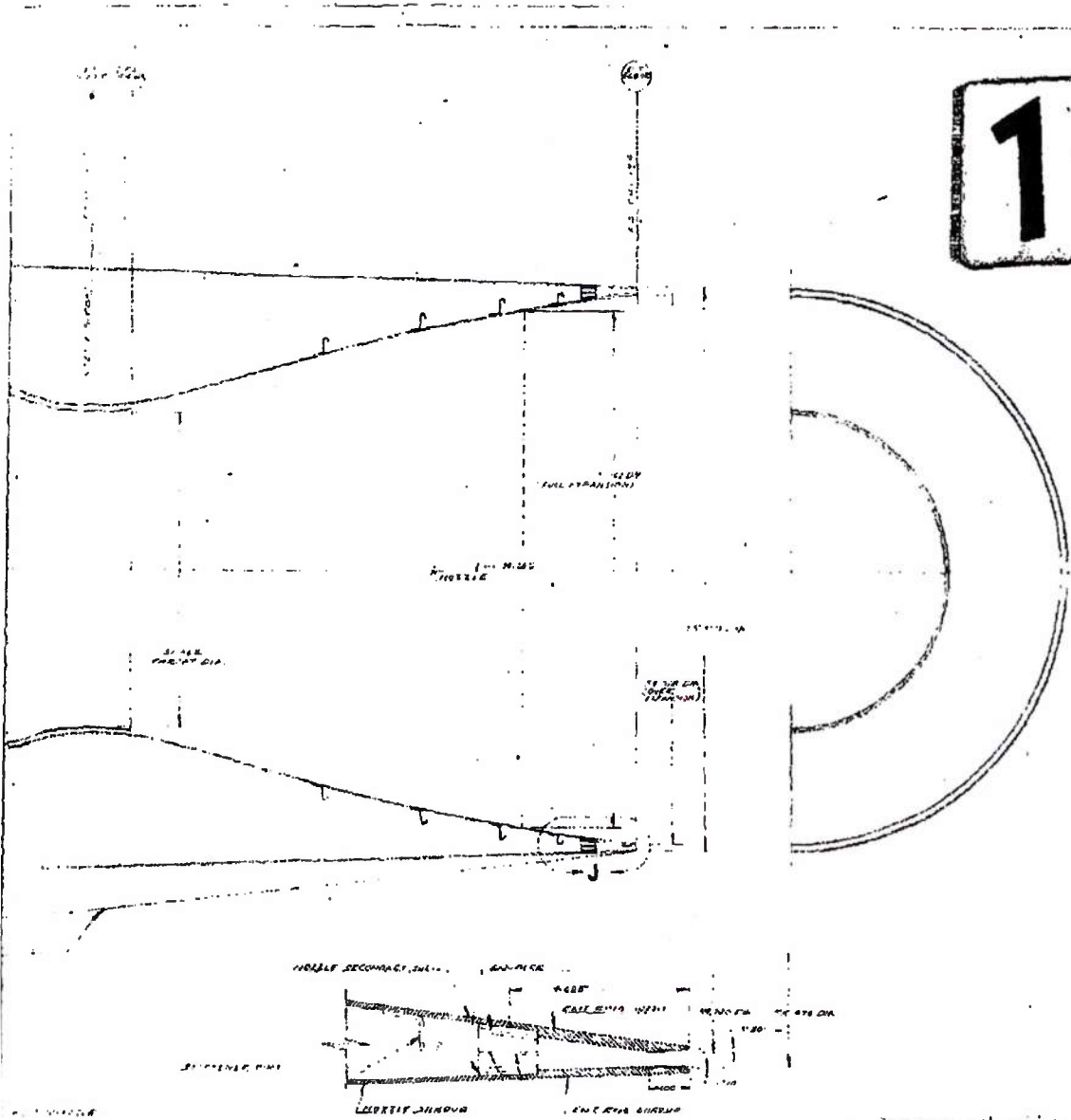


FIGURE 1. Master Layc
Pluto Propulsion System, A50-XCB

1. DESIGN OF THE PLUTO PROPULSION SYSTEM
2. ENGINE LAYOUT AND INSTALLATION
3. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
4. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
5. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
6. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
7. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
8. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
9. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM
10. DESIGN AND CONSTRUCTION OF THE PLUTO PROPULSION SYSTEM

1	2	3	4	5	6	7	8	9	10

Arquand
01420

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

ASD-TDR-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

HEADLINE

(TERRAIN AVOIDANCE
ANTENNA (REF))

TERRAIN AVOIDANCE ANTENNA
(REF)

LONGITUDINAL STABILIZER (A)

ELEVATOR (REF)

WAGON (REF)

Alt. 45,650

VIEW LOOKING AFT

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

15

14

13

STABILIZER (REF)

WL 100/100

TERRAIN AVOIDANCE ANTENNA
(REF)

PS
100000

PS
100000

PS
200000

PS
300000

RUDDER
(REF)

ELEVATOR
(REF)

2

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

X81425

162900
F.S.
(247,961)

49340

F. S.
66.136

1000

QUODIN -
1951

ELEVATOR -
1951

1945.77.0.6

CONFIDENTIAL

• 1 A 10. 100. 400. 600. 800. 1000.

8.1

3

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

19

5

CG OF INLET ASSEMBLY
(WEIGHT: 1712.1 POUNDS)

ES
(164.900)

ES
(176.938)

ES
(166.354)

AIRFRAME, OUTER AL

CG OF
DIFFUSOR
(WEIGHT: 1)

SONIC
DUCT
(WEIGHT: 1)

CG OF CONTROL
SURFACE STRUCTURE
(WEIGHT: 121.1 POUNDS)

ES
(164.900)

ES
(164.900)

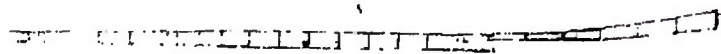
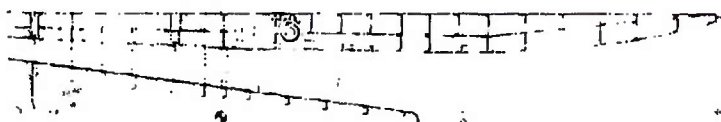
REACTION CONTROL
REACTION CONTROL
AND MECHANISM

REACTION CONTROL

REACTION CONTROL

REACTION CONTROL

REACTION CONTROL
REACTION CONTROL
REACTION CONTROL



REACTION CONTROL AND MECHANISM

REACTION CONTROL AND MECHANISM

REACTION CONTROL AND MECHANISM

REACTION CONTROL

REACTION CONTROL

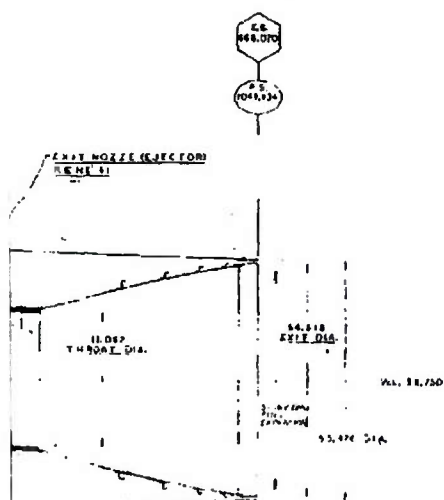
4

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

X81425

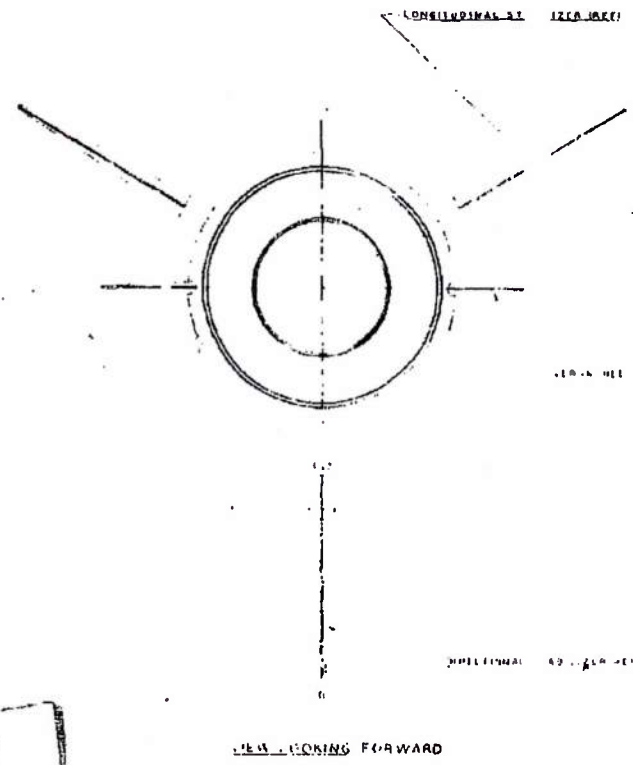
DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

U.S.
(N.S.)



1. ATTACH
NE II

2. STEM
A. RING II



6

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

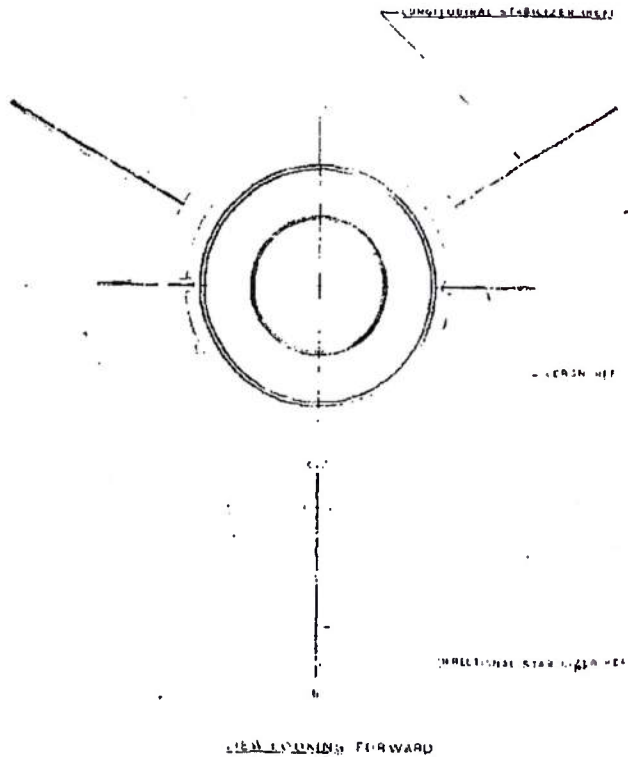
Pluto

X81425

2

- 143-6-11-07

Rept 6003



7

FIGURE 2. Installation
Pluto Propulsion System, MA50-XCB

PLUTO PROPULSION SYSTEM	
MA50-XCB	MA50-XCB
INSTALLATION	
MA50-XCB	
X81425	

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

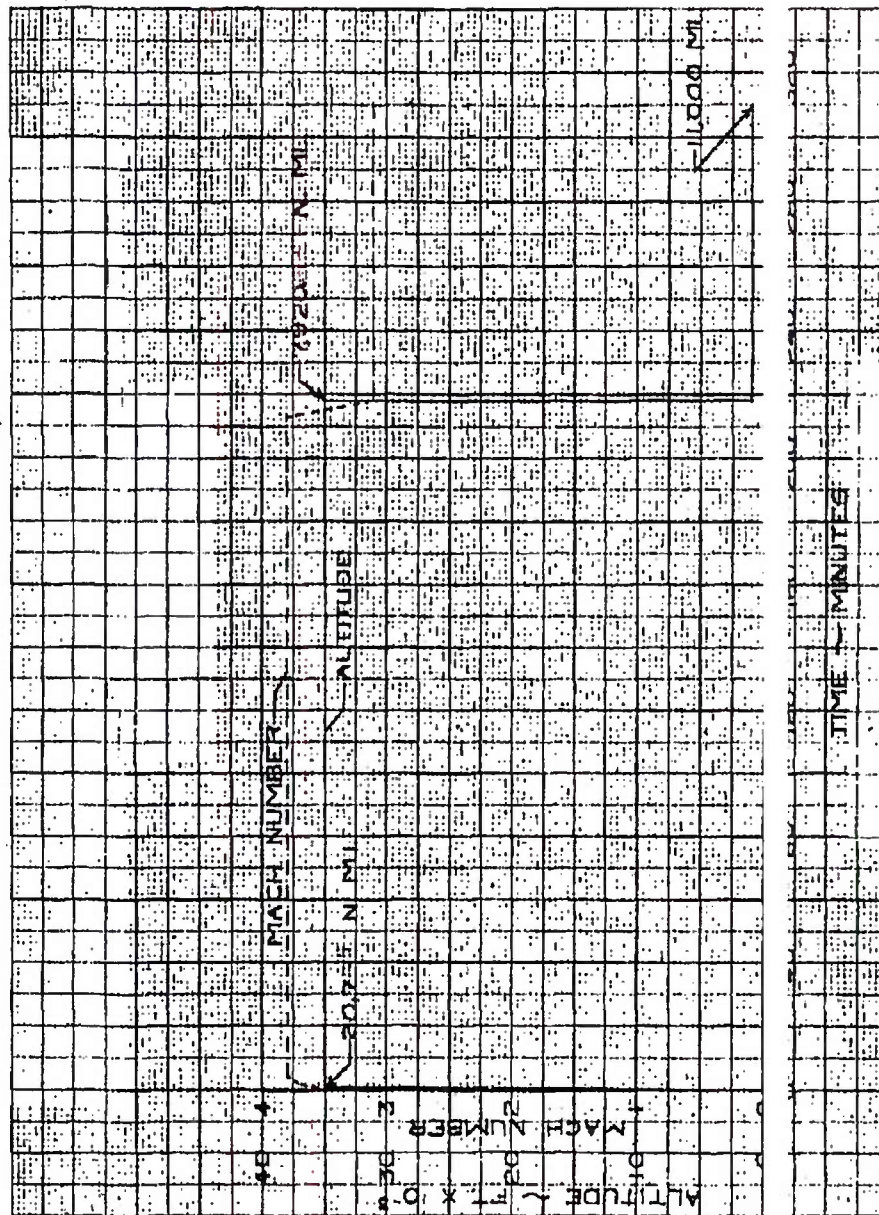


FIGURE 3. Typical Mission Profile for 11,000 Nautical Miles (ICAO Standard Day)

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

MAC 1603

N22E643

- 145 -

DECLASSIFIED IN FULL

Authority: EO 13526

Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 003

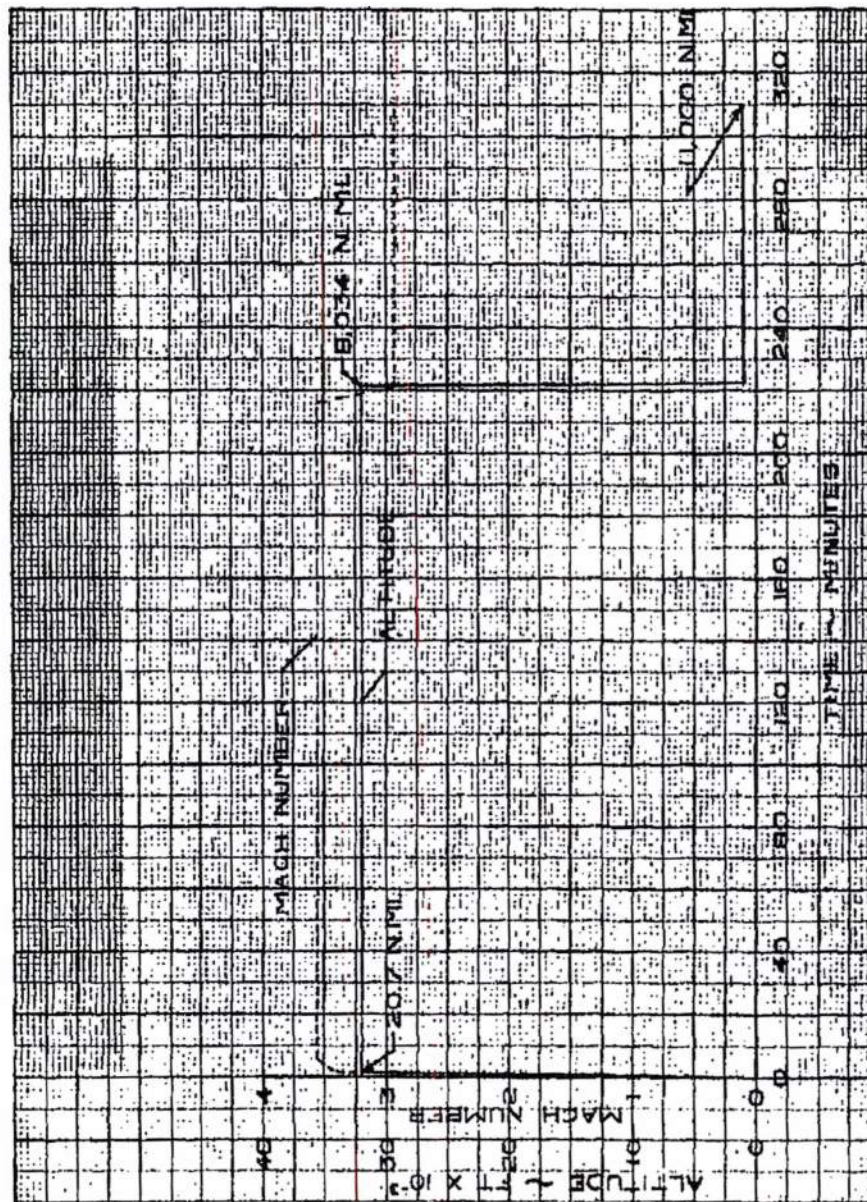


FIGURE 4. Typical Mission Profile for 11,000 Nautical Miles (ANA Hot Day)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E644

- 1/6 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

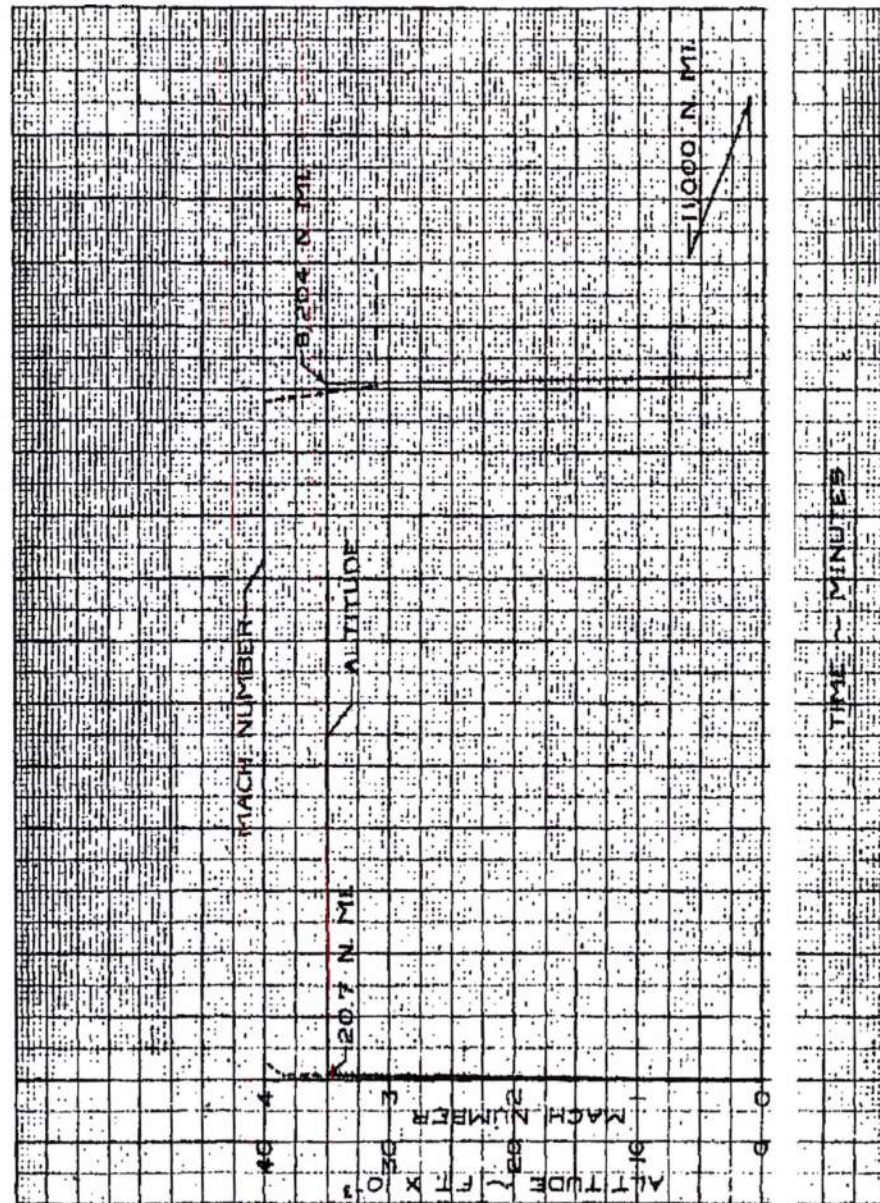


FIGURE 5. Typical Mission Profile for 11,000 Nautical Miles (ANA Cold Day)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E645

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 003

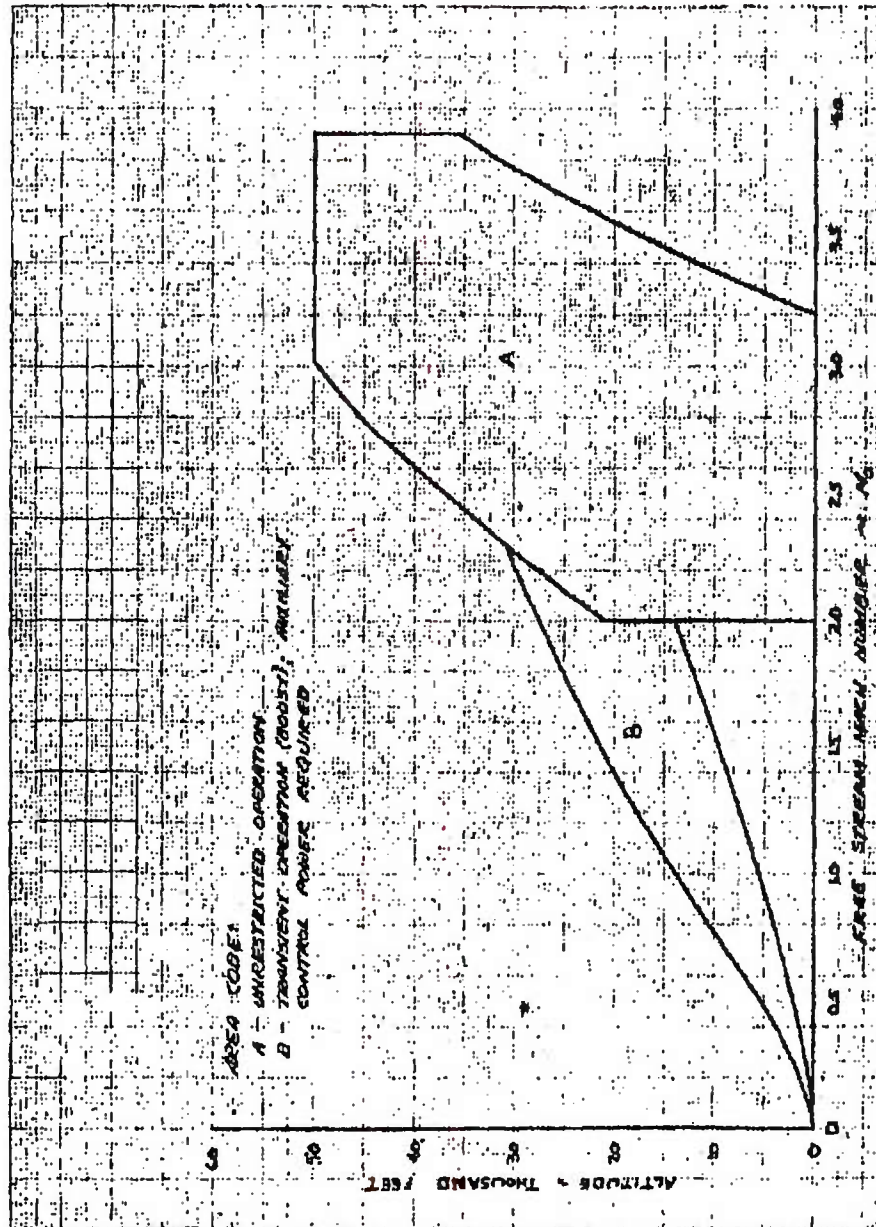


FIGURE 6. Estimated Operating Envelope for Marquardt Model MA50-XCB Ramjet (ICAO Standard Day)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E670

- 148 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

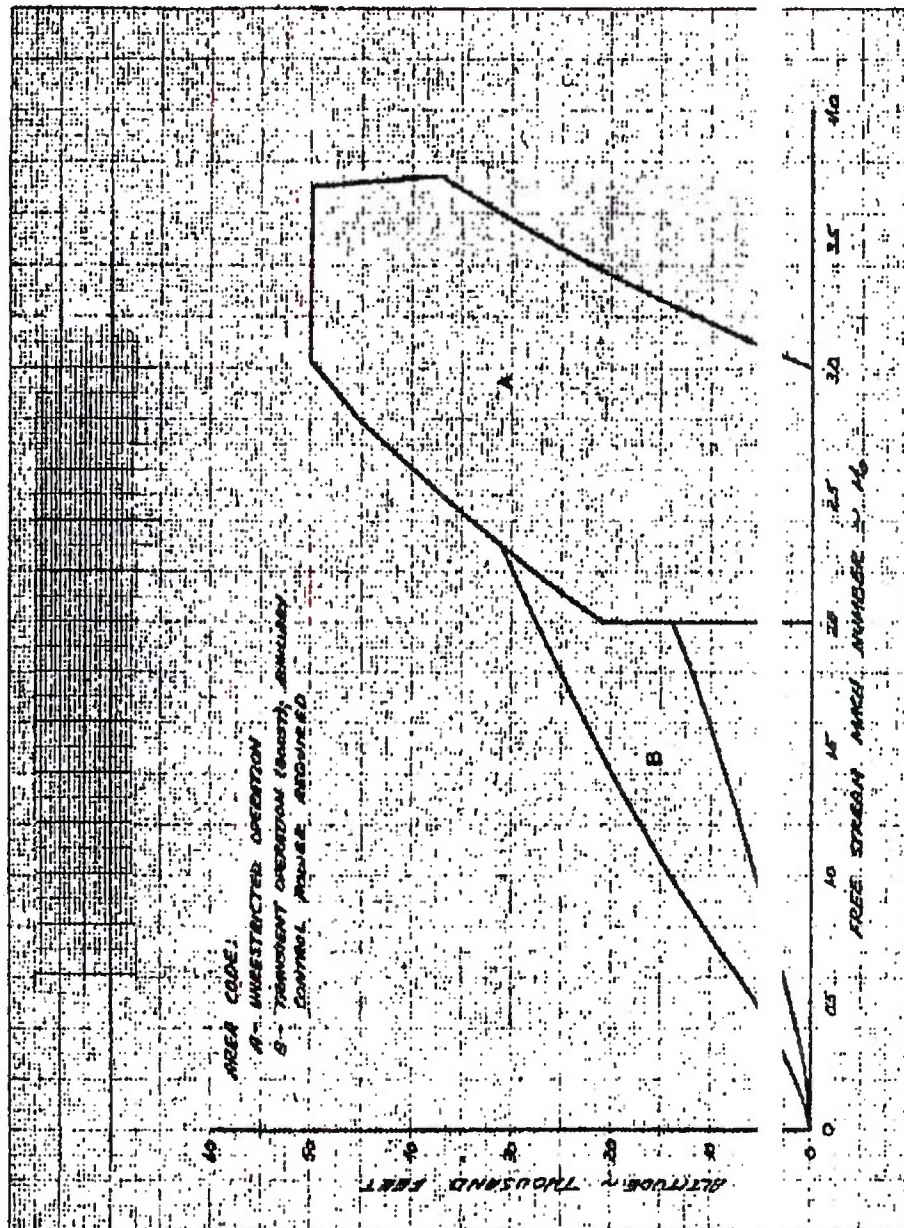


FIGURE 1. Estimated Operating Envelope for Marquardt Model MA50-XCB Ramjet (ANA Hot Day)

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

N22E668

- 149 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

ASD-TDR-63-277, Vol. IV

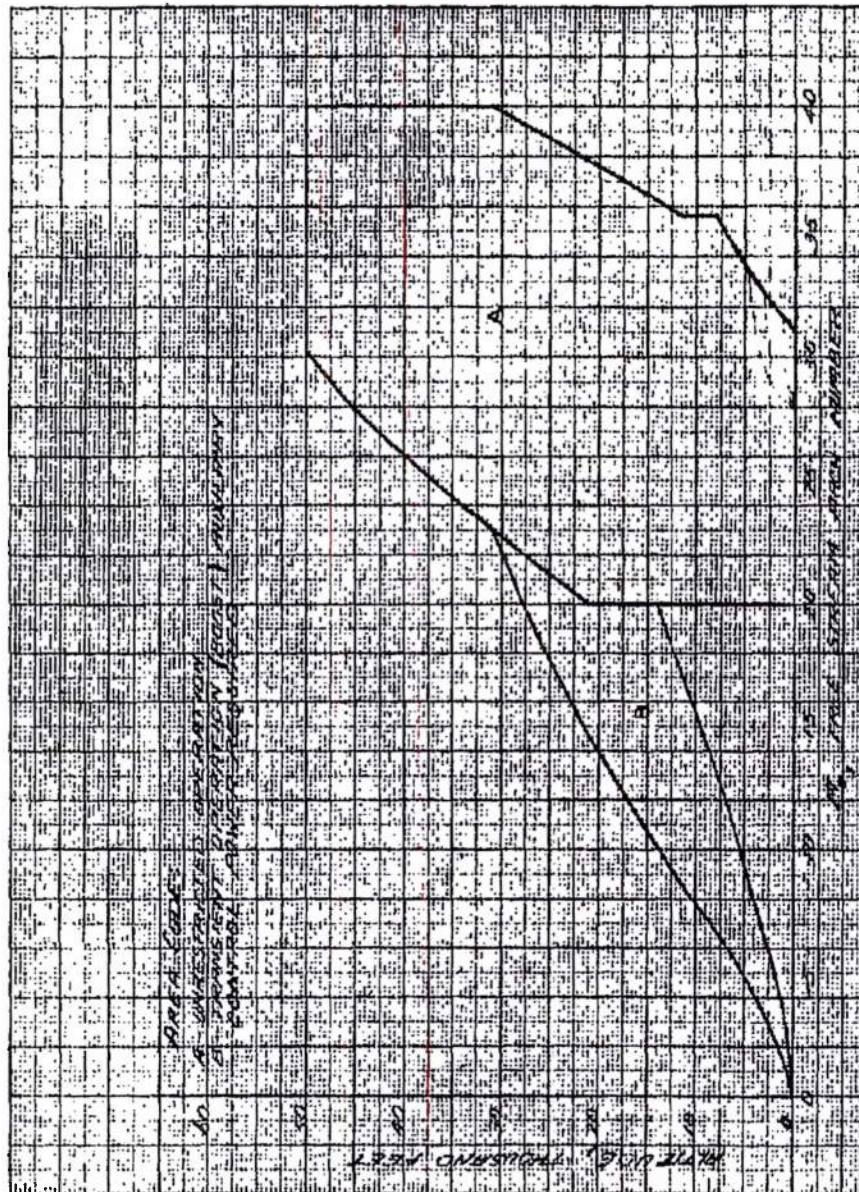


FIGURE 8. Estimated Operating Envelope for Marquardt Model MA50-XCB Ramjet (AVA Cold Day)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E669

- 150 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

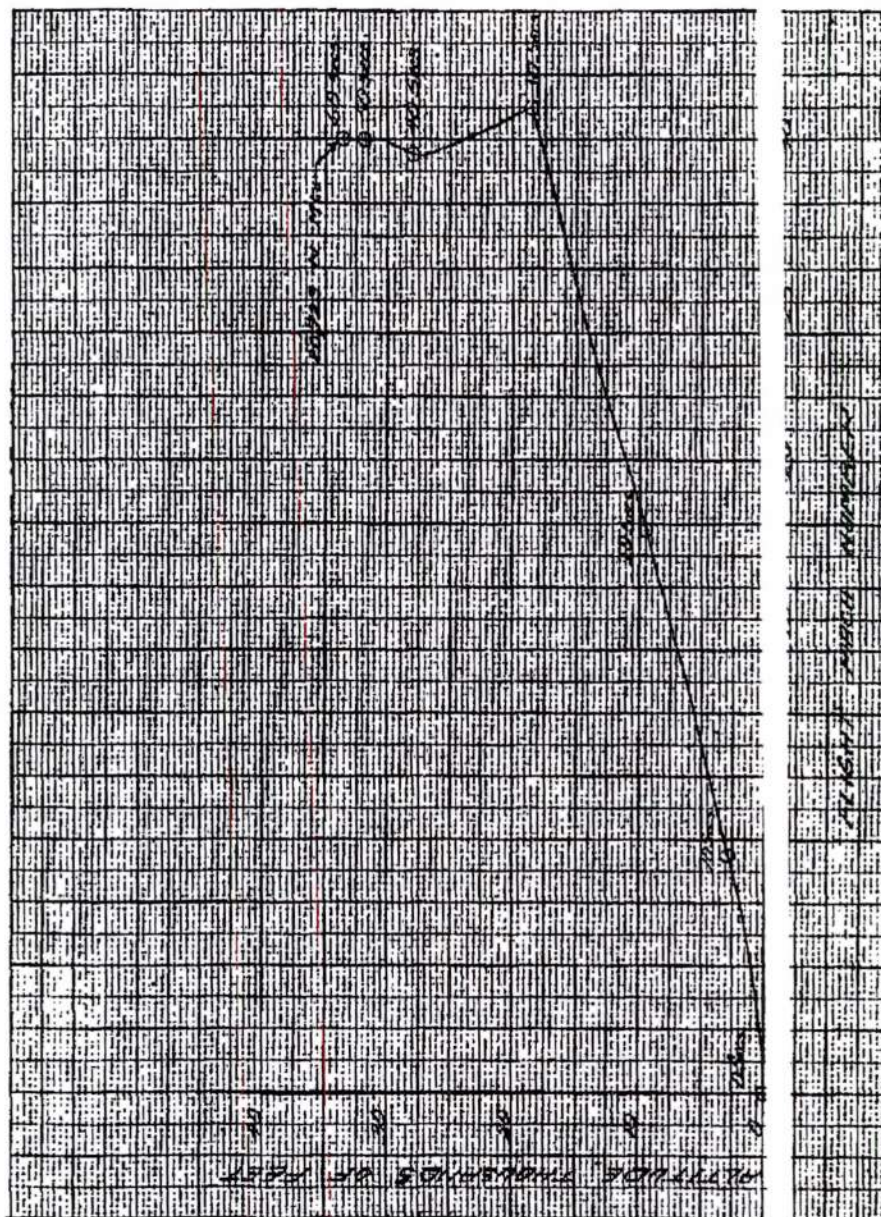


FIGURE 9. Typical Pluto Boost Trajectory

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E646

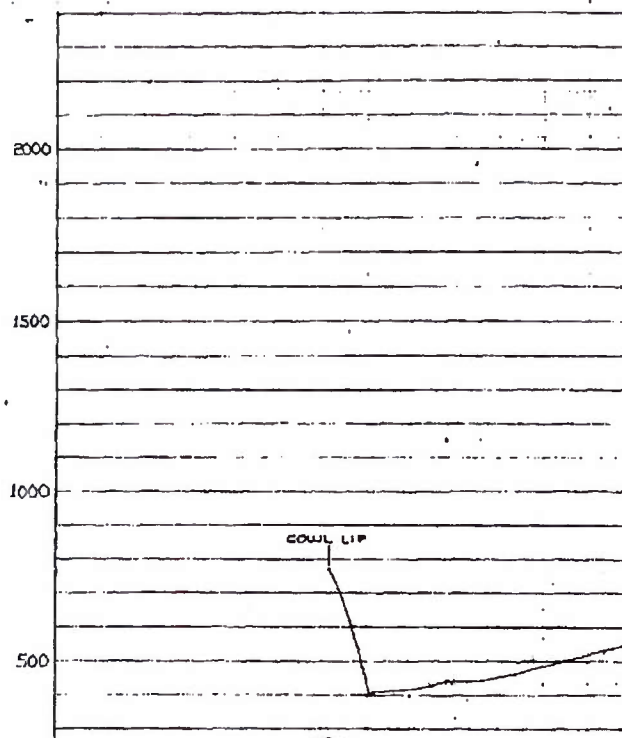
- 151 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief Records & Declass Div, WHS
Date: OCT 02 2015

ASD-TDR-63-277, Vol. IV

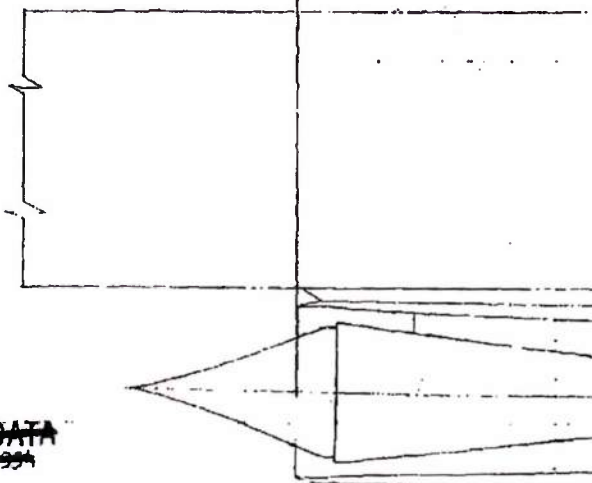
~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~



E. S.
24-1734

1



~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

2

COUL LIP

E. B.
08-734

E. B.
08-734

FLOW AREA ASSUMED NORM
AT INTERSECTION OF CENTER

TO DUCT CENTERLINE
L.V. BYSTON

3

FRONT FACE OF
SUPPORT GRID

0.75 = 1.00 IN.
0.10 = 1.00 IN.
GRAPH SCALE

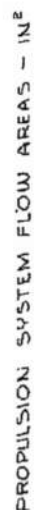
TOTAL ENGINE
PRIMARY ENGINE
COOLING FLOW

NET FLOW AREA DEFINED BY:
THE NET FLOW AREA DEFINED BY
THE AREA DEFINED BY DIFFERENCE

FLOW AREA ASSUMED NORMAL TO DUCT CENTERLINE
BY INTERSECTION OF CENTERLINE & STATION

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100
---	---	---	---	---	---	---	---	---	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	----	-----



4



[REDACTED]

NAME	ADDRESS	CITY	STATE	ZIP
2	STEM			
EAS				
1	WING			



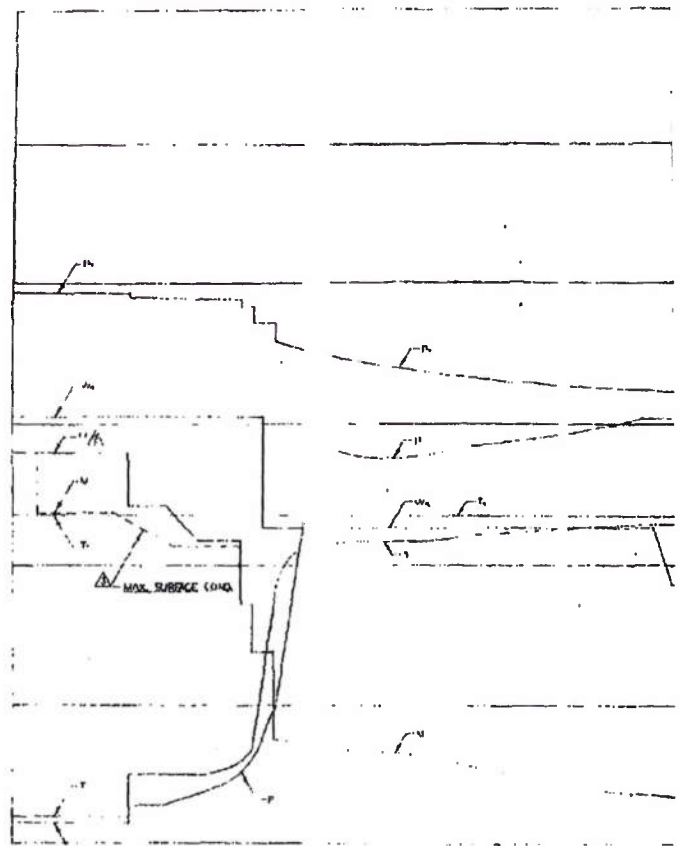
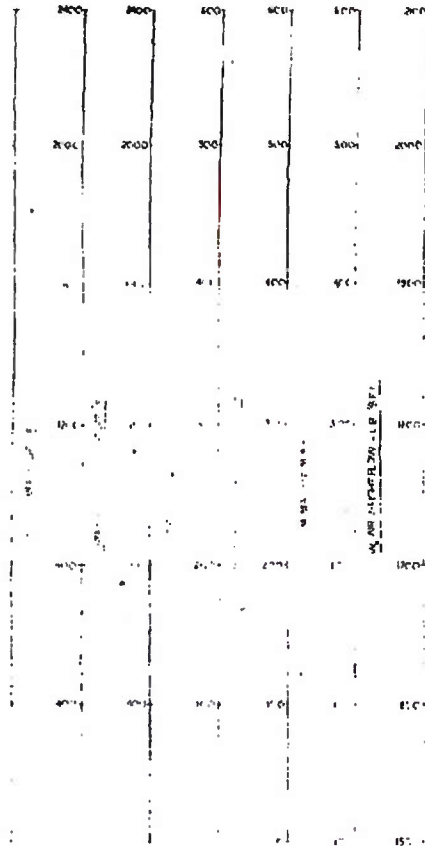
Marquardt

Our products include equipment for
the design and development of
new products, systems, and
processes.

81.575

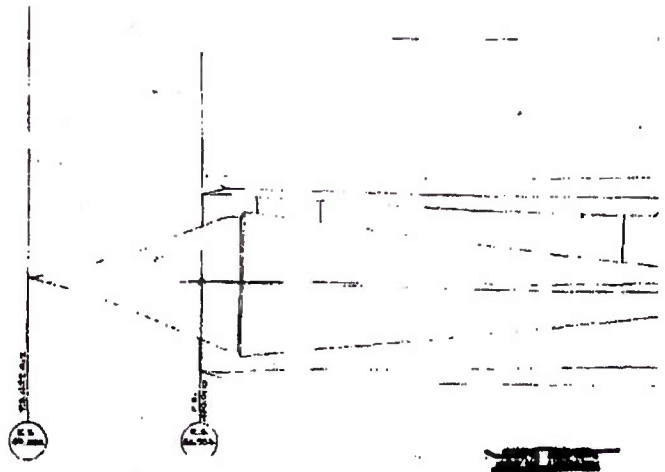
DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WMS
 Date: OCT 02 2015

ASB-JDR-63-277, Vol. IV
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

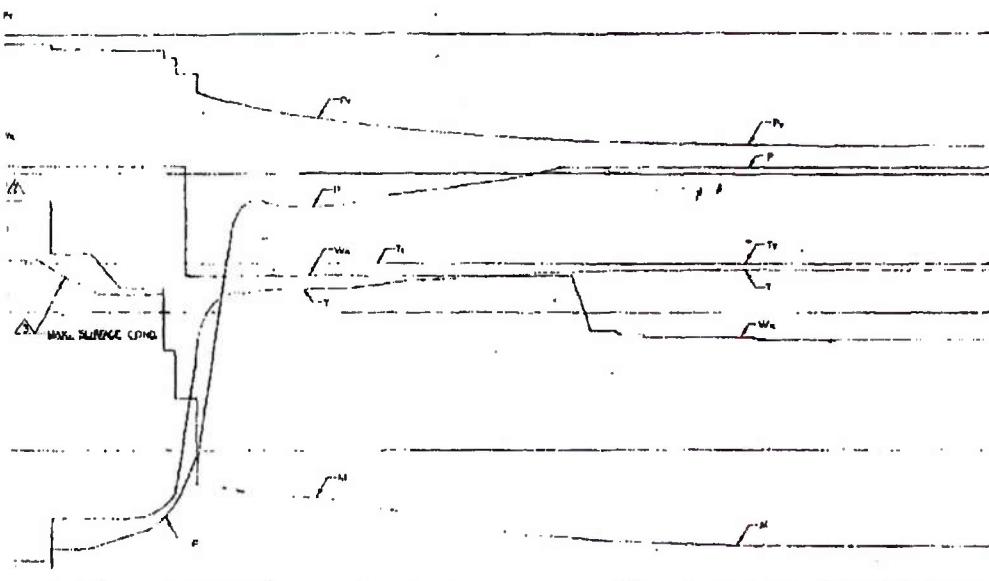


1

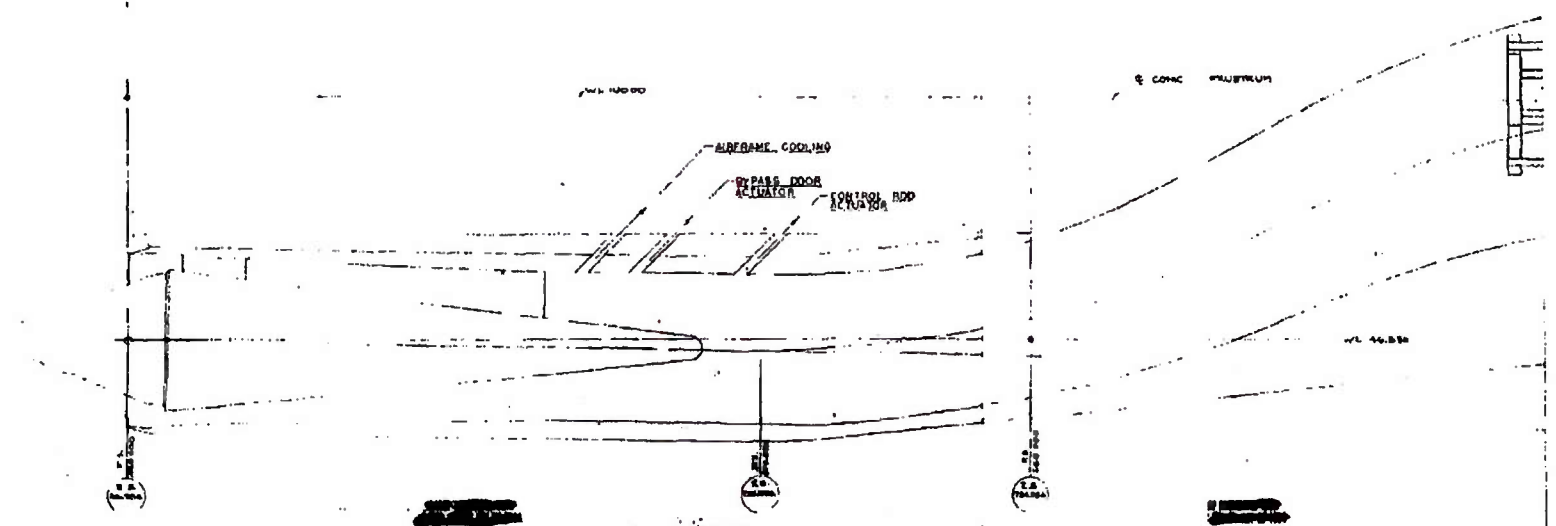
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~



2



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

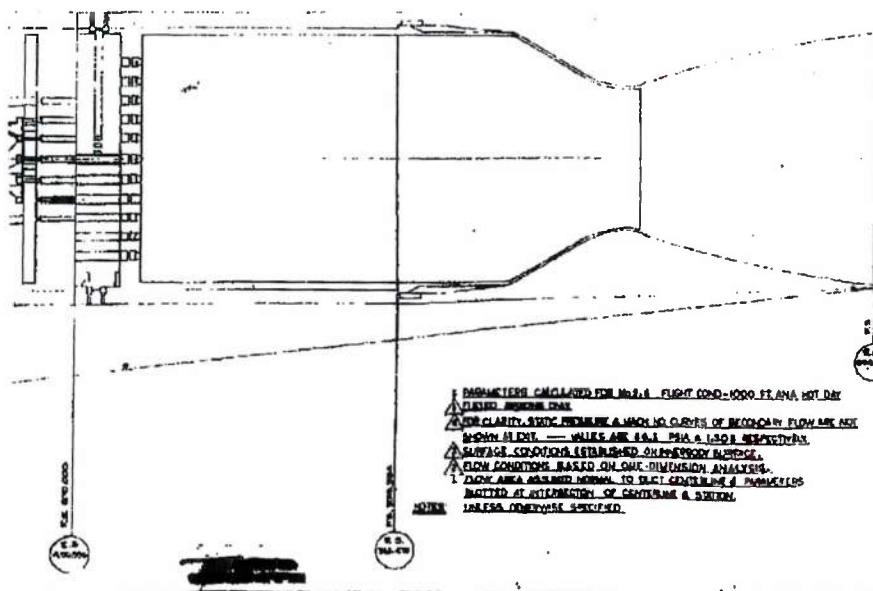
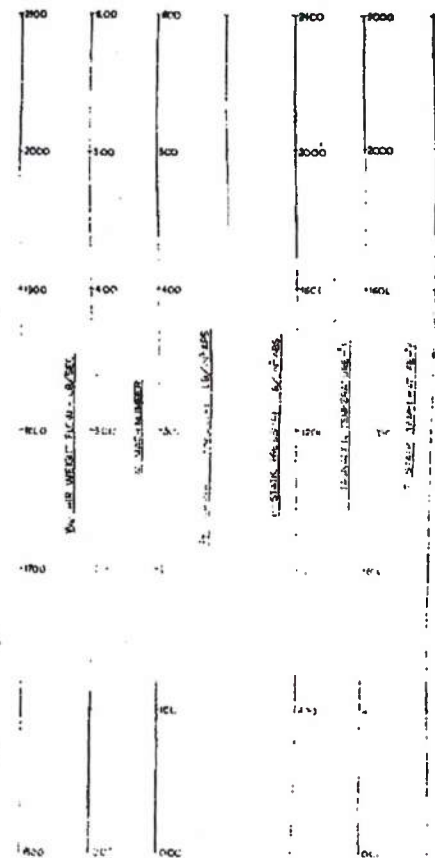
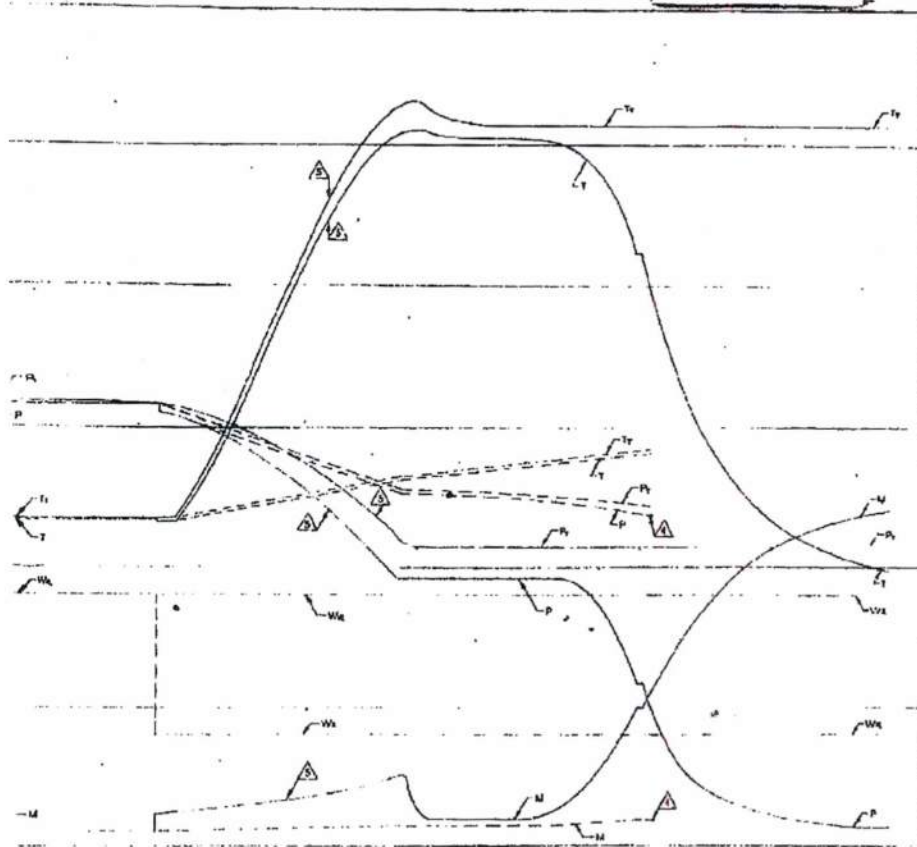


DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

CONICAL MUSEUM

WL 40.000

1. IN THE CASE OF
2. IN THE CASE OF
3. IN THE CASE OF
4. IN THE CASE OF
5. IN THE CASE OF
6. IN THE CASE OF
7. IN THE CASE OF
8. IN THE CASE OF
9. IN THE CASE OF
10. IN THE CASE OF



PARAMETERS CALCULATED FOR No. 8 FLIGHT COND-1000 FLAME HOT DAY
 FLIGHTED, STEADY STATE
 FOR CLARITY, SOME PARAMETERS WHICH NO CURVES OF SECONDARY FLOW ARE NOT
 BEING PLotted. — VALUES ARE 16.1 MM A 1300 RESPECTIVELY
 SURFACE CONDITIONS ESTABLISHED ON BASIS OF PREVIOUS WORK
 FLOW CONDITIONS BASED ON OUR PREVIOUS ANALYSIS
 DATA AREA ASSUMED NORMAL TO FLOW CONTINUING A PARAMETERS
 BASED ON INTERPRETATION OF CONTINUING A STUDY
 UNLESS OTHERWISE SPECIFIED

CODE

ITEM	NAME
1	PRIMARY FLOW
2	SECONDARY FLOW

W.L. 94.750

FIGURE 11. Internal Engine Parameters for Design Point Operation



- 155 & 156 -

480-CDR-63-277, Vol. IV
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

BOUNDARY LAYER AIR BLEED
 LOCATION: BLEED INLET
 W₀ --- 80
 A --- 65
 T₀ --- 942

COOLING AIRFLOW - INLET INNERBODY
 LOCATION: INSIDE INNERBODY STRUTS
 W₀ --- 80
 A --- 130
 T₀ --- 942

ACTUATING AIRFLOW - BY PASS DOORS MOTOR
 LOCATION: AIR INLET FOR MOTOR
 W₀ --- 5 MAX.
 T₀ --- 942
 P --- 45 TO 350

COOLING AIRFLOW - IFRAME
 LOCATION: PASS TO RANGE FROM
 W₀ --- 41
 T₀ --- 942

ACTUAL
 LOCATION: AIRFLOW - BY PASS DOORS MOTOR
 OVERBOARD EXHAUST
 W₀ --- MAX.

1

ACTUATING AIRFLOW - NOSE SPIKE MOTOR
 LOCATION: AIR INLET FOR MOTOR
 W₀ --- 5 MAX.

COOLING AIRFLOW - INLET INNERBODY
 LOCATION: INSIDE INNER
 W₀ --- 80
 T₀ --- 942

ACTUATING AIRFLOW - NOSE SPIKE MOTOR
 LOCATION: OVERBOARD EXHAUST
 W₀ --- 5 MAX.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

2

ACTUATING AIRFLOW - BY PASS DOORS MOTOR
LOCATION: OVERBOARD EXHAUST
W₀ - 3 MAX.

COOLING AIRFLOW - SHADOW SHIELD
LOCATION: AVERAGE SECTION THRU SHIELD

W₀ - 3.0

T₀ - 84.2

TEMP OF SHIELD - 1700

AIRFLOW - BY PASS DOORS MOTOR
LOC. INLET, 700 MPOR
MAX.

TO 350

COOLING AIRFLOW - AIRFRAME
LOCATION: BURO ENTRANCE FROM MAIN DUCT
W₀ - 4.1

T₀ - 84.2

AIR CONDITIONING SYSTEM
LOCATION: EQUIPMENT COMPARTMENT
W₀ - 3.5

ACTUATING AIRFLOW
LOCATION: AIR I
W₀ - 17.04

T₀ - 84.2

P₀ - 0.13

CONTROL ROOM MOTOR
FOR MOTOR

EXHAUST AIRFLOW - CO
LOCATION: SHUTTER
W₀ - 17 MAX.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

COOLING AIRFLOW - INLET INNERBODY
LOCATION: INSOLE INNERBODY
W₀ - 8.1

T₀ - 84.2

COOLING AIRFLOW - INLET INNERBODY
LOCATION: OVERBOARD EXHAUST
W₀ - 8.0

A - 110

COOLING AIRFLOW - AIRFRAME
LOCATION: FAN NO. ENGINE
W₀ - 4.2

A - VARIOUS

T₀ - 84.2

TEMP OF AIR - 1180
TEMP OF FAN - 1180
TEMP OF SKI - 1000

AIRFLOW - NOSE SPIKE MOTOR
OVERBOARD EXHAUST
MAX.

SHUT SHOWN ROTATED TO POSITION 1 FOR THIS CLARITY ONLY
LOWER STRUTS ACTUAL POSITION - 100° AND 120° FROM TOP 1

X815

4

KLING AIRFLOW - SHADOW SHIELD
ATTN: EYE PAGE SECTION THRU SHIELD

--- 34
--- 34 2

IR OF SHIELD - 1100

3

ACTUATING AIRFLOW - CONTROL ROD MOTOR
LOCATION: AIR UNIT FOR MOTOR
W --- 117 MAX.
T --- 843
P --- 313

EXHAUST AIRFLOW - CONTROL ROD MOTOR
LOCATION: AIRFLOW FLOW FROM AIR UNIT
W --- 117 MAX.

COOLING AIRFLOW - REACTOR SUPPORT
LOCATION: AROUND REACTOR
W --- 100
A --- 100
T --- 1058
P --- 270
V --- 263
T₂ --- 1180
TEMP OF SPRINGS --- 1100
TEMP OF PRESSURE SHELL --- 1200

COOLING AIRFLOW - REACTOR CENTRA
LOCATION: CENTRA
W --- 100
A --- 72
T --- 1058
P --- 270
V --- 263
T₂ --- 1180

OUT NOZZLE
TO NOZZLE

COOLING AIRFLOW - EXIT
LOCATION: AIR FOR EXIT
W --- 100
A --- 42
T --- 1130
P --- 543
V --- 248
T₂ --- 1210

COOLING AIRFLOW - AIRFRAME
LOCATION: AROUND REACTOR
W --- 41
A --- 600
T --- 942
TEMP OF SUPPORT --- 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: AIRFRAME
W --- 41
A --- 600
T --- 942
TEMP OF AIR --- 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: AIRFRAME
W --- 41
A --- 600
T --- 942
TEMP OF AIR --- 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: AIRFRAME
W --- 41
A --- 600
T --- 942
TEMP OF AIR --- 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: AIRFRAME
W --- 41
A --- 600
T --- 942
TEMP OF AIR --- 1200

TEMP OF DUCT --- 1180
TEMP OF FRAME --- 1180
TEMP OF SKIN --- 1000

W --- 100
A --- 72
T --- 1058
P --- 270
V --- 263
T₂ --- 1180
TEMP OF AIR --- 1200

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

PRELIMINARY
IF SHOWN, ADDITIONAL DATA WILL BE SHOWN WHEN
DESIGN POINT OPERATION AT M, 2.0, 1000 FT, 2.0

NOTE:

X01576

1

1

3

2

4

ING AIRFLOW - REACTOR SUPPORT
LOCATION: AROUND REACTOR

W ₁	100
A	100
T ₁	542
P ₁	315

OF SPRINGS - 1400
OF PRESSURE - 1200

COOLING AIRFLOW - EXIT NOZZLE
LOCATION: ENTRANCE TO NOZZLE

W ₁	100
A	72
T ₁	1062
P ₁	270
V ₁	865
T ₂	1150

COOLING AIRFLOW - EXIT NOZZLE
LOCATION: EJECTOR EXIT

W ₁	100
A	42
T ₁	1130
P ₁	665
V ₁	2442
T ₂	1240



COOLING AIRFLOW - AIRFRAME
LOCATION: AROUND REACTOR

W ₁	44
A	600
T ₁	542

TEMP OF AIRFRAME - 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: THRU EJECTOR FRAME

W ₁	41
A	82
T ₁	1084
P ₁	48
V ₁	103

TEMP OF AIRFRAME - 1200

COOLING AIRFLOW - AIRFRAME
LOCATION: AROUND EXIT NOZZLE THROAT

W ₁	41
A	1876
T ₁	1076
P ₁	47.5
V ₁	50

COOLING AIRFLOW - AIRFRAME
LOCATION: AIRFLOW EXIT

W ₁	41
A	82
T ₁	1030
P ₁	47
V ₁	1750

SYMBOLS

W₁ AIRFLOW - LBS/SEC
A AIRFLOW AREA - IN²
T₁ AIR TEMPERATURE - °F
P₁ AIR PRESSURE - LBS/IN² ABS.
V₁ AIR VELOCITY - FT/SEC
T₂ TEMPERATURE OF AIRFRAME - °F
T₂ TEMPERATURE OF NOZZLE - °F

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

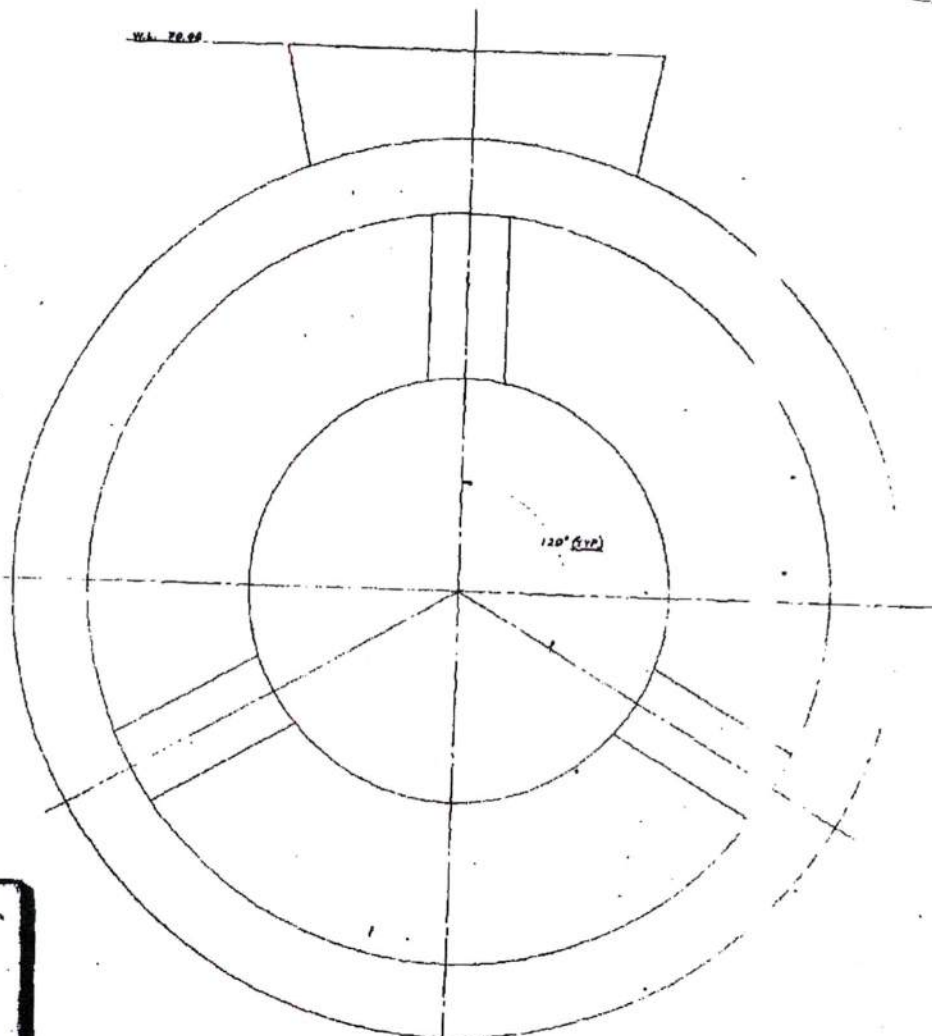
FIGURE 12. MA50-X80 P propulsion System Auxiliary Air Req. elements

1. PRELIMINARY DATA SHOWN, ADDITIONAL DATA WILL BE SHOWN WHEN AVAILABLE.
2. ALL DATA FOR DESIGN POINT OPERATION AT W₁ 2.4, 1000 FT, AND 101 DAY.

NOTE:

MA50-X80 P PROPELLANT: CY AUXILIARY REQUIRE DEVELOPMENT		Marquardt 61576
--	--	-------------------------------

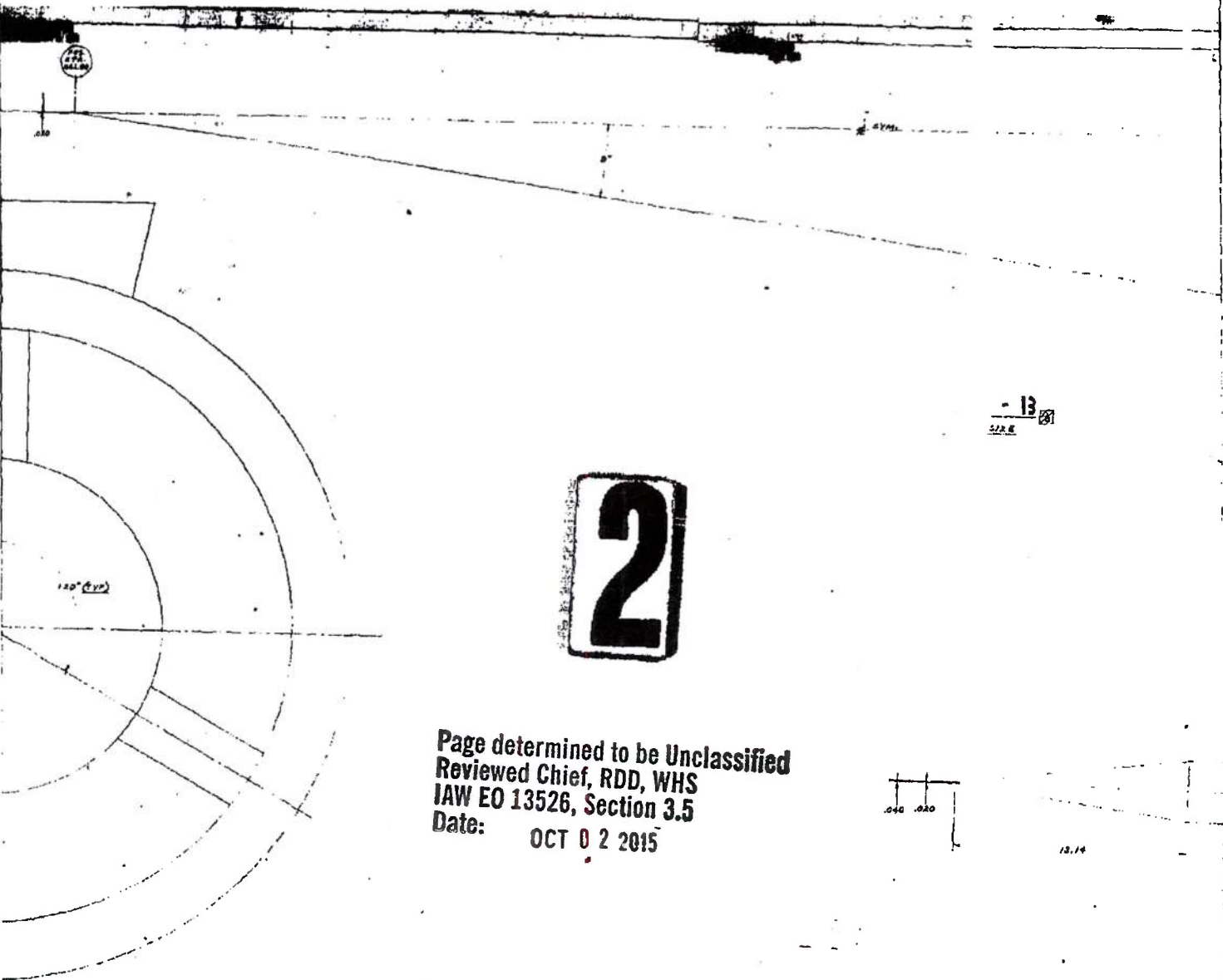
ASD TBR-53-2776 Vol. IV
~~SECRET RESTRICTED DATA~~
ATOMIC ENERGY ACT OF 1954



1

~~SECRET RESTRICTED DATA~~
ATOMIC ENERGY ACT OF 1954

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



2

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

.040	.020

13.14

X 01419

13 - 13

3

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

.048 .020

13.76

A - A

18.76

X 814187

20

19



STA	W.L.	Q	R.	R.
101+00	44.88			8.48
101+10	44.88			8.48
101+20	44.88			8.48
101+30	44.88			8.48
101+40	44.88			8.48
101+50	44.88			8.48
101+60	44.88			8.48
101+70	44.88			8.48
101+80	44.88			8.48
101+90	44.88			8.48
102+00	44.88			8.48
102+10	44.88			8.48
102+20	44.88			8.48
102+30	44.88			8.48
102+40	44.88			8.48
102+50	44.88			8.48
102+60	44.88			8.48
102+70	44.88			8.48
102+80	44.88			8.48
102+90	44.88			8.48
103+00	44.88			8.48
103+10	44.88			8.48
103+20	44.88			8.48
103+30	44.88			8.48
103+40	44.88			8.48
103+50	44.88			8.48
103+60	44.88			8.48
103+70	44.88			8.48
103+80	44.88			8.48
103+90	44.88			8.48
104+00	44.88			8.48
104+10	44.88			8.48
104+20	44.88			8.48
104+30	44.88			8.48
104+40	44.88			8.48
104+50	44.88			8.48
104+60	44.88			8.48
104+70	44.88			8.48
104+80	44.88			8.48
104+90	44.88			8.48
105+00	44.88			8.48
105+10	44.88			8.48
105+20	44.88			8.48
105+30	44.88			8.48
105+40	44.88			8.48
105+50	44.88			8.48
105+60	44.88			8.48
105+70	44.88			8.48
105+80	44.88			8.48
105+90	44.88			8.48
106+00	44.88			8.48
106+10	44.88			8.48
106+20	44.88			8.48
106+30	44.88			8.48
106+40	44.88			8.48
106+50	44.88			8.48
106+60	44.88			8.48
106+70	44.88			8.48
106+80	44.88			8.48
106+90	44.88			8.48
107+00	44.88			8.48
107+10	44.88			8.48
107+20	44.88			8.48
107+30	44.88			8.48
107+40	44.88			8.48
107+50	44.88			8.48
107+60	44.88			8.48
107+70	44.88			8.48
107+80	44.88			8.48
107+90	44.88			8.48
108+00	44.88			8.48
108+10	44.88			8.48
108+20	44.88			8.48
108+30	44.88			8.48
108+40	44.88			8.48
108+50	44.88			8.48
108+60	44.88			8.48
108+70	44.88			8.48
108+80	44.88			8.48
108+90	44.88			8.48
109+00	44.88			8.48
109+10	44.88			8.48
109+20	44.88			8.48
109+30	44.88			8.48
109+40	44.88			8.48
109+50	44.88			8.48
109+60	44.88			8.48
109+70	44.88			8.48
109+80	44.88			8.48
109+90	44.88			8.48
110+00	44.88			8.48
110+10	44.88			8.48
110+20	44.88			8.48
110+30	44.88			8.48
110+40	44.88			8.48
110+50	44.88			8.48
110+60	44.88			8.48
110+70	44.88			8.48
110+80	44.88			8.48
110+90	44.88			8.48
111+00	44.88			8.48
111+10	44.88			8.48
111+20	44.88			8.48
111+30	44.88			8.48
111+40	44.88			8.48
111+50	44.88			8.48
111+60	44.88			8.48
111+70	44.88			8.48
111+80	44.88			8.48
111+90	44.88			8.48
112+00	44.88			8.48
112+10	44.88			8.48
112+20	44.88			8.48
112+30	44.88			8.48
112+40	44.88			8.48
112+50	44.88			8.48
112+60	44.88			8.48
112+70	44.88			8.48
112+80	44.88			8.48
112+90	44.88			8.48
113+00	44.88			8.48
113+10	44.88			8.48
113+20	44.88			8.48
113+30	44.88			8.48
113+40	44.88			8.48
113+50	44.88			8.48
113+60	44.88			8.48
113+70	44.88			8.48
113+80	44.88			8.48
113+90	44.88			8.48
114+00	44.88			8.48
114+10	44.88			8.48
114+20	44.88			8.48
114+30	44.88			8.48
114+40	44.88			8.48
114+50	44.88			8.48
114+60	44.88			8.48
114+70	44.88			8.48
114+80	44.88			8.48
114+90	44.88			8.48
115+00	44.88			8.48
115+10	44.88			8.48
115+20	44.88			8.48
115+30	44.88			8.48
115+40	44.88			8.48
115+50	44.88			8.48
115+60	44.88			8.48
115+70	44.88			8.48
115+80	44.88			8.48
115+90	44.88			8.48
116+00	44.88			8.48
116+10	44.88			8.48
116+20	44.88			8.48
116+30	44.88			8.48
116+40	44.88			8.48
116+50	44.88			8.48
116+60	44.88			8.48
116+70	44.88			8.48
116+80	44.88			8.48
116+90	44.88			8.48
117+00	44.88			8.48
117+10	44.88			8.48
117+20	44.88			8.48
117+30	44.88			8.48
117+40	44.88			8.48
117+50	44.88			8.48
117+60	44.88			8.48
117+70	44.88			8.48
117+80	44.88			8.48
117+90	44.88			8.48
118+00	44.88			8.48
118+10	44.88			8.48
118+20	44.88			8.48
118+30	44.88			8.48
118+40	44.88			8.48
118+50	44.88			8.48
118+60	44.88			8.48
118+70	44.88			8.48
118+80	44.88			8.48
118+90	44.88			8.48
119+00	44.88			8.48
119+10	44.88			8.48
119+20	44.88			8.48
119+30	44.88			8.48
119+40	44.88			8.48
119+50	44.88			8.48
119+60	44.88			8.48
119+70	44.88			8.48
119+80	44.88			8.48
119+90	44.88			8.48
120+00	44.88			8.48
120+10	44.88			8.48
120+20	44.88			8.48
120+30	44.88			8.48
120+40	44.88			8.48
120+50	44.88			8.48
120+60	44.88			8.48
120+70	44.88			8.48
120+80	44.88			8.48
120+90	44.88			8.48

AS SHOWN ON PLAN OF 1944-45

4

W.L. 44.88

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



X 8141911

6

PUS
STA



A₁

R₁

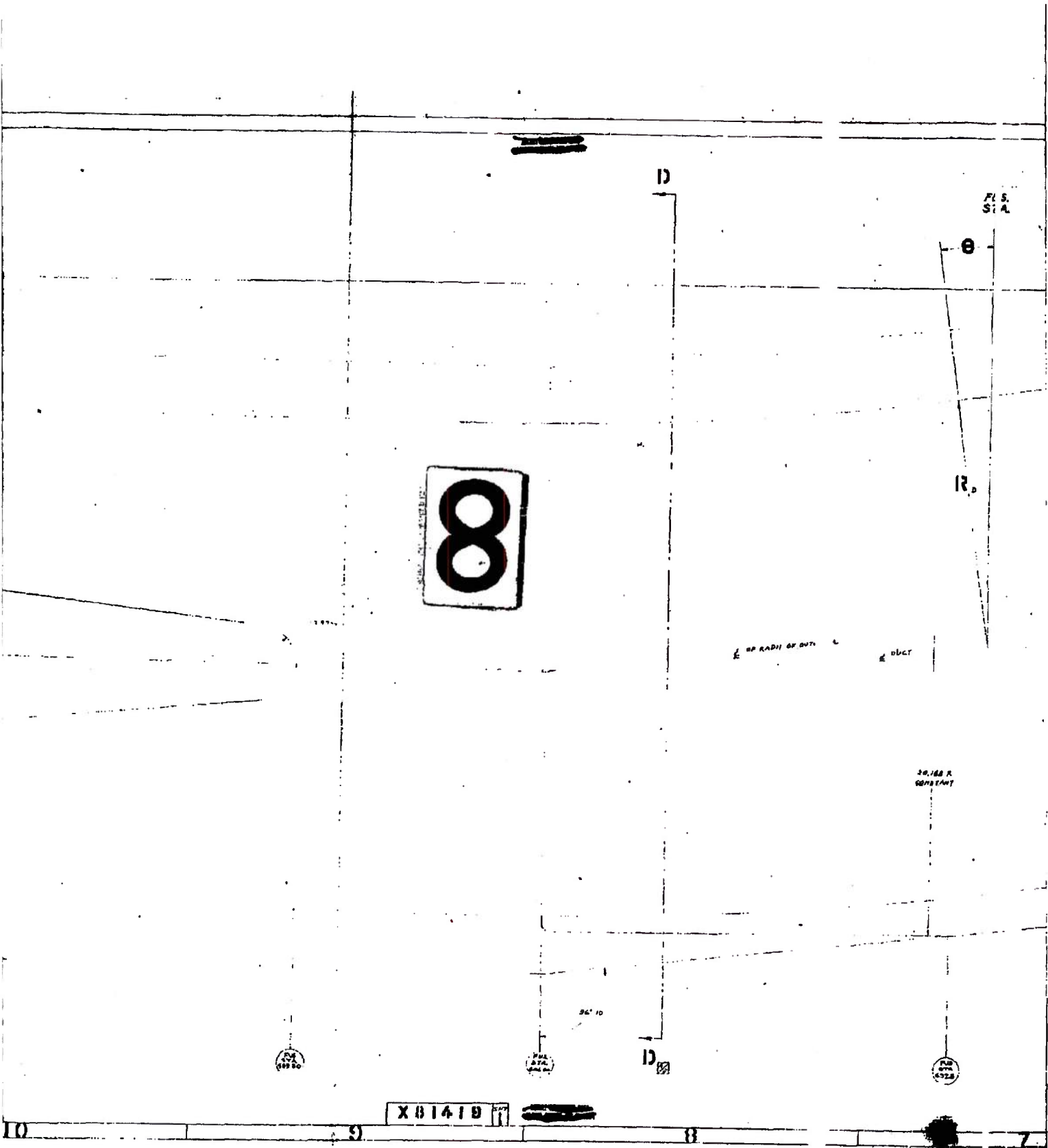
20.186 R
CONSTANT

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

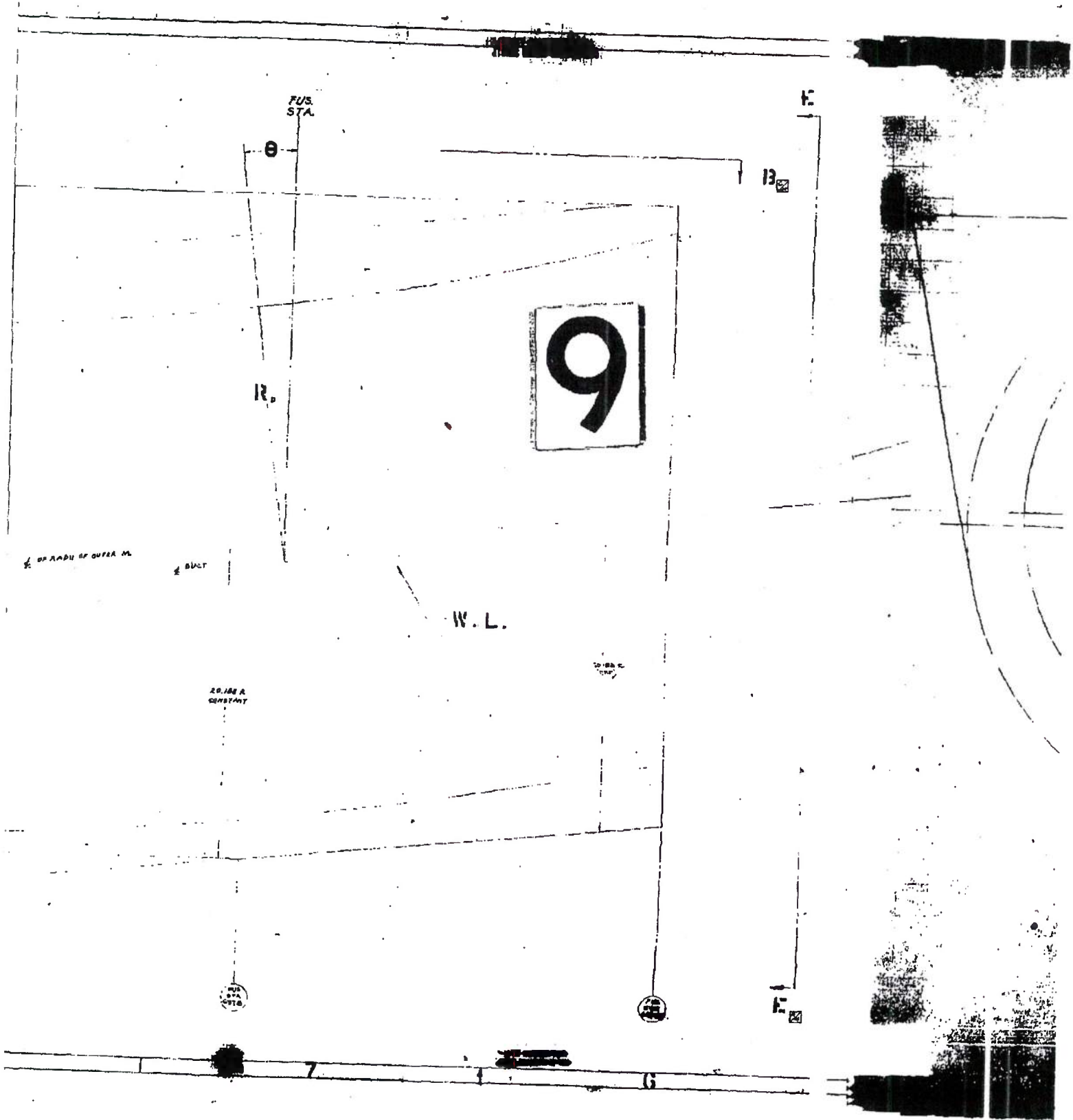
C₁₀

7

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

10

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

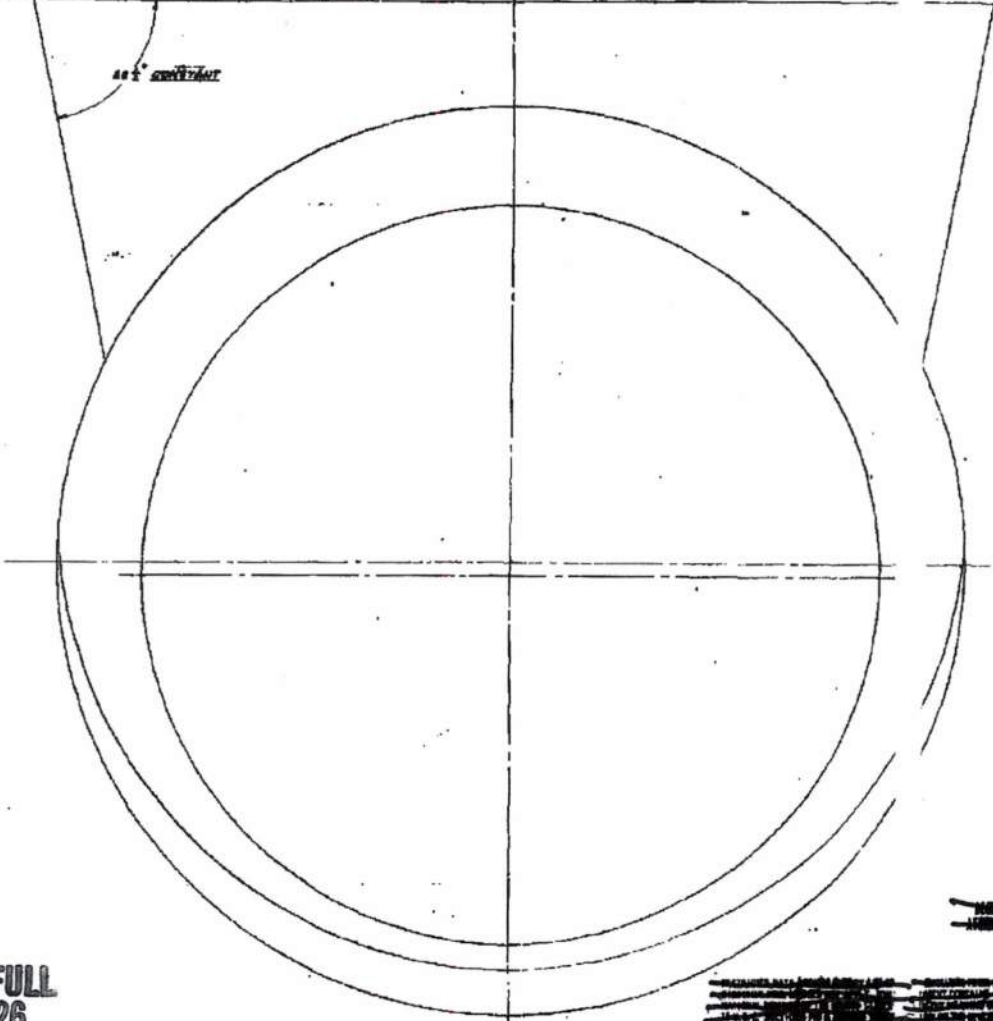
SYM

VIEW LOOKING FIVE

11

W.L. 79.00

AS 1 1/2" DIA



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 13

12 - 12



159 160

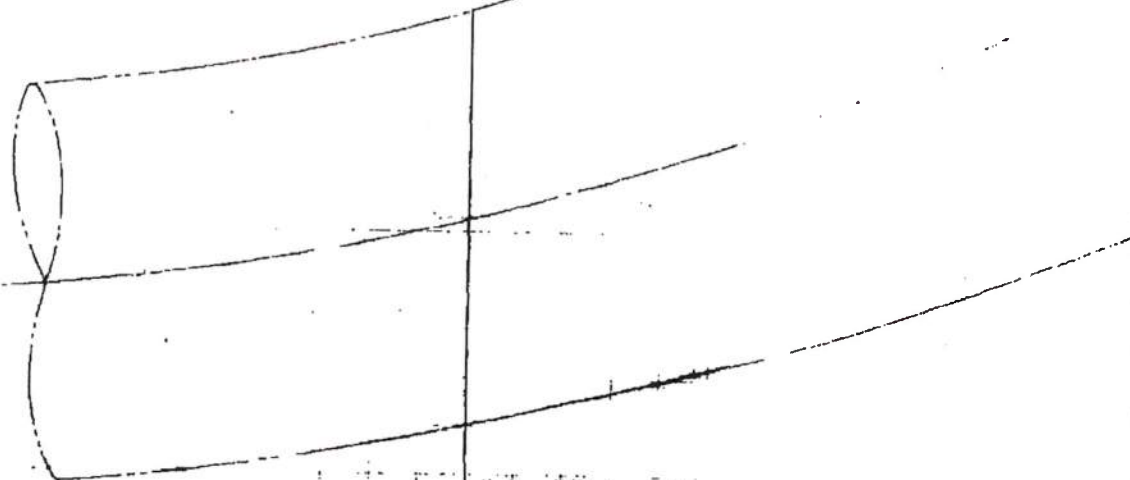
ASD-TDR-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

1



~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

X8

2

FUS.
STA.

9

R.

W. L

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

DUCT FROM FL STA 650.00 TO FUS. S. A. 670.00
± 3126

AS PER TO DO L. ONE THOUSAND NO DIMENSIONS

X61419

2

5

4

3

3

FUS.
STA.

6

R.

W. L.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 11. Design

DUCT FROM FUS. STA. 660.00 TO FUS. STA. 870.00

4 3/4"

APPROX. TO CORR. FOR TYPICAL 60° DIMENSIONS

4

3

1

2

3

REPT F 6003

4

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 1/4, Design Layout, Inlet and Duct Lines

TO FVS, STA. 870.00

EXPLANATIONS

NO.	DESCRIPTION	DATE
1	DESIGN L.	
2	INLET & DUCT	

Marguardt	
OUT-	
INES.	
K 61419	

ASD-TDR-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

K (SHEET 3 OF 3) SEE

1

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

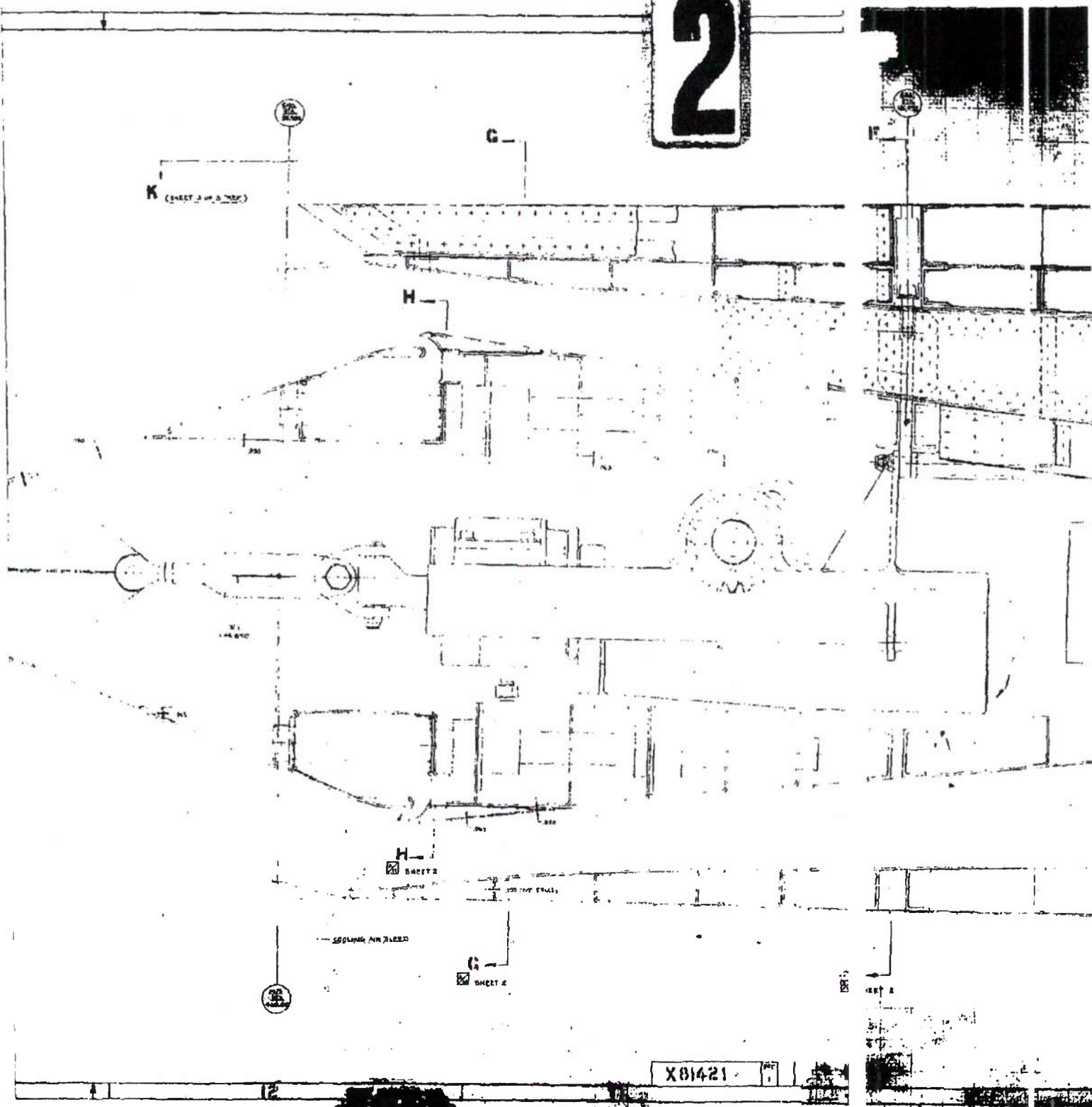
14

13

12

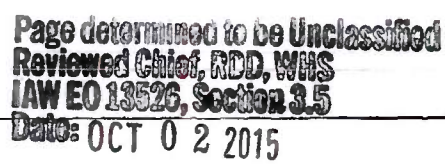
2

K (SHEET 3 OF 5 SHEETS)

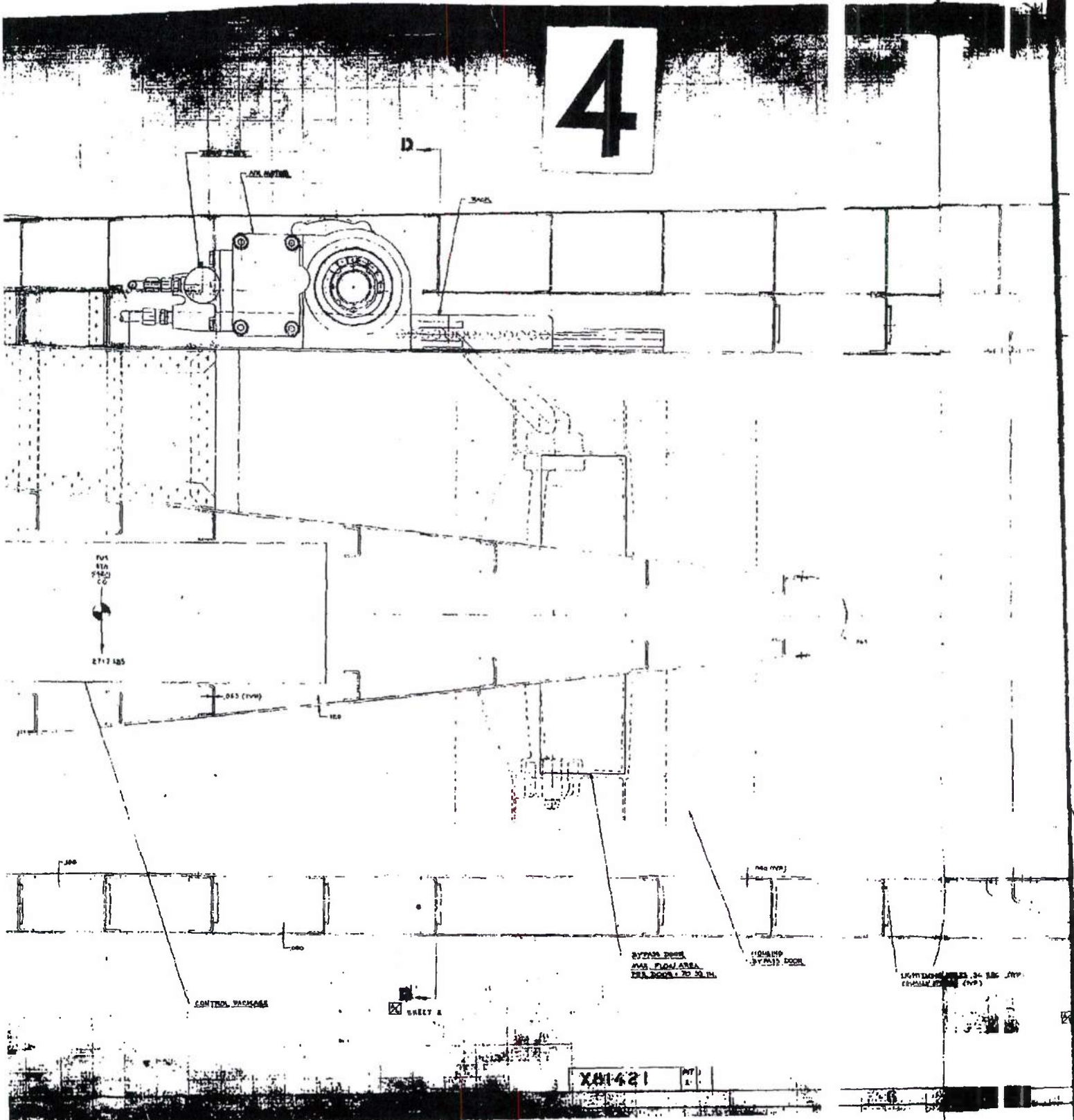


X81421

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



4



5

C

B

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

140 (WP)

NO. 140 (WP)

UNTERKAMP (WP) (WP)

C-1
UNTERKAMP

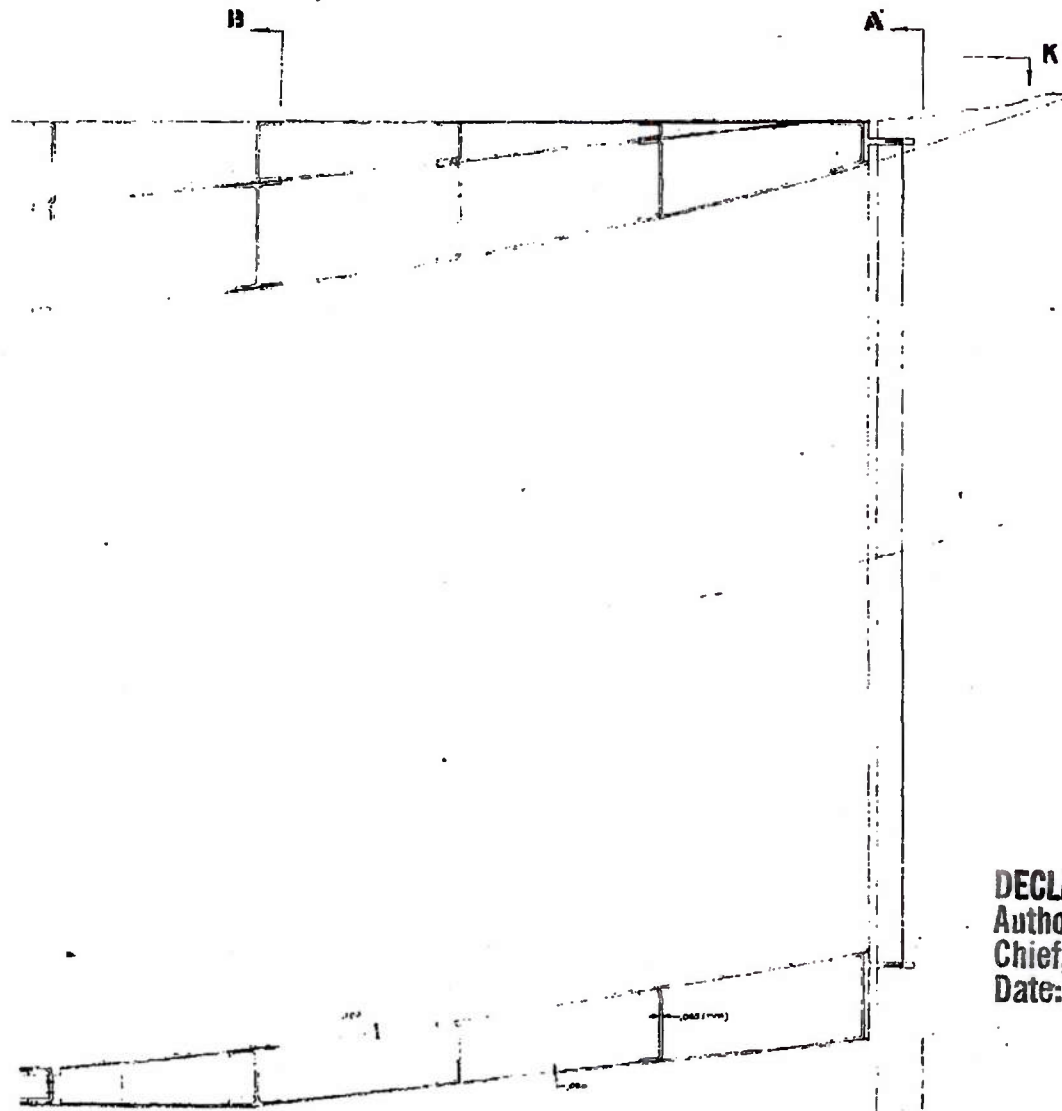
140 (WP)

UNTERKAMP (WP) (WP)

B

5

4



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE

B-B
SHEET 1

A-A
SHEET 2



- NOTES:
- 1. SHEET 2 OF 2 FOR B-B, A-A TO N.Y.
 - 2. T AND BUILT UP LINES TO BE IN ACCORDANCE
 - 3. EXHIBIT LINES & SHEET LINES
 - 4. APPROXIMATE DIMS TO BE USED
 - 5. SCALE OF MATERIAL
 - 6. OTHERWISE SPECIFIED

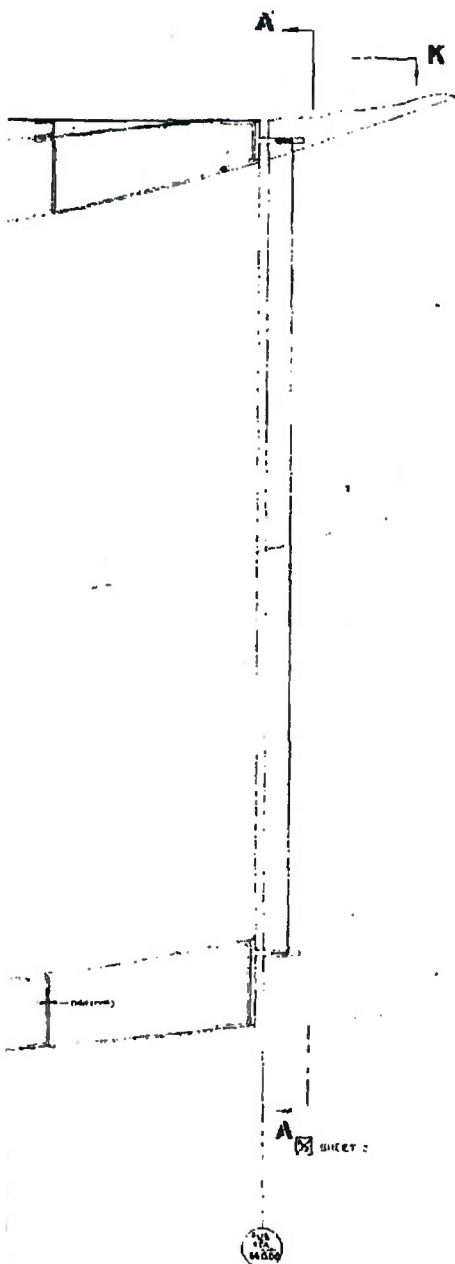
X81421

REPORT 6003

7

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 15. Design Layout, Isymmetric Inlet



SHEET 2

3 SEE SHEET 1 OF 3 FOR REG. A-A TO H-H
4 INLET AND DUCT SHALL BE IN ACCORDANCE
WITH AIR FORCE SPEC. 15-100
5 ALL STRUCTURAL PARTS TO BE MADE
FROM 2024-T3 ALUMINUM
NOTES: 1. ALL DIMENSIONS ARE IN INCHES
2. ALL DIMENSIONS ARE TO CENTER UNLESS OTHERWISE SPECIFIED

X81421

DESIGN LAYOUT TRIC INLET	X81421
--------------------------------	--------

~~SECRET RESTRICTED DATA~~

~~TOP SECRET AOT OF 1954~~

1

62-0000000 4017C 2018
A.4' 0000000

2000 2001 2002 2003 2004 2005 2006 2007 2008 2009 2010 2011 2012 2013 2014 2015 2016 2017 2018 2019 2020 2021 2022 2023 2024 2025 2026 2027 2028 2029 2030 2031 2032 2033 2034 2035 2036 2037 2038 2039 2040 2041 2042 2043 2044 2045 2046 2047 2048 2049 2050 2051 2052 2053 2054 2055 2056 2057 2058 2059 2060 2061 2062 2063 2064 2065 2066 2067 2068 2069 2070 2071 2072 2073 2074 2075 2076 2077 2078 2079 2080 2081 2082 2083 2084 2085 2086 2087 2088 2089 2090 2091 2092 2093 2094 2095 2096 2097 2098 2099 2100 2101 2102 2103 2104 2105 2106 2107 2108 2109 2110 2111 2112 2113 2114 2115 2116 2117 2118 2119 2120 2121 2122 2123 2124 2125 2126 2127 2128 2129 2130 2131 2132 2133 2134 2135 2136 2137 2138 2139 2140 2141 2142 2143 2144 2145 2146 2147 2148 2149 2150 2151 2152 2153 2154 2155 2156 2157 2158 2159 2160 2161 2162 2163 2164 2165 2166 2167 2168 2169 2170 2171 2172 2173 2174 2175 2176 2177 2178 2179 2180 2181 2182 2183 2184 2185 2186 2187 2188 2189 2190 2191 2192 2193 2194 2195 2196 2197 2198 2199 2200 2201 2202 2203 2204 2205 2206 2207 2208 2209 2210 2211 2212 2213 2214 2215 2216 2217 2218 2219 2220 2221 2222 2223 2224 2225 2226 2227 2228 2229 2230 2231 2232 2233 2234 2235 2236 2237 2238 2239 2240 2241 2242 2243 2244 2245 2246 2247 2248 2249 2250 2251 2252 2253 2254 2255 2256 2257 2258 2259 2260 2261 2262 2263 2264 2265 2266 2267 2268 2269 2270 2271 2272 2273 2274 2275 2276 2277 2278 2279 2280 2281 2282 2283 2284 2285 2286 2287 2288 2289 2290 2291 2292 2293 2294 2295 2296 2297 2298 2299 2300 2301 2302 2303 2304 2305 2306 2307 2308 2309 2310 2311 2312 2313 2314 2315 2316 2317 2318 2319 2320 2321 2322 2323 2324 2325 2326 2327 2328 2329 2330 2331 2332 2333 2334 2335 2336 2337 2338 2339 2340 2341 2342 2343 2344 2345 2346 2347 2348 2349 2350 2351 2352 2353 2354 2355 2356 2357 2358 2359 2360 2361 2362 2363 2364 2365 2366 2367 2368 2369 2370 2371 2372 2373 2374 2375 2376 2377 2378 2379 2380 2381 2382 2383 2384 2385 2386 2387 2388 2389 2390 2391 2392 2393 2394 2395 2396 2397 2398 2399 2400 2401 2402 2403 2404 2405 2406 2407 2408 2409 2410 2411 2412 2413 2414 2415 2416 2417 2418 2419 2420 2421 2422 2423 2424 2425 2426 2427 2428 2429 2430 2431 2432 2433 2434 2435 2436 2437 2438 2439 2440 2441 2442 2443 2444 2445 2446 2447 2448 2449 2450 2451 2452 2453 2454 2455 2456 2457 2458 2459 2460 2461 2462 2463 2464 2465 2466 2467 2468 2469 2470 2471 2472 2473 2474 2475 2476 2477 2478 2479 2480 2481 2482 2483 2484 2485 2486 2487 2488 2489 2490 2491 2492 2493 2494 2495 2496 2497 2498 2499 2500 2501 2502 2503 2504 2505 2506 2507 2508 2509 2510 2511 2512 2513 2514 2515 2516 2517 2518 2519 2520 2521 2522 2523 2524 2525 2526 2527 2528 2529 2530 2531 2532 2533 2534 2535 2536 2537 2538 2539 2540 2541 2542 2543 2544 2545 2546 2547 2548 2549 2550 2551 2552 2553 2554 2555 2556 2557 2558 2559 2560 2561 2562 2563 2564 2565 2566 2567 2568 2569 2570 2571 2572 2573 2574 2575 2576 2577 2578 2579 2580 2581 2582 2583 2584 2585 2586 2587 2588 2589 2590 2591 2592 2593 2594 2595 2596 2597 2598 2599 2600 2601 2602 2603 2604 2605 2606 2607 2608 2609 2610 2611 2612 2613 2614 2615 2616 2617 2618 2619 2620 2621 2622 2623 2624 2625 2626 2627 2628 2629 2630 2631 2632 2633 2634 2635 2636 2637 2638 2639 2640 2641 2642 2643 2644 2645 2646 2647 2648 2649 2650 2651 2652 2653 2654 2655 2656 2657 2658 2659 2660 2661 2662 2663 2664 2665 2666 2667 2668 2669 2670 2671 2672 2673 2674 2675 2676 2677 2678 2679 2680 2681 2682 2683 2684 2685 2686 2687 2688 2689 2690 2691 2692 2693 2694 2695 2696 2697 2698 2699 2700 2701 2702 2703 2704 2705 2706 2707 2708 2709 2710 2711 2712 2713 2714 2715 2716 2717 2718 2719 2720 2721 2722 2723 2724 2725 2726 2727 2728 2729 2730 2731 2732 2733 2734 2735 2736 2737 2738 2739 2740 2741 2742 2743 2744 2745 2746 2747 2748 2749 2750 2751 2752 2753 2754 2755 2756 2757 2758 2759 2760 2761 2762 2763 2764 2765 2766 2767 2768 2769 2770 2771 2772 2773 2774 2775 2776 2777 2778 2779 2780 2781 2782 2783 2784 2785 2786 2787 2788 2789 2790 2791 2792 2793 2794 2795 2796 2797 2798 2799 2800 2801 2802 2803 2804 2805 2806 2807 2808 2809 2810 2811 2812 2813 2814 2815 2816 2817 2818

244 JANUARY 1965

4441 CA 2422 2-24-16/86

INTERVIEW PAGE

SECTION H-H ☒ SHEET:

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

12

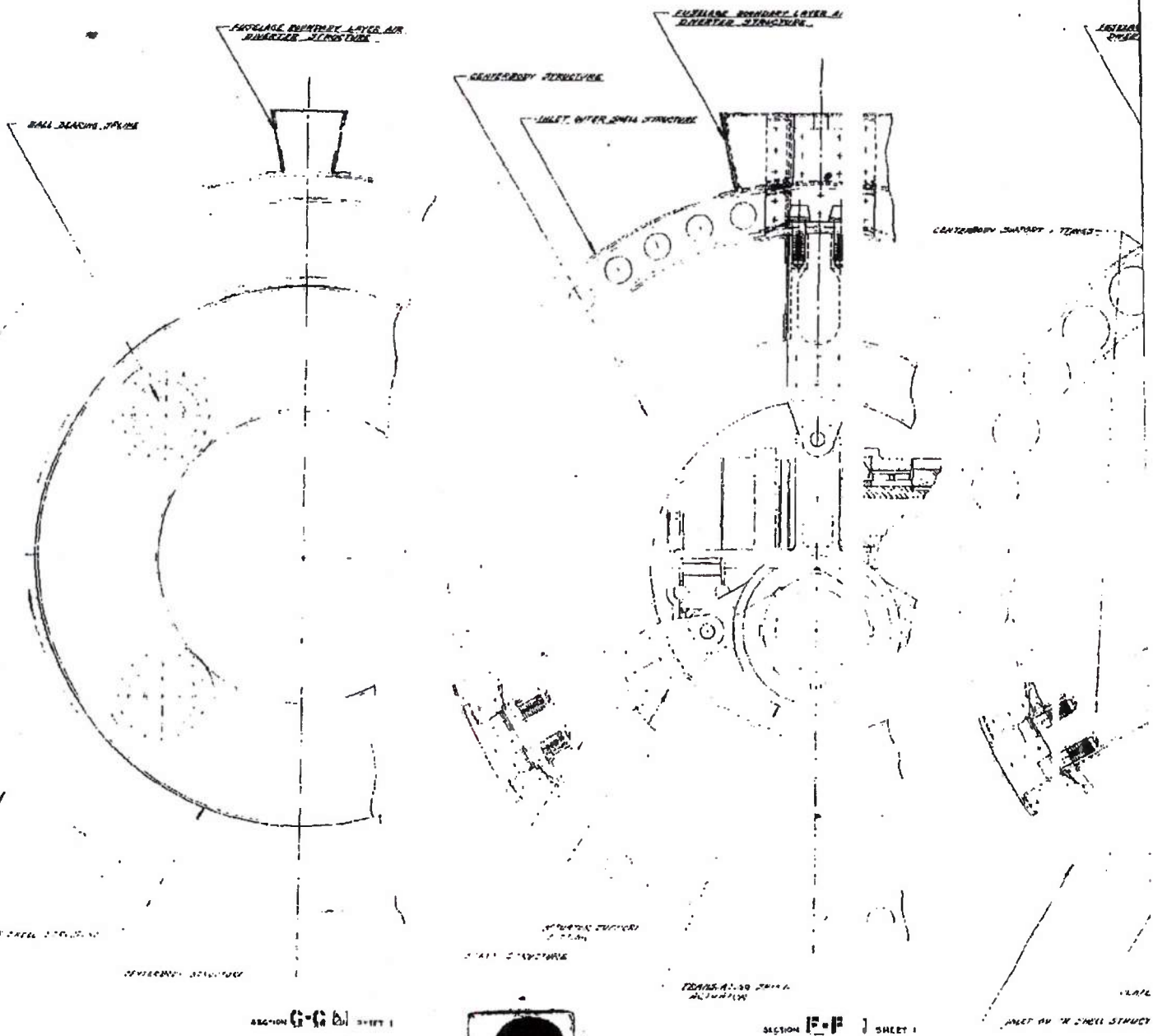
T

10

Authority: EO 13526

Chief, Records & Declass Div, WHS

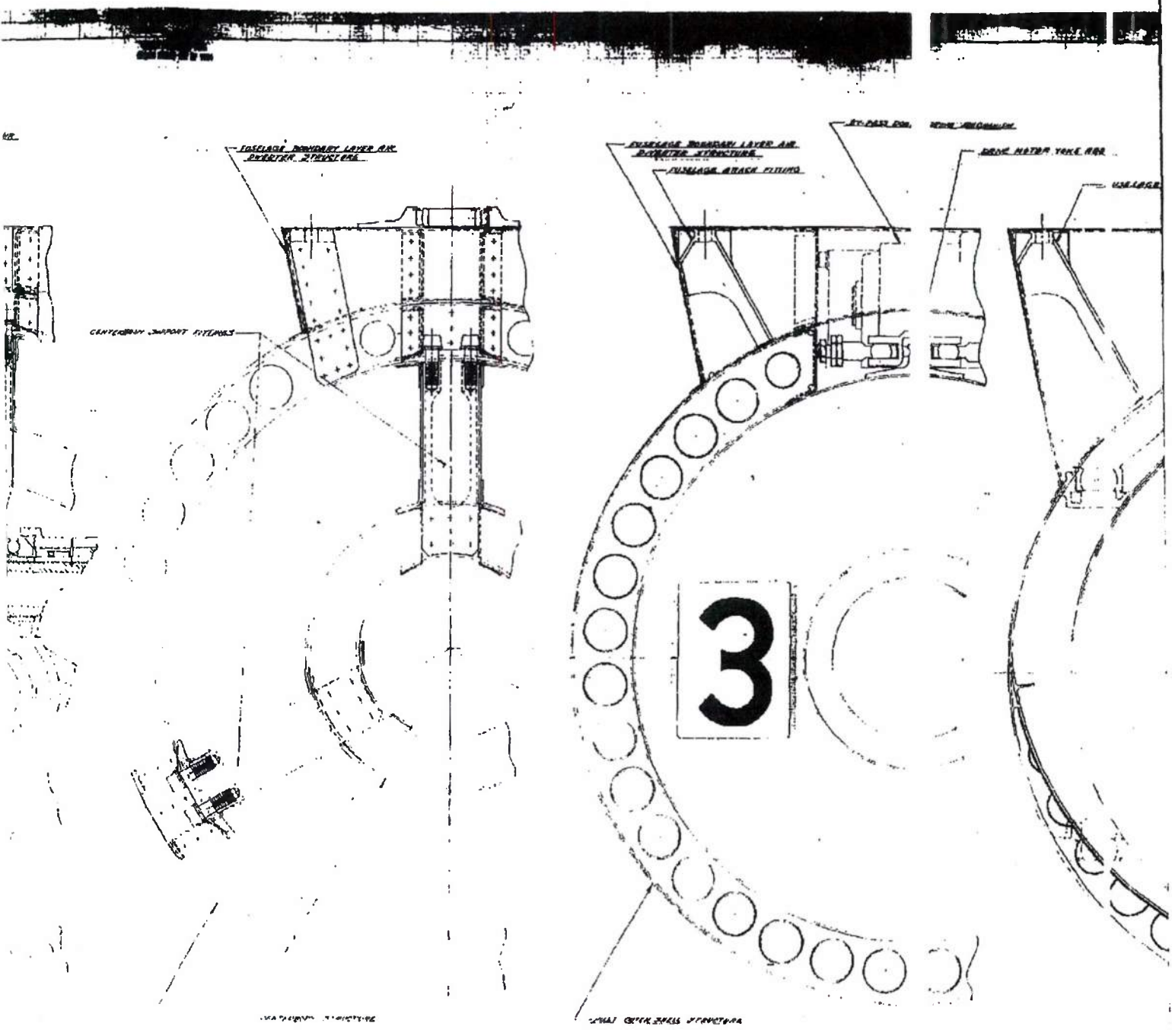
Date: OCT 02 2015



2

X 81421

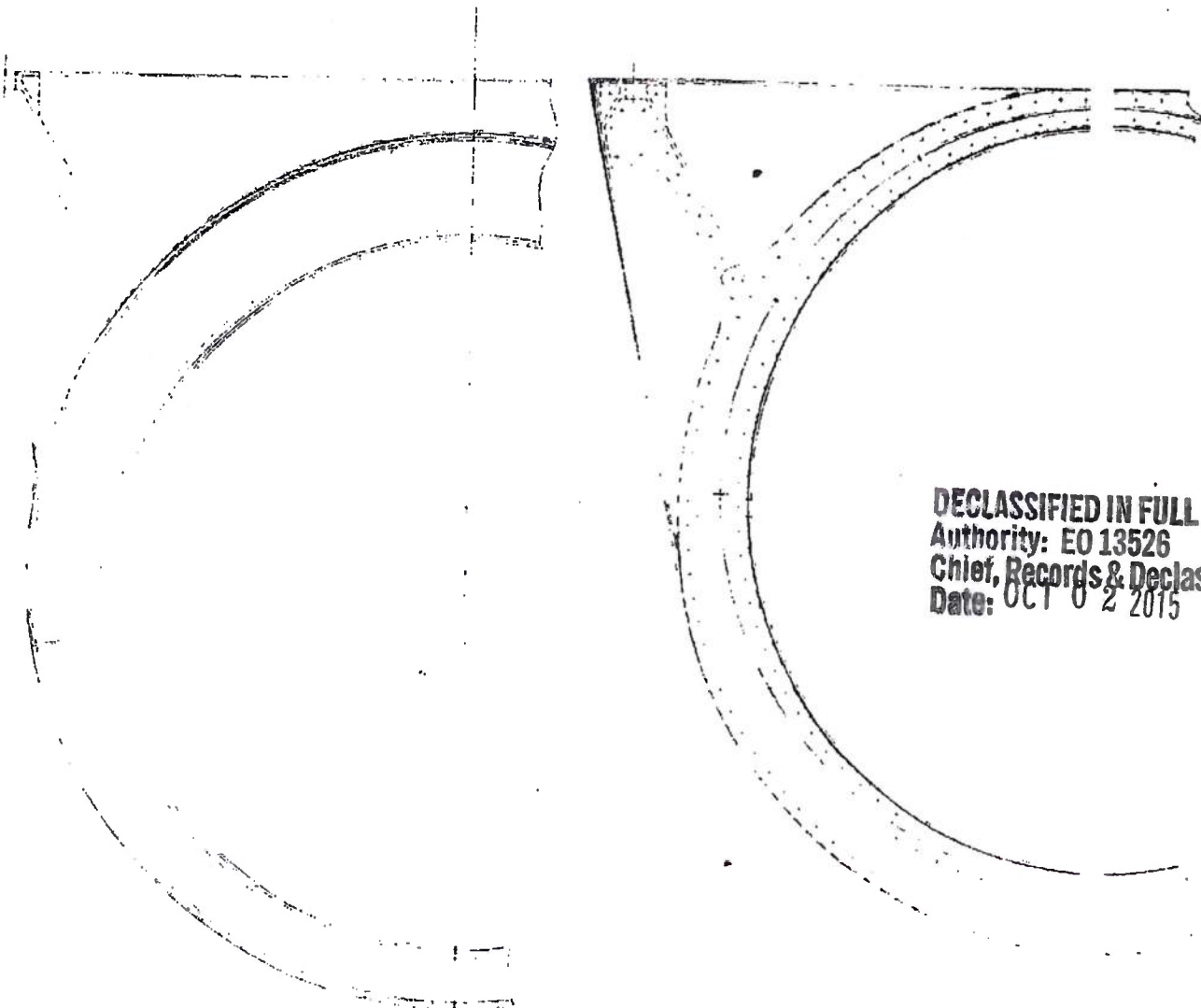
DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015



SECTION 15-15 SHEET 1

SECTION 16-16 SHEET 1

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

5

FIGU

D-B 50 SHEET 1

SCALE A-A 2 SHEET 2

NOTES: SEE SHEET 1 FOR DETAIL. DIMENSIONS

FIGURE 16. Design Layout, Asymmetric Inlet

A close-up photograph of a document, likely a passport or identification card. The document is white with black text. At the top, there is a stamp that reads "DESIGN" in large, bold, capital letters. Below this, the words "AXIS YANETI" are visible. The document also features a small, rectangular stamp on the left side, which appears to be a date stamp. The overall image is somewhat blurry and has a high-contrast, grainy appearance.

A close-up photograph of a document, likely a check or receipt. The word "Payable" is visible in a stylized font. Below it, the number "01421" is printed. The document is slightly tilted and has some other faint markings and text.

SECRET RESTRICTED DATA
ATOMIC ENERGY ACT OF 1954

CS
140,340

1

*SEPARATING INLET STEEL

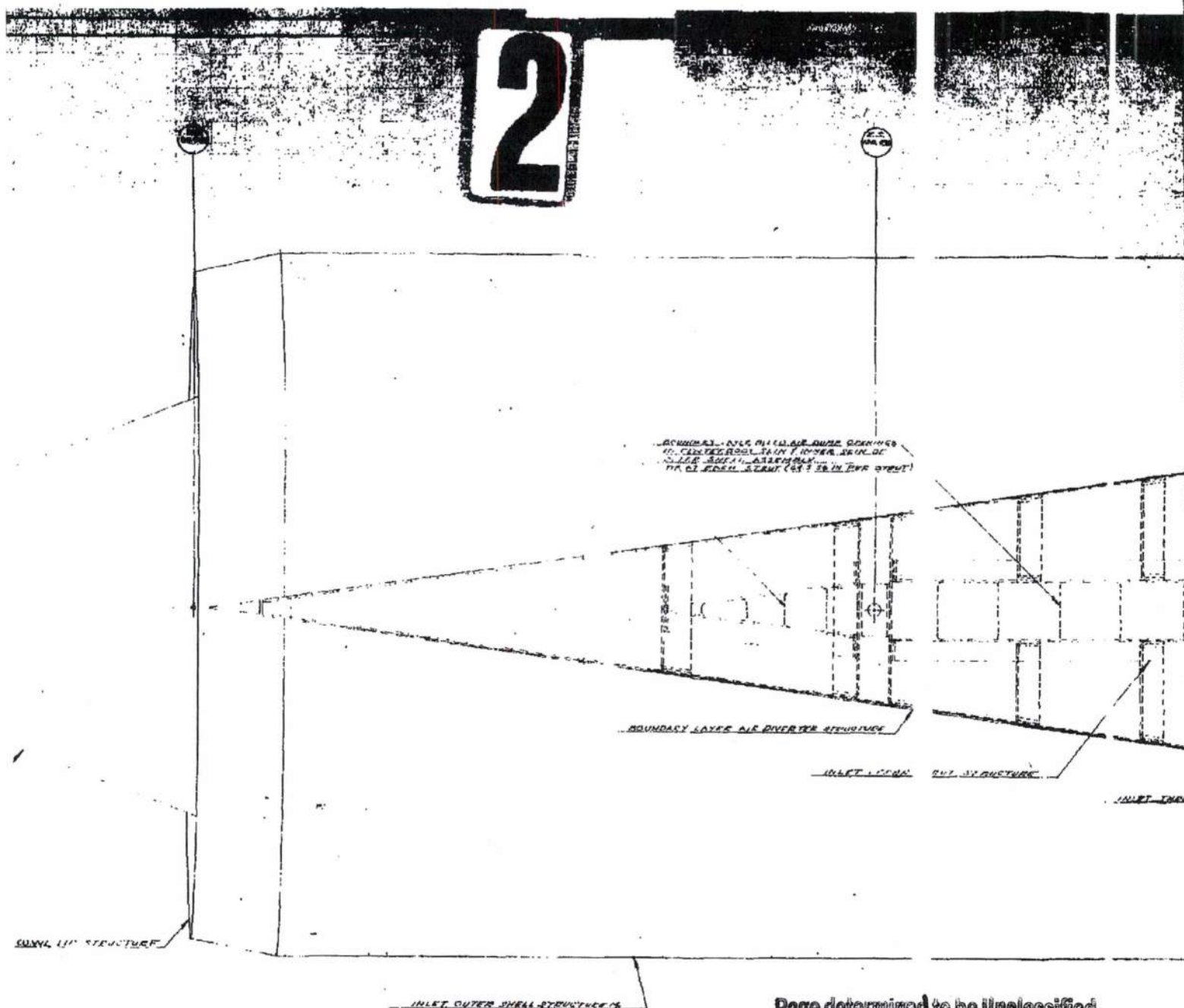
CONV. LIP STRUCTURE

SECRET RESTRICTED DATA
ATOMIC ENERGY ACT OF 1954

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

X81421 3

2



**Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015**

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNDER OPERATIONS
177th STATION, 22
12th ST. (CROSS STREET)

PERSONALITY

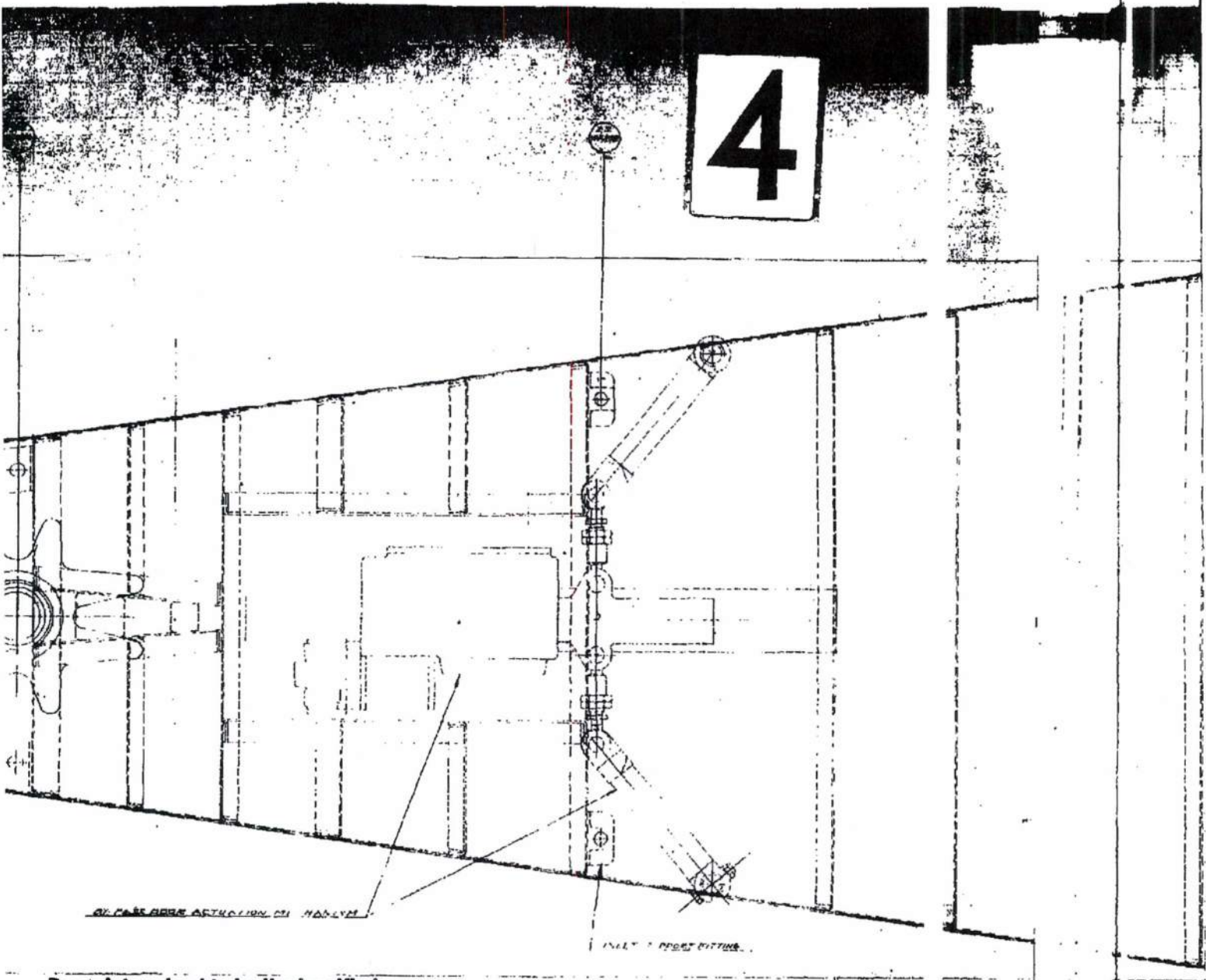
INLET - FAN SPEED MEASUREMENT

INLET THRUOUT MAINT

AT CALIFORNIA ACTUATION 24

VIEW E-1 51

4



Page determined to be Unclassified
Reviewed Chief, RDD, WHS
LAW EO 13526, Section 3.5
Date: OCT 02 2015

VIEW K-K
SECTION K-K

5

100-100000

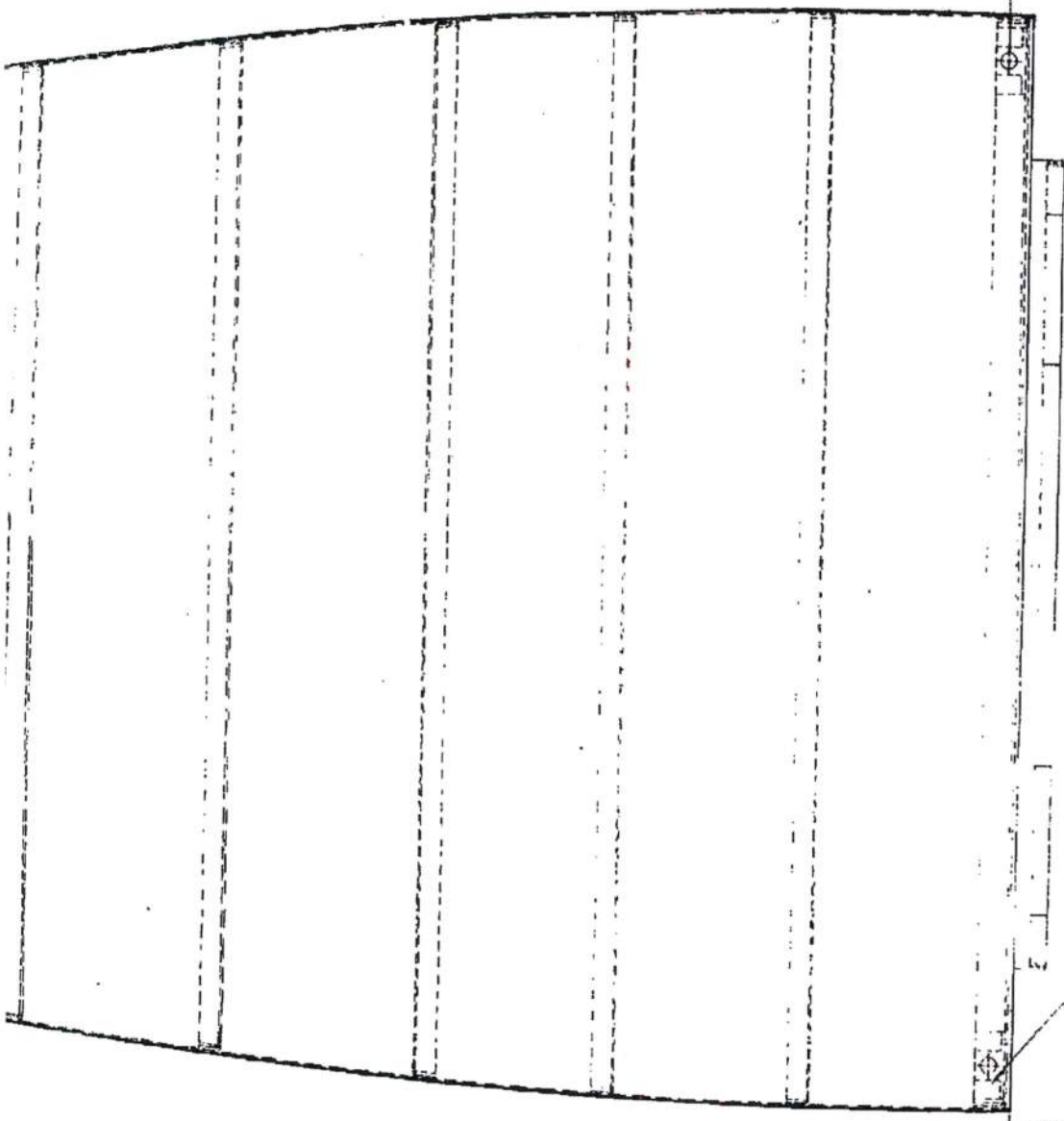
100-100000

100-100000

100-100000

X 814213

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



[REDACTED]

DE-5
AXIS
1944

EPORT 6003

7

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 17. Design Layout, xisymmetric Inlet

DESIGN	AXISYMMETRIC	INLET
DEVELOPER		
Marquardt		81421
DRAWING		Sheet 3 of 3

CONFIDENTIAL

ASD-TDR-63-277, Vol. IV

REPORT 6003

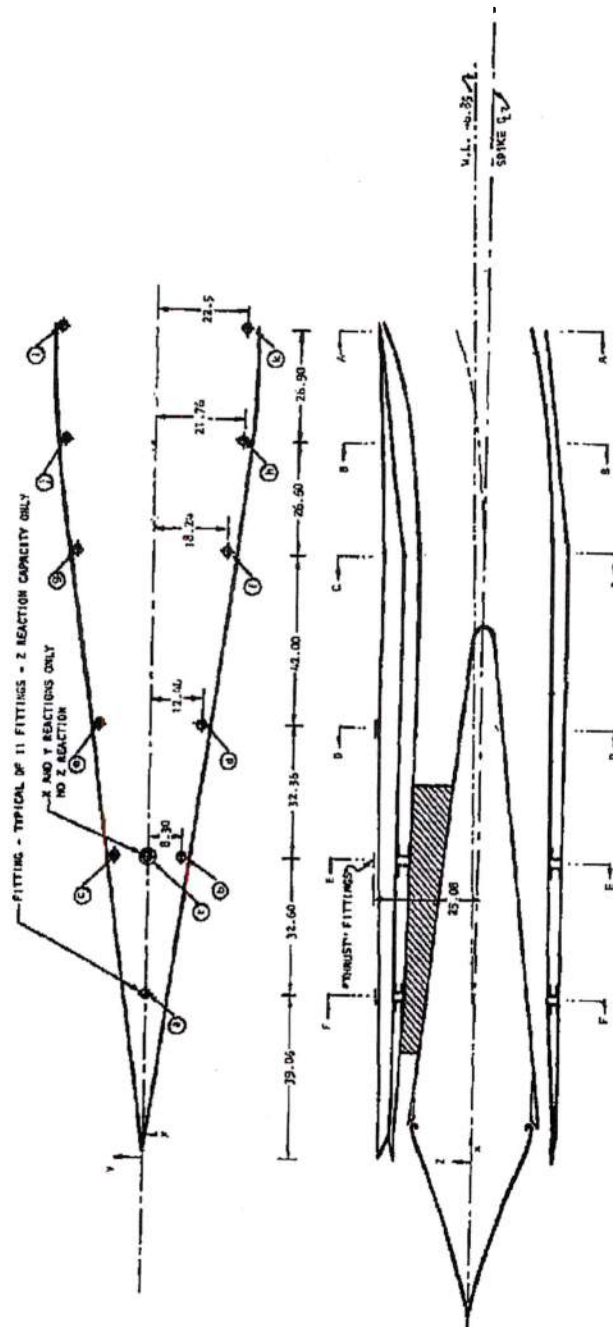


FIGURE 18. Sketch of Inlet Assembly for Inlet Structural Analysis

CONFIDENTIAL

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

6003

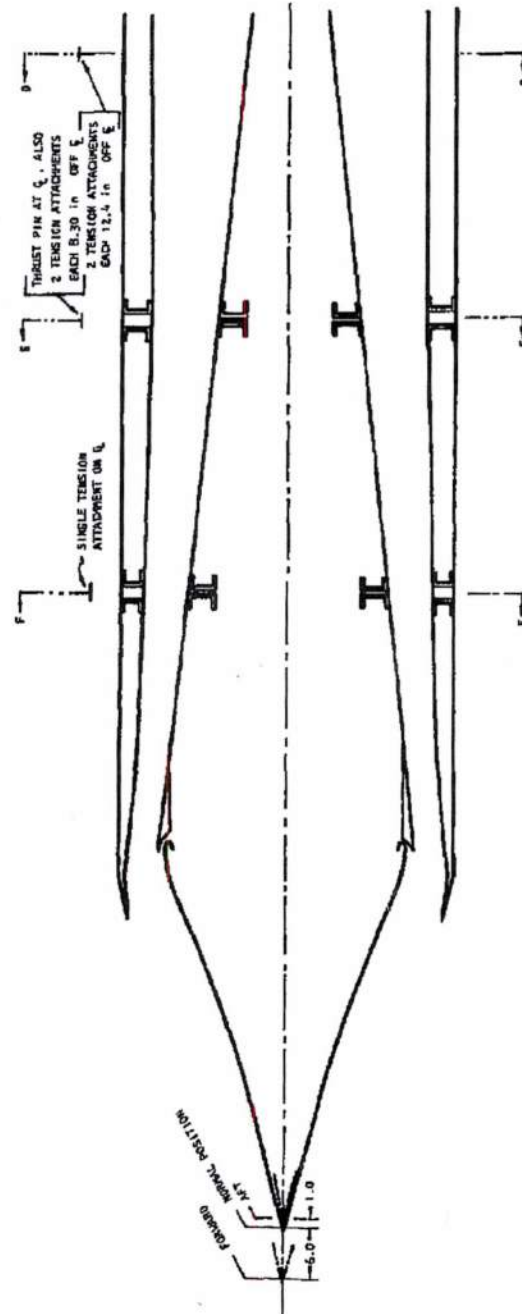


FIGURE 19. Sketch of Spike and Innerbody for Inlet Structural Analysis

~~CONFIDENTIAL~~

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

ASD-TDR-63-277, Vol. IV

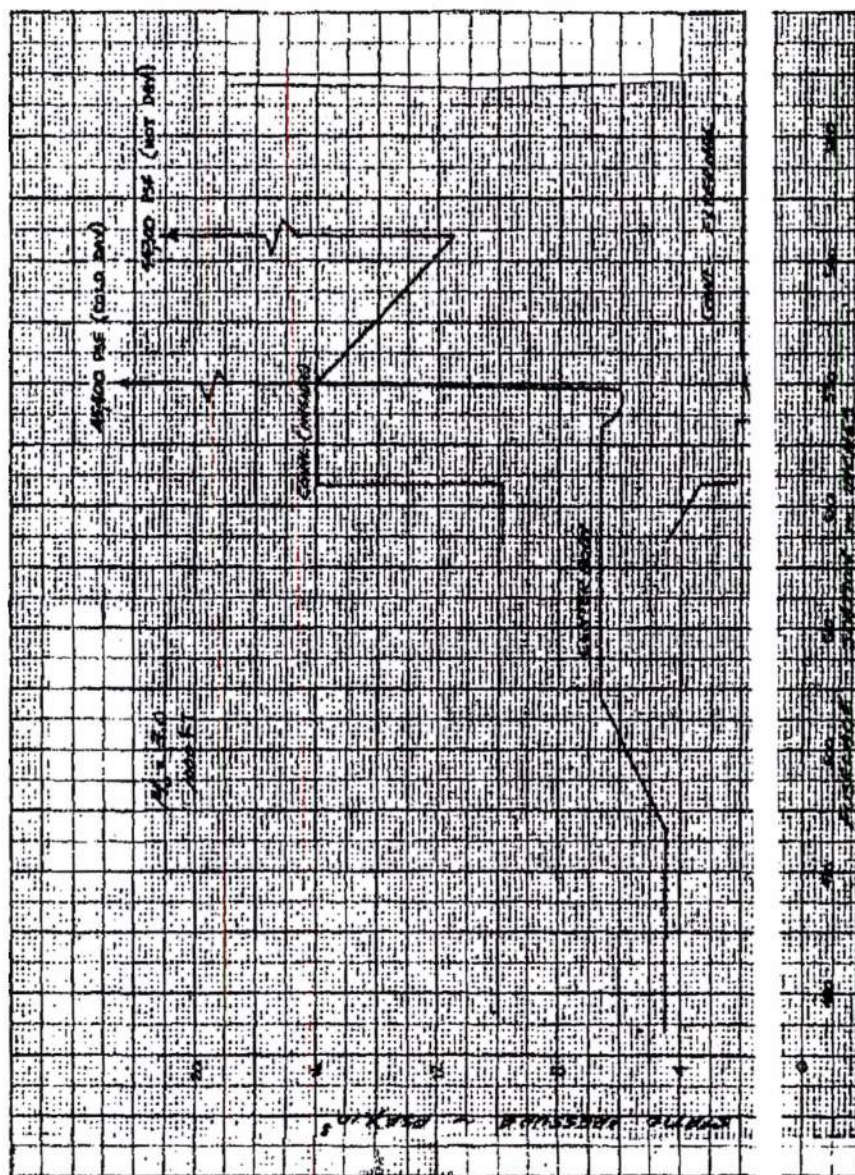


FIGURE 20. MA50-XCA Inlet-Diffuser Static Pressure

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E615

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

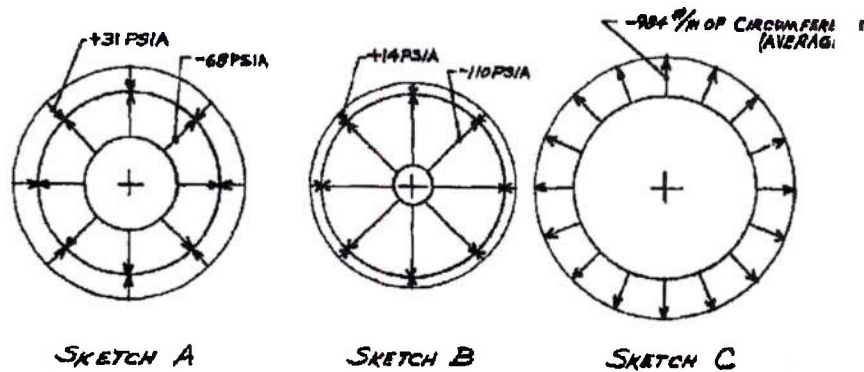


FIGURE 21. Varying Pressures on Inside and Outside Lip and Cowl Surfaces

UNCLASSIFIED

22J65

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date:

OCT 02 2015

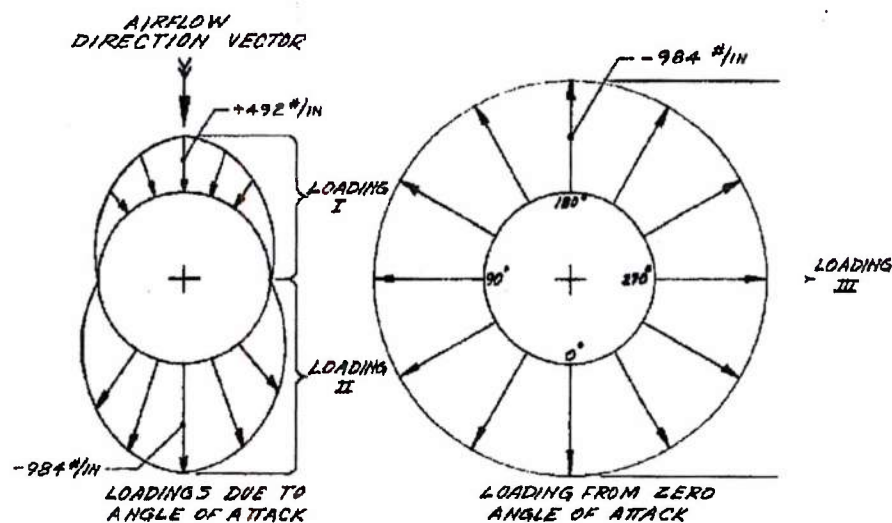


FIGURE 22. Three Loadings for Analysis Structure

MAC AED

UNCLASSIFIED

22J66

- 173 -

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

RPCH

6003

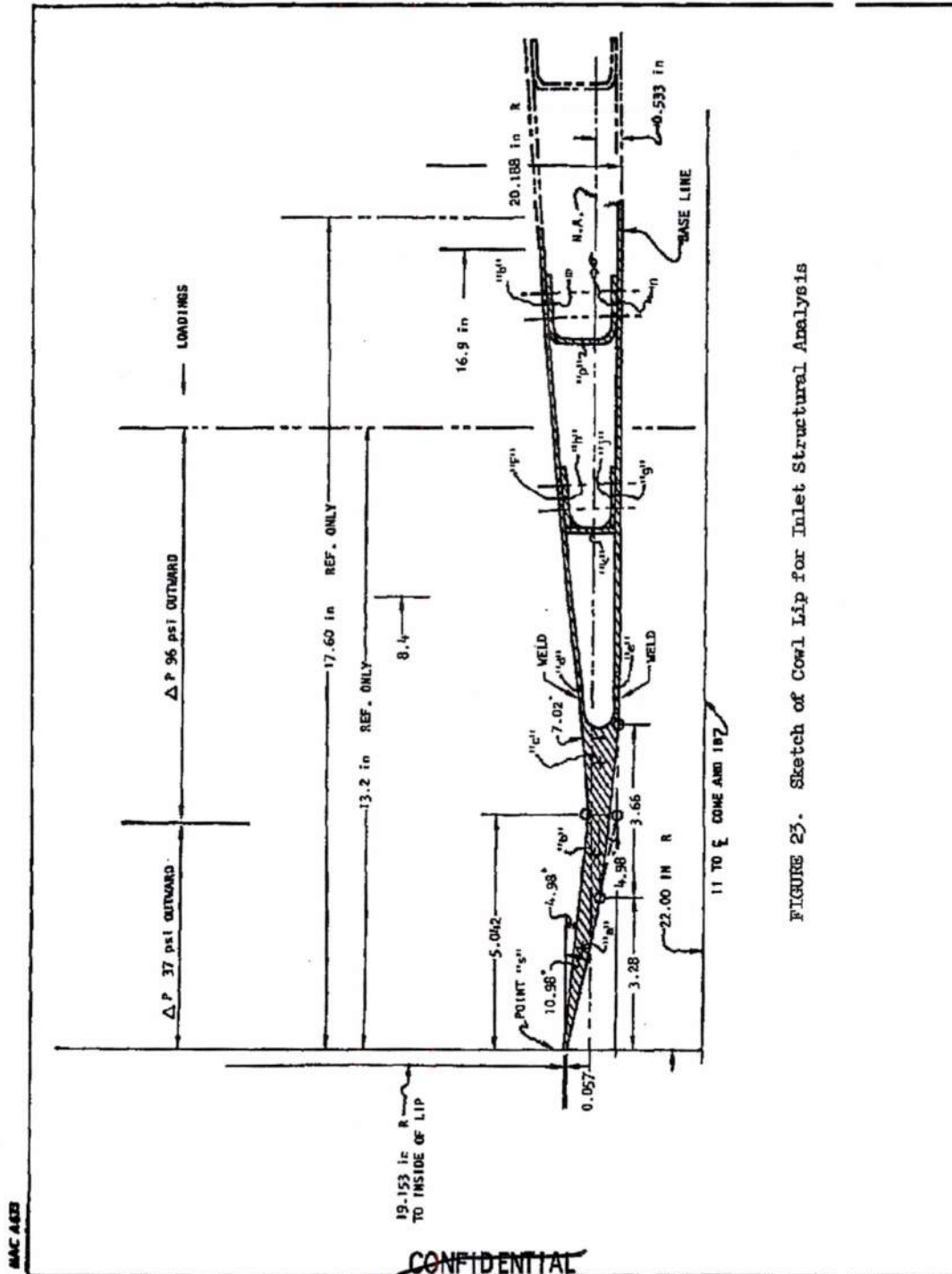


FIGURE 23. Sketch of Cowl Lip for Inlet Structural Analysis

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

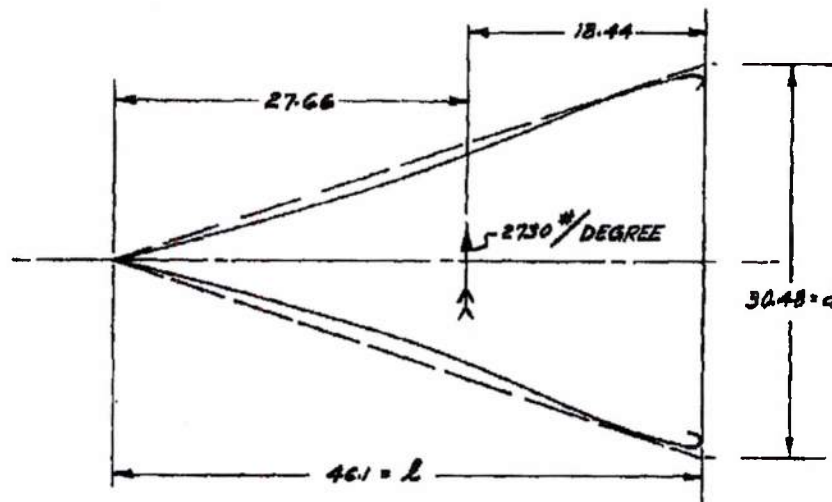


FIGURE 24. Spike Geometry

MAC 1603

UNCLASSIFIED

22J67

- 175 -

UNCLASSIFIED

ASD-TOR-63-277, Vol. IV

DRY 6003

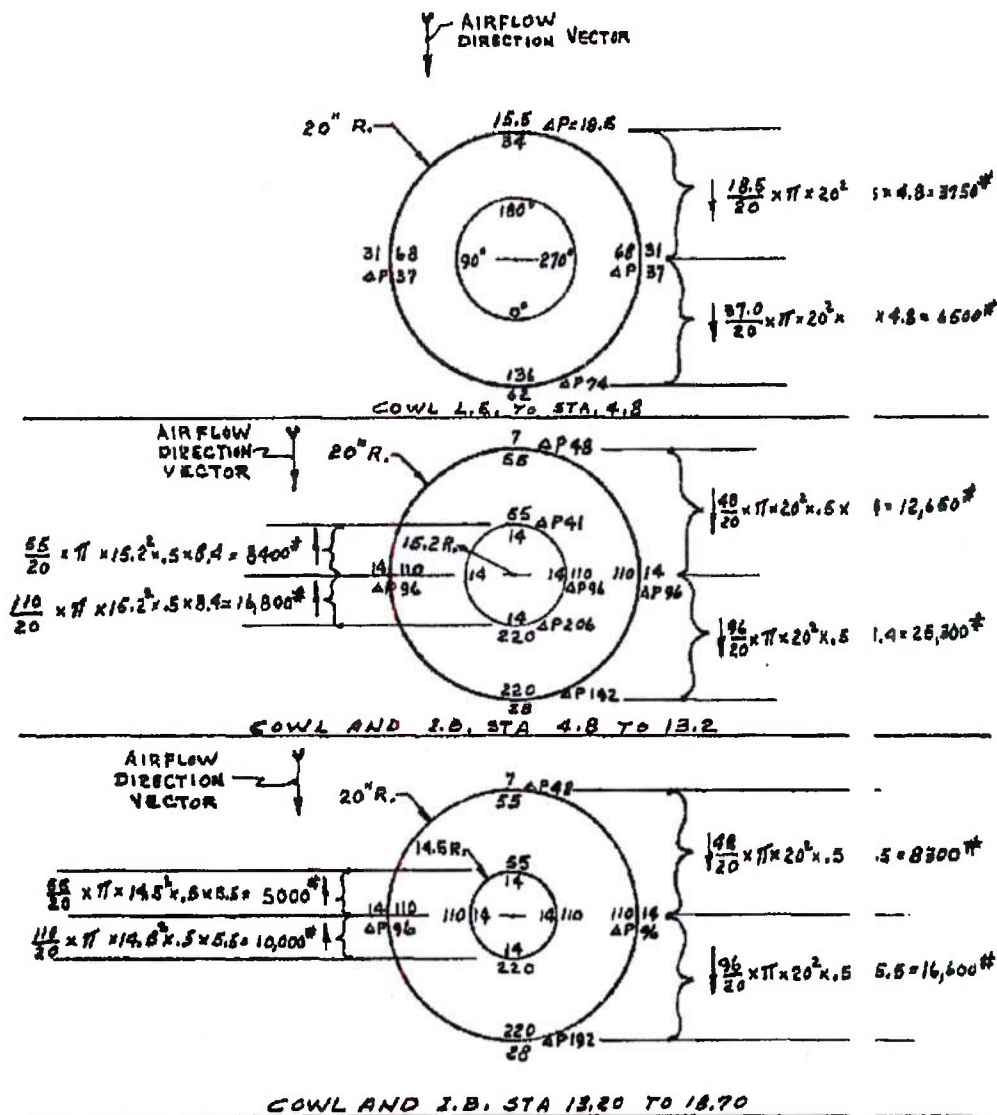


FIGURE 25. Cowl and Innerbody Unsymmetrical Loadings

UNCLASSIFIED

~~CONFIDENTIAL~~

REPORT 6003



- 177 -

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

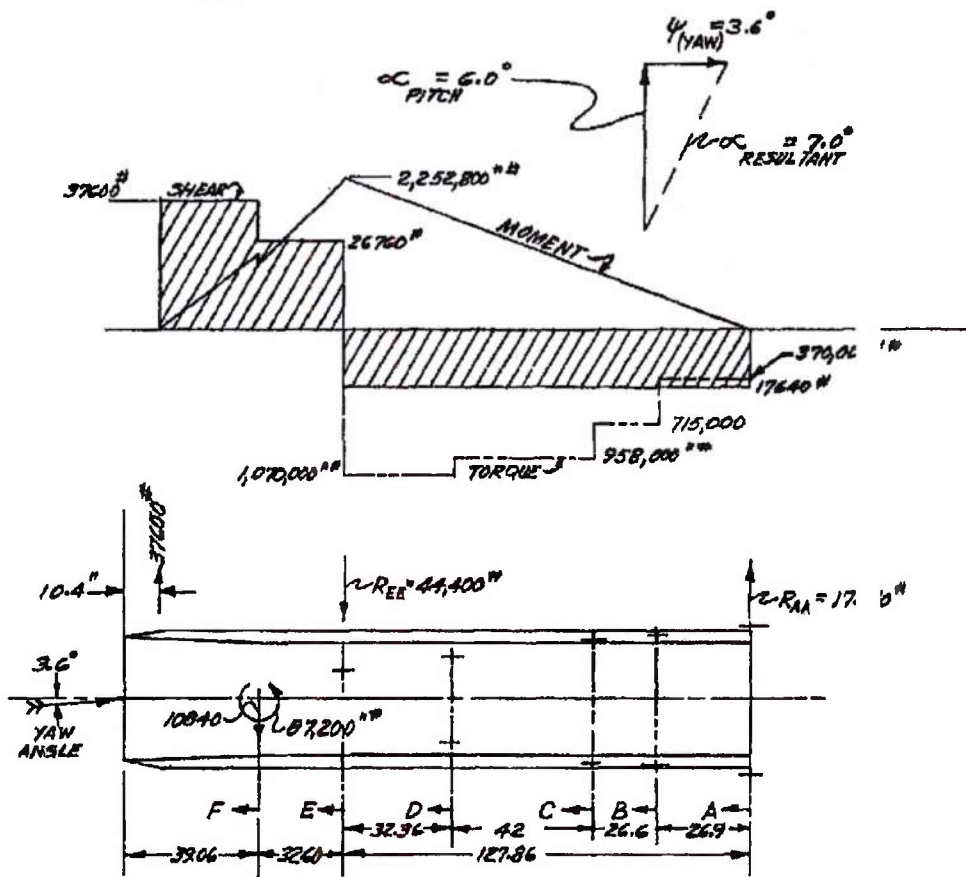


FIGURE 27. Unsymmetrical Air Load Distribution

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

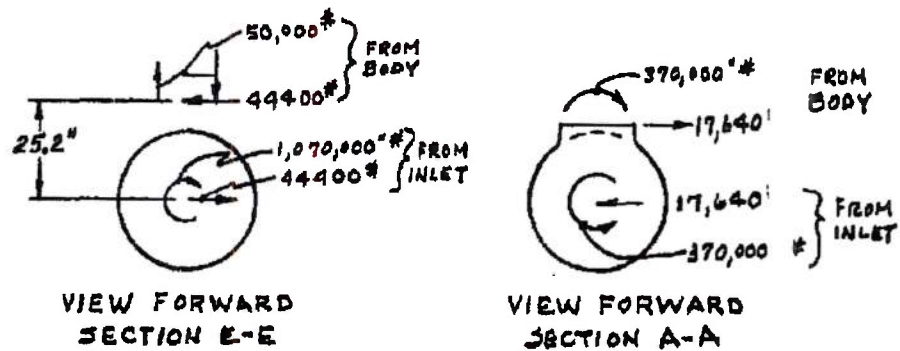
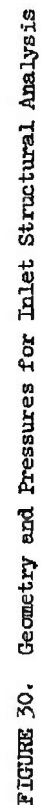


FIGURE 28. Shears, Moments, and Torques from Y Direction

UNCLASSIFIED

~~CONFIDENTIAL~~

REPORT 6003



UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

IT 6003

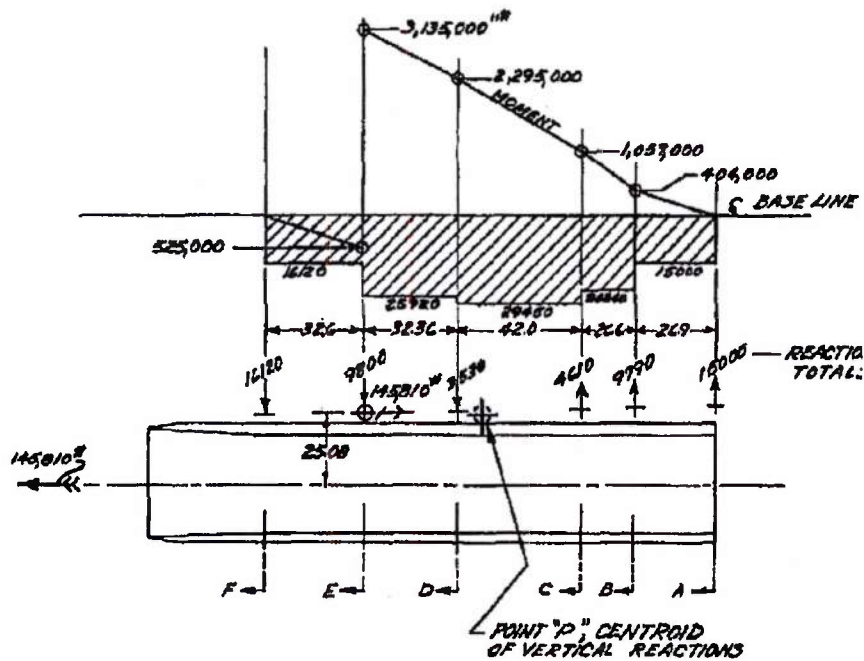


FIGURE 31. Derived Load Distribution on Fittings

UNCLASSIFIED

22J72

- 182 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

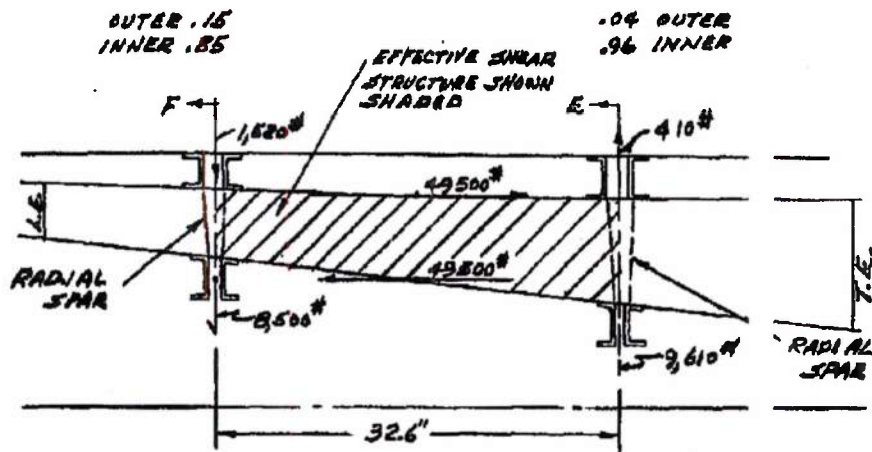


FIGURE 32. Longitudinal Section Through Typical Strut

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

PORT 6003

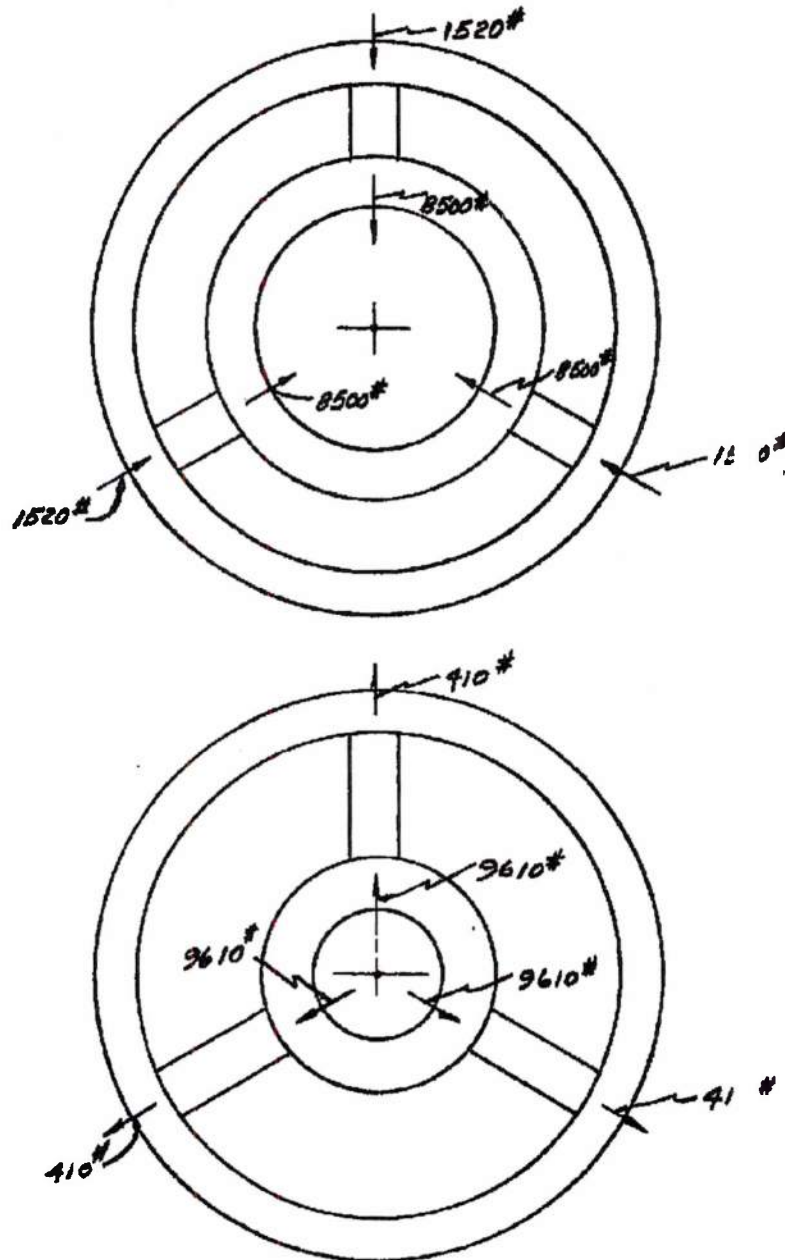


FIGURE 33. Load on Struts from Supporting Rings

UNCLASSIFIED

22J74

- 184 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

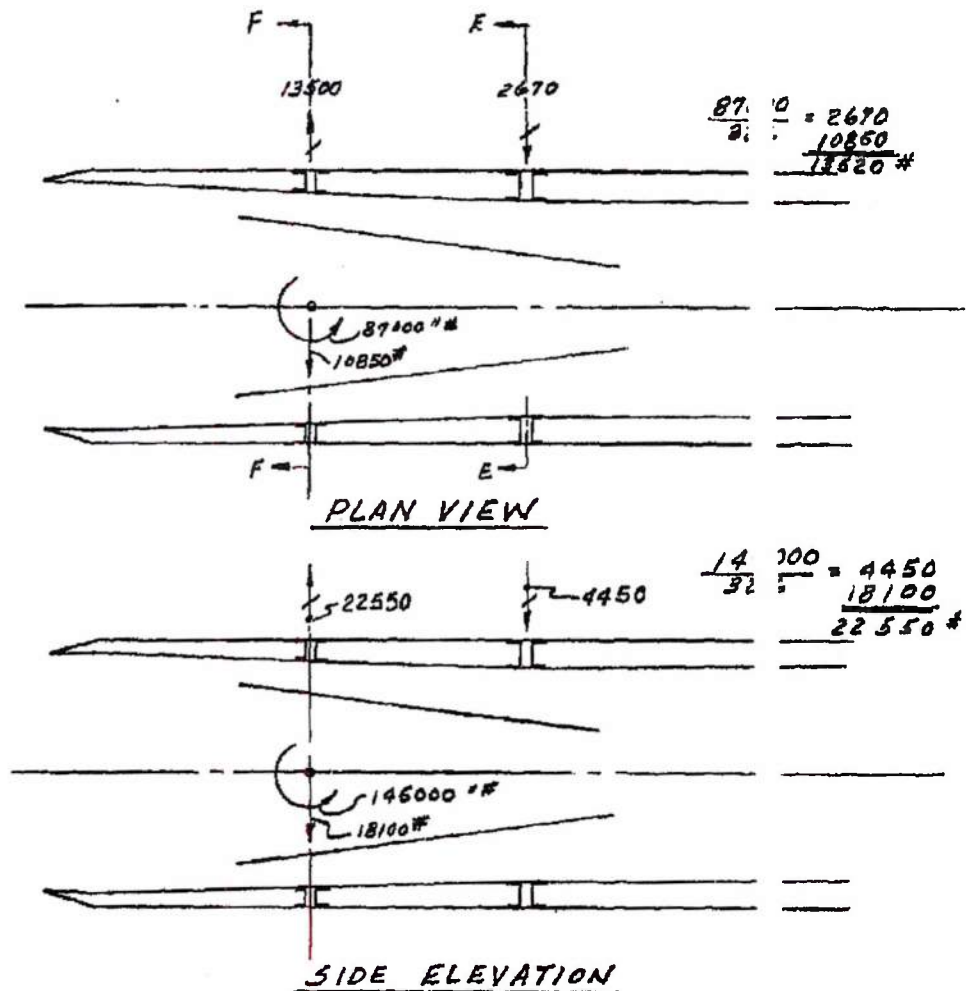


FIGURE 34. Loads on Strut Spars from Unsymmetrical Ls

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPO

6003

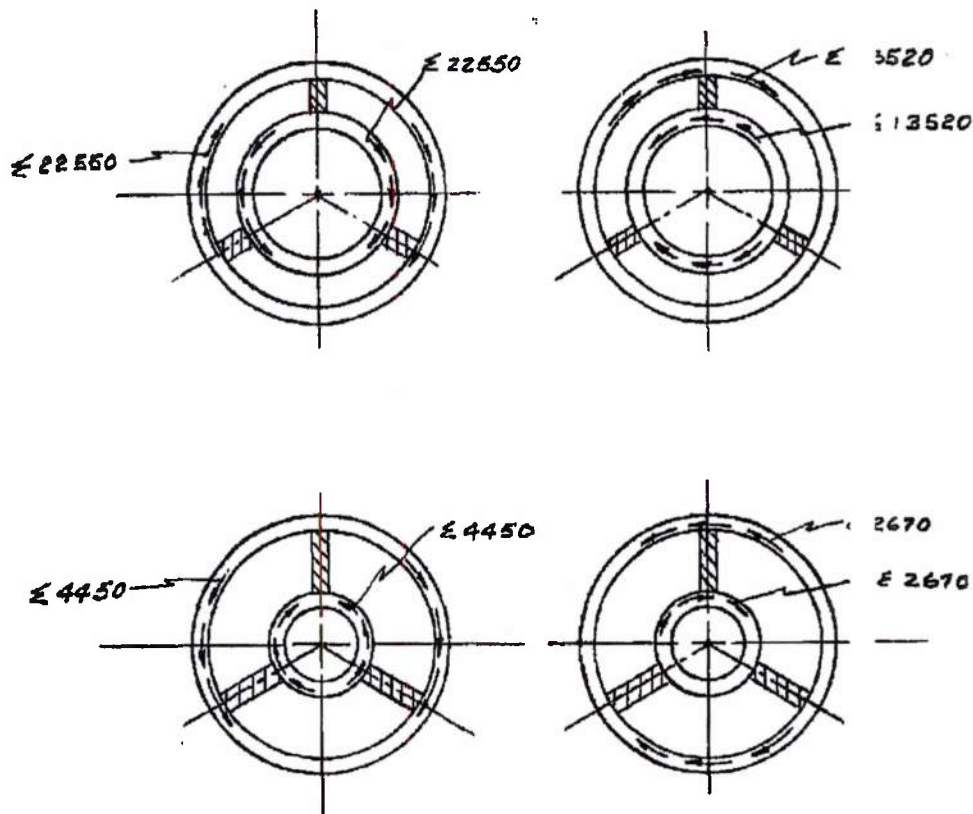


FIGURE 35. Vertical and Horizontal Loads on Strut Spars

UNCLASSIFIED

22J76

- 186 -

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV



1. MINIMAL POSITION GAP SHOWN. GAP CAN VARY FROM 1.0 INCH LESS TO 6.0 INCHES MORE AS SPIKE MOVES.
2. SHEAR ACROSS TUBE AT THIS GAP, $18,100 \div 4 = 4525$ LBS.
3. LOCUS OF POINT "A" TRANSFERRED FOR REFERENCE PURPOSES.
4. PRIMARY MOMENT OF $168,500 \div 4$ MUST BE CARRIED BY EACH TUBE IN ADDITION TO SECONDARY MOMENTS.

MAC 7400

~~CONFIDENTIAL~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

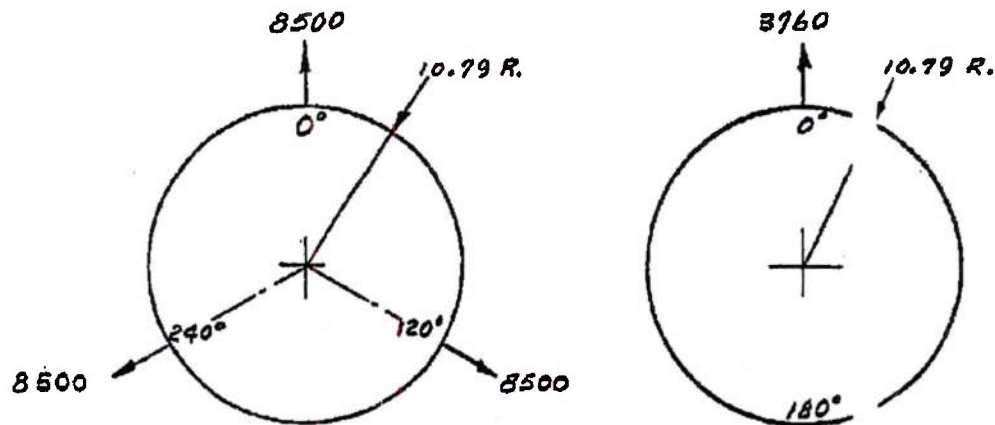


FIGURE 37. Innerbody Ring Loads

UNCLASSIFIED

22J77

- 188 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

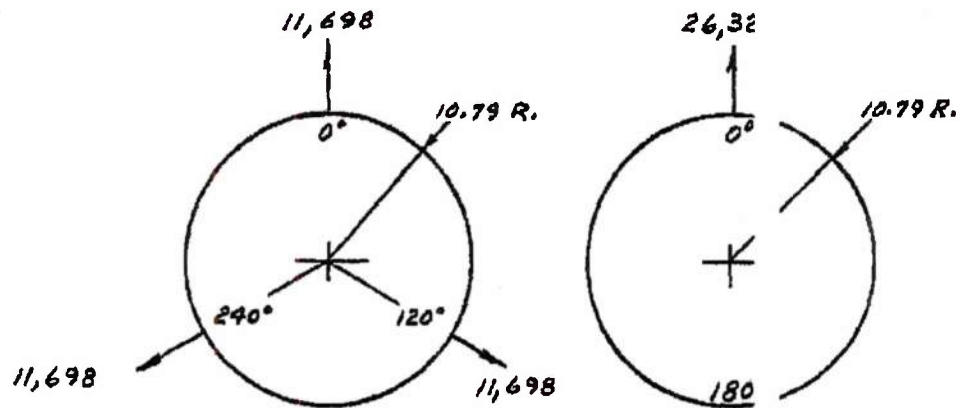


FIGURE 38. Symmetrical and Unsymmetrical Loadings

UNCLASSIFIED

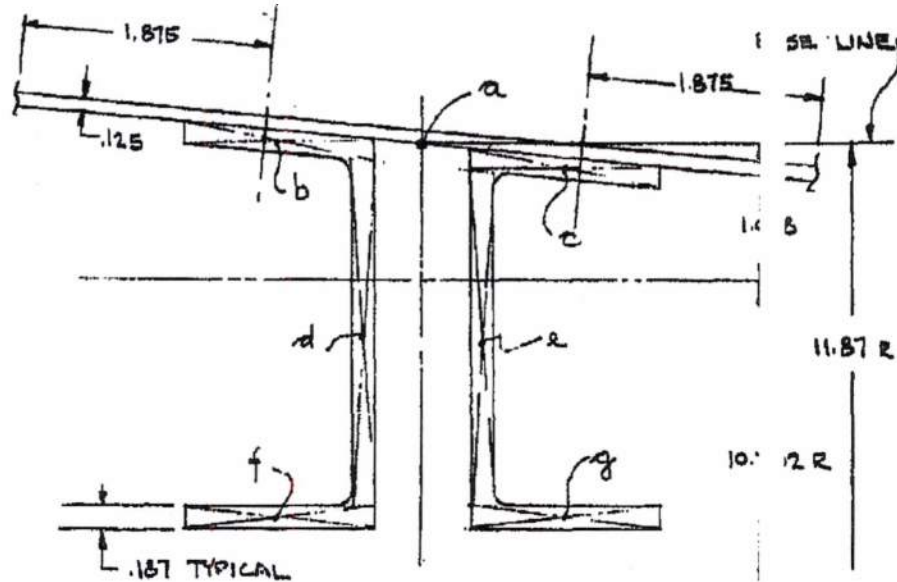
22J78

- 189 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

RT 6003



Item	SIZE & AREA, A In ²	Z	A · Z	$\bar{Z}$	A · $\bar{Z}$	A · $\bar{Z}^2$	I _o
a	.125 x 6.25 = .7810	-.05	-.0390	-1.128	-.8810	.9930	0
b	.18 x 1.50 = .2700	0	0	-1.078	-.2910	.3140	0
c	.18 x 1.50 = .2700	.20	.0540	-.878	-.2370	.2080	0
d	.18 x 2.70 = .4860	1.50	.7290	+.422	+.2030	.0865	.294
e	.18 x 2.60 = .4680	1.51	.7060	+.432	+.2022	.0875	.262
f	.18 x 1.50 = .2700	2.92	.7890	+1.842	+.4975	.9160	0
u	.18 x 1.50 = .2700	2.92	.7890	+1.842	+.4975	.9160	0
	2.8150	1.078	3.0280		-1.4090 +1.4022	3.5230 4.0790	.5560 1.61

I of section with holes is estimated as $.80 \times 4.079 = 3.27 \text{ in}^4$

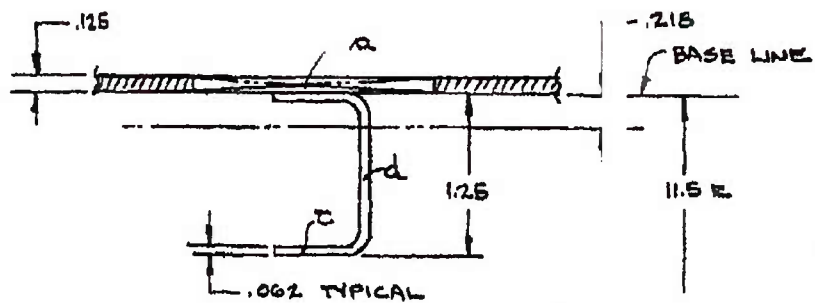
FIGURE 39. Ring Section Properties

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003



ITEM	SIZE	AREA	ARM Z	A·Z	To N.A.	A·Z	A·Z ²	I _o
a	.125 x 1.86	.2330	-.0625	-.01460	-.2805	-.0653	.018380	0
b	.062 x .75	.0465	+.0310	+.00144	-.1870	+.0087	.001624	0
c	.062 x .75	.0465	+1.1930	+.05530	+.9750	+.0434	.044206	0
d	.062 x 1.1	.0684	+.6120	+.04180	+.3940	+.02695	.010600	.00693
		.3944	.218	+.08614			.074804	.00693
							.08	34 in. ⁴

FIGURE 40. Ring Section Properties

UNCLASSIFIED

22J80

- 191 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

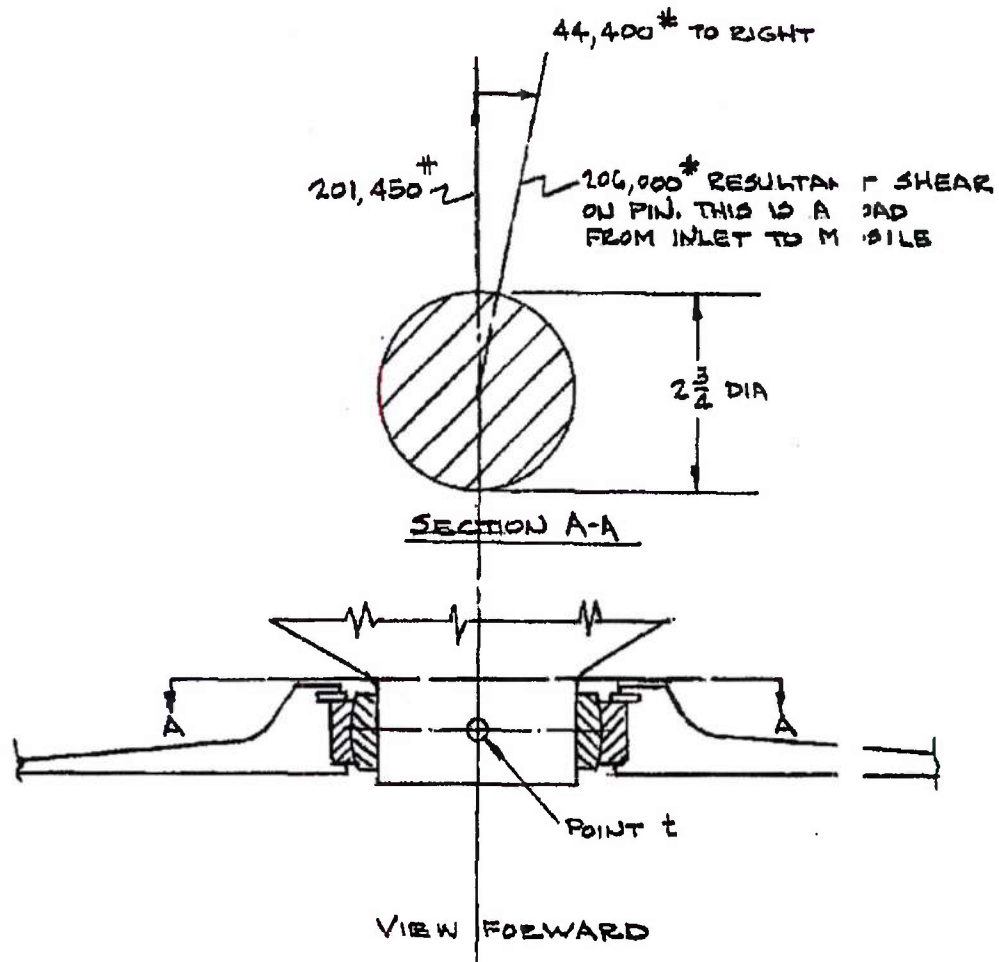


FIGURE 41. Thrust Fitting Loads

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

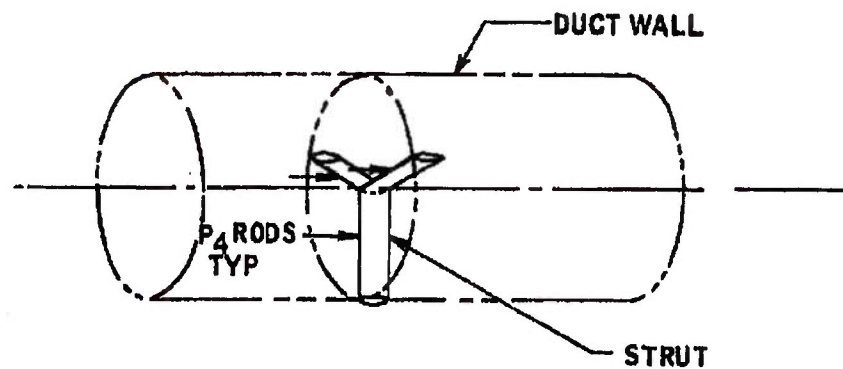


FIGURE 42. Struts

MAC AGS

UNCLASSIFIED

22J82

- 193 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

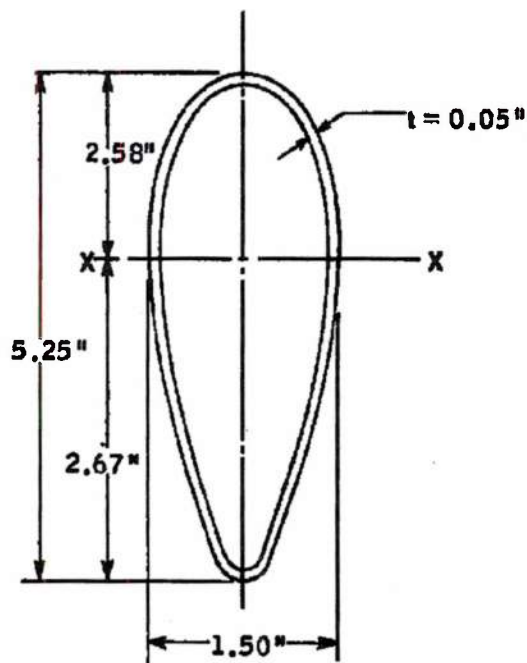


FIGURE 43. Strut Cross Section

UNCLASSIFIED

22J83

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

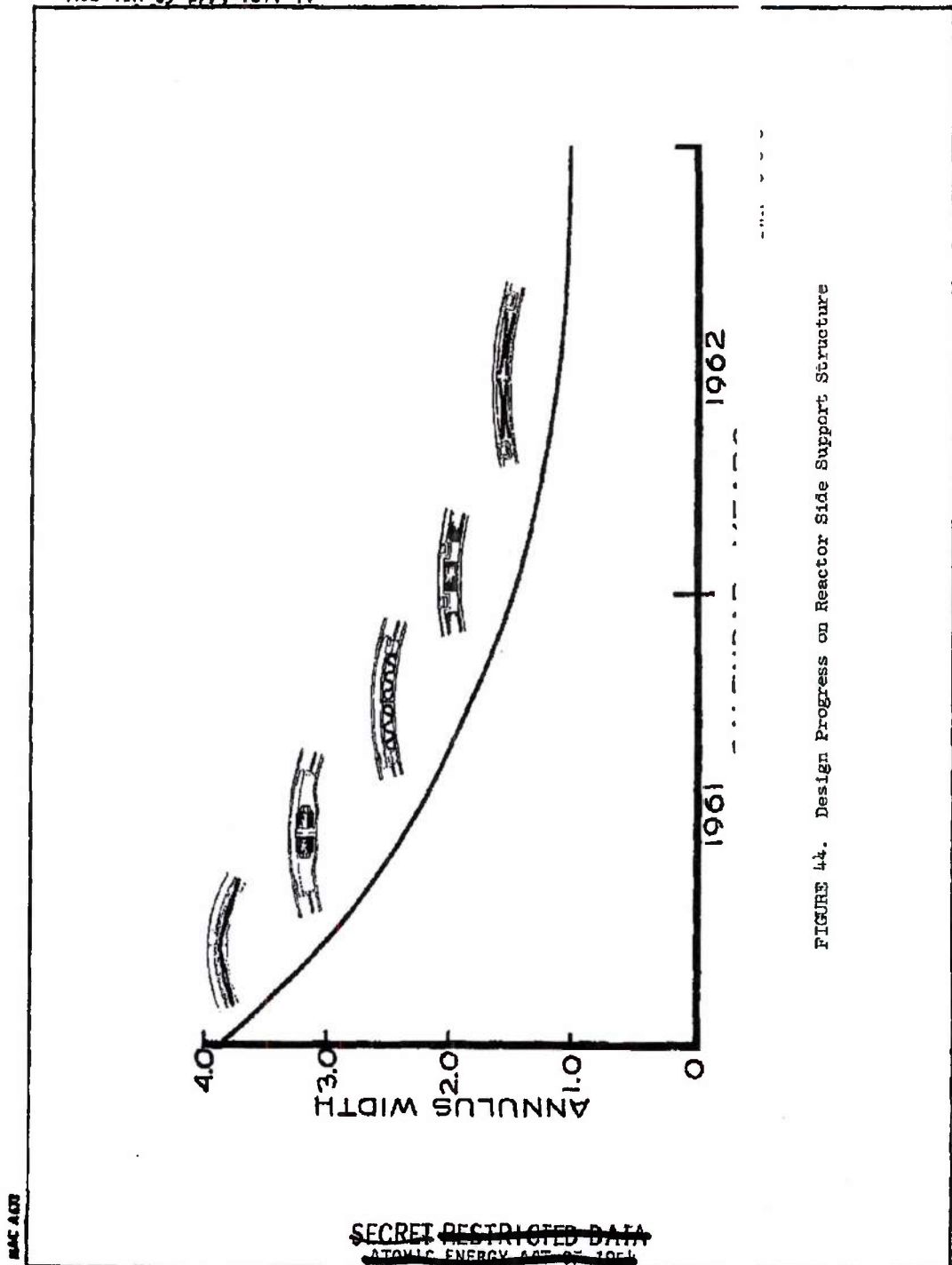


FIGURE 44. Design Progress on Reactor Side Support Structure

ASD-TDR-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

1

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

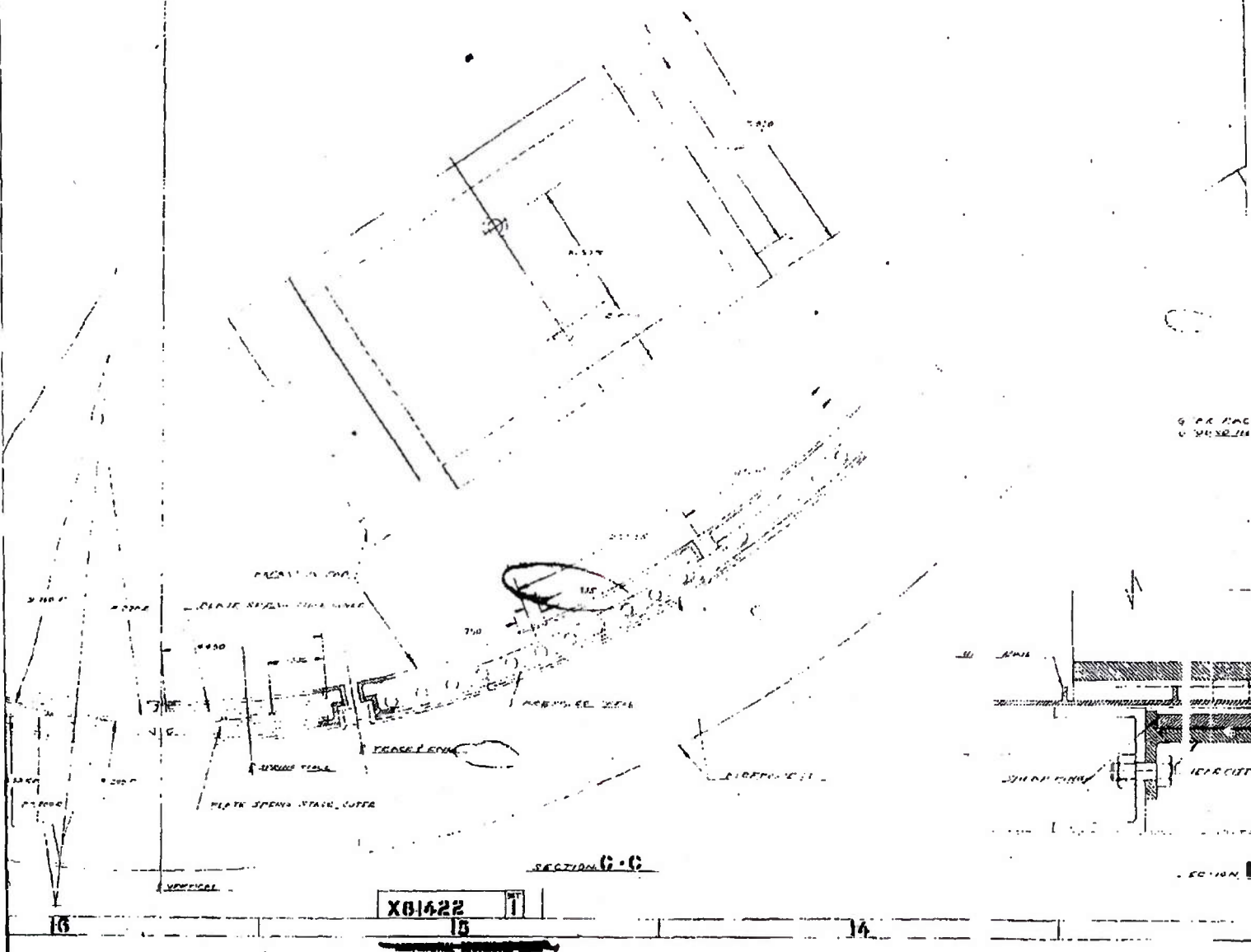
17

18

X81422

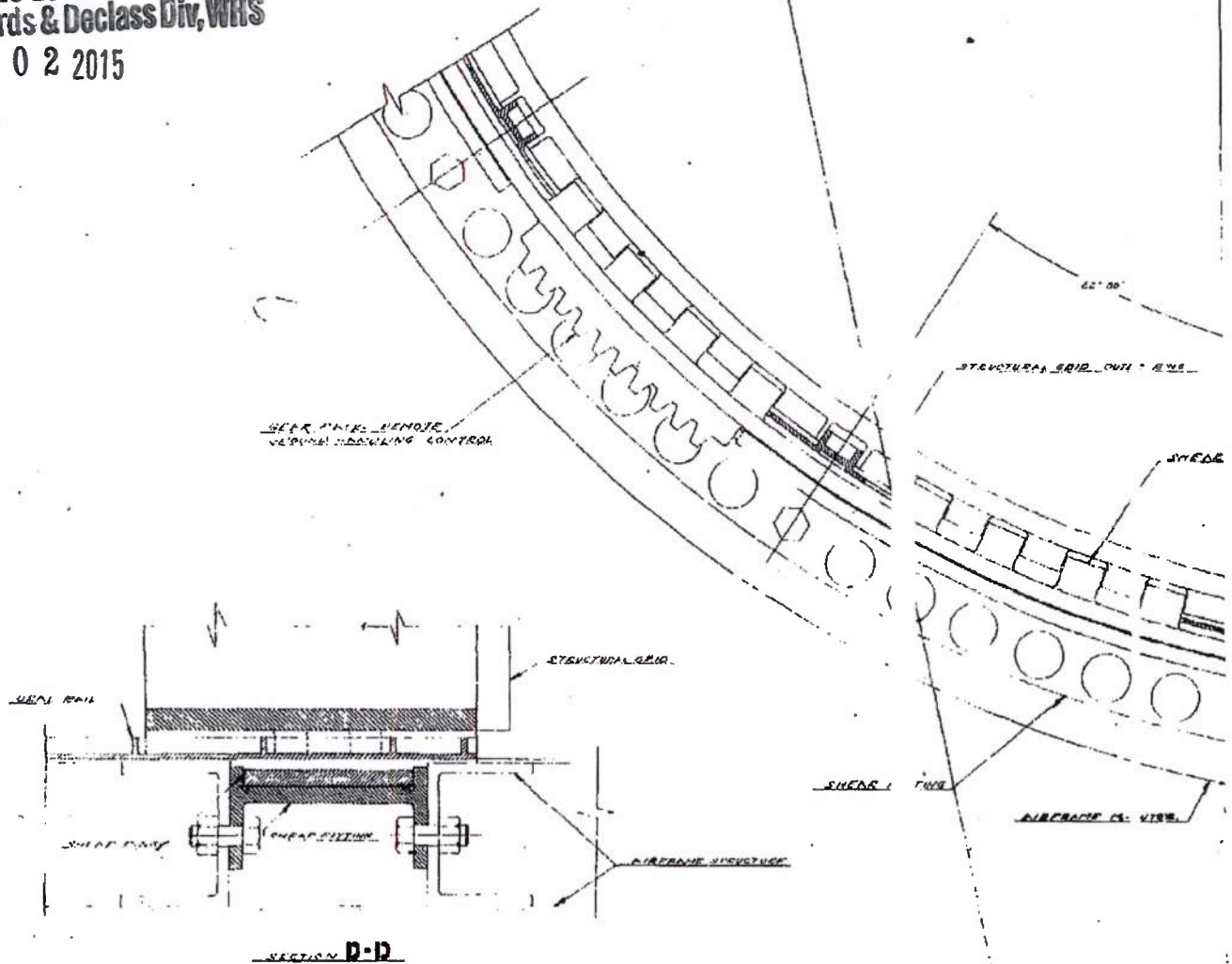
15

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



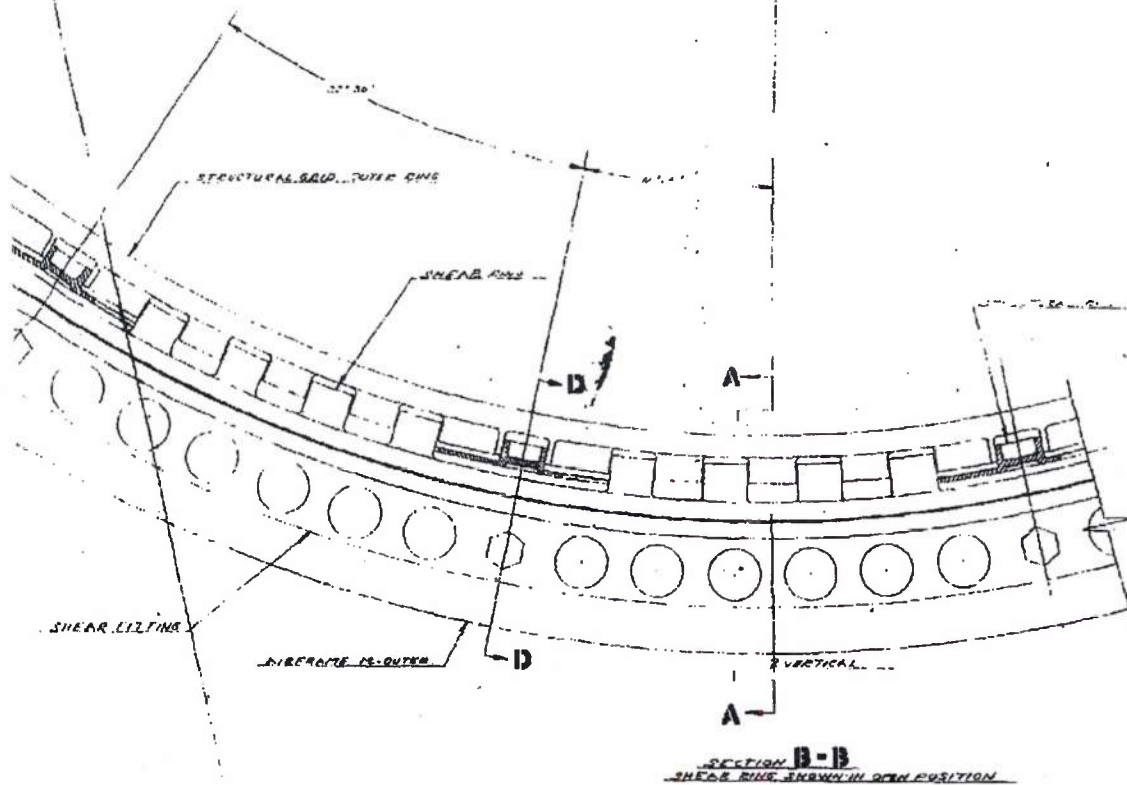
3

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015



4

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015



X01422

1

12

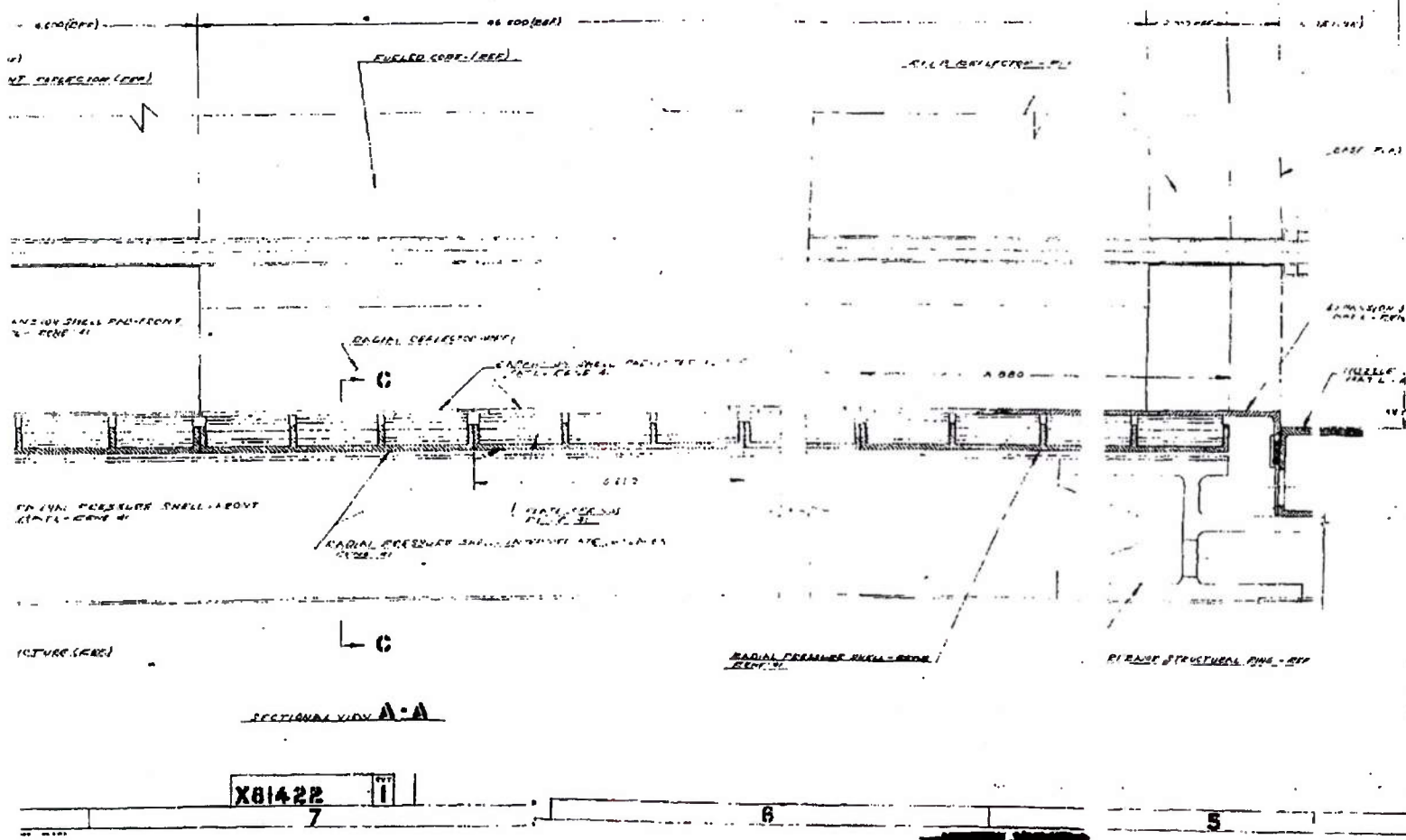


11

7

6

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015



॥ श्रीगणेशाय नमः ॥



13:

—ツツク—

— 227 —

11. L.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

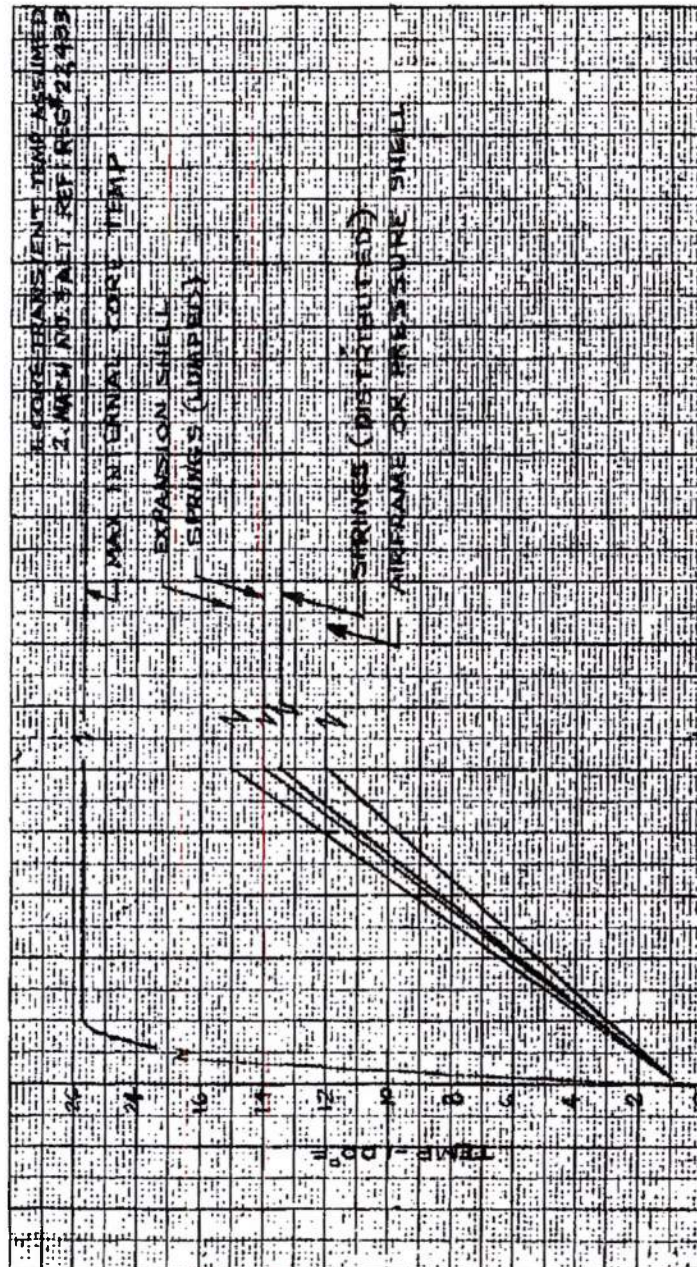


FIGURE 46. Assumed Side Support Temperatures (ANA Hot Day)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

ORT. 6003

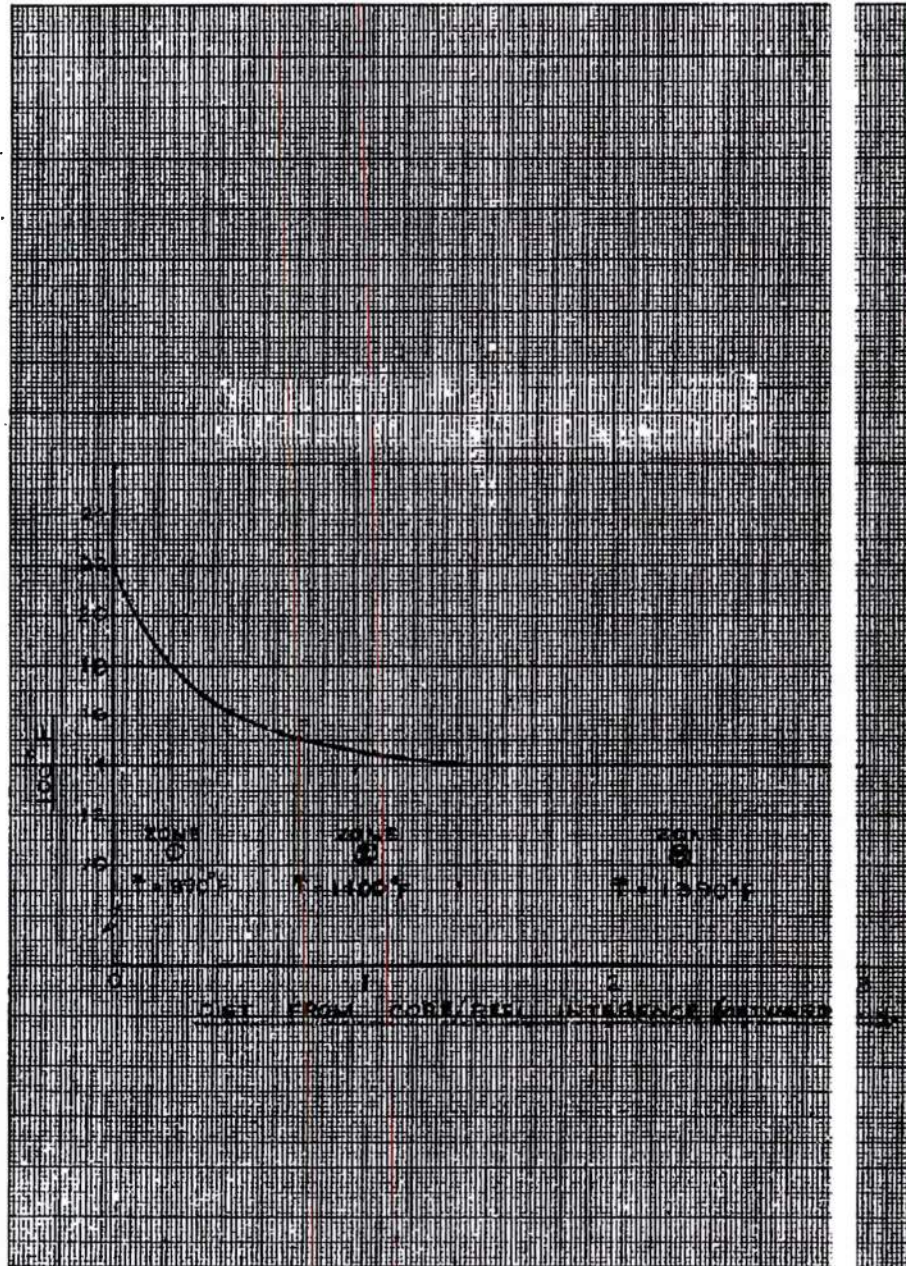


FIGURE 47. Assumed Side Reflector Temperature Distribution

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E958

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

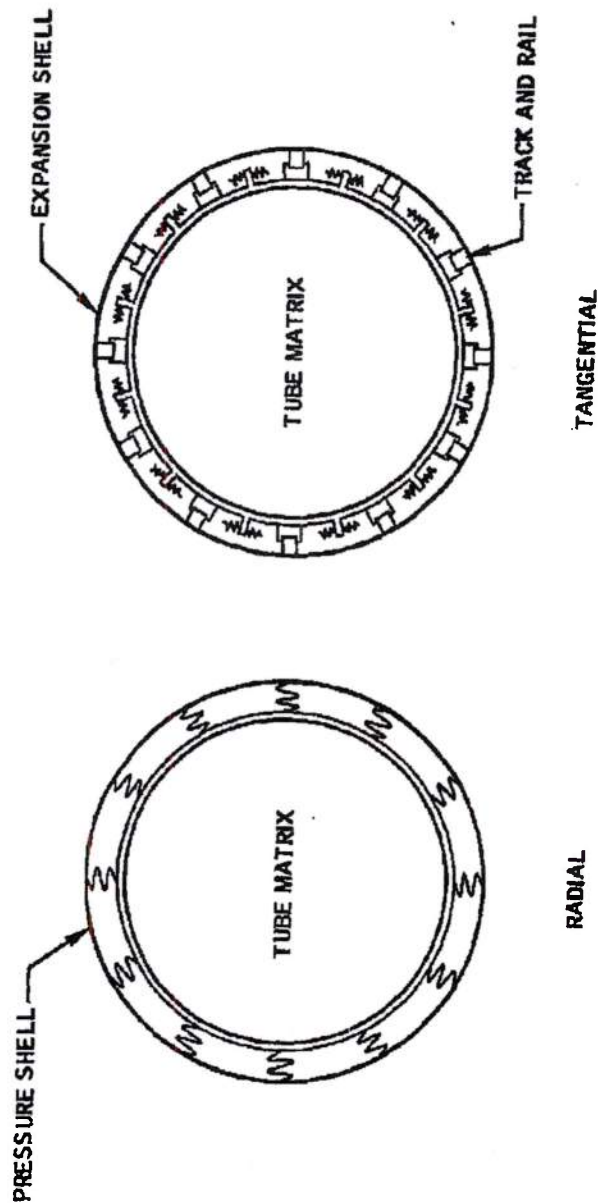


FIGURE 48. Schematic of Radial and Tangential Support Structures

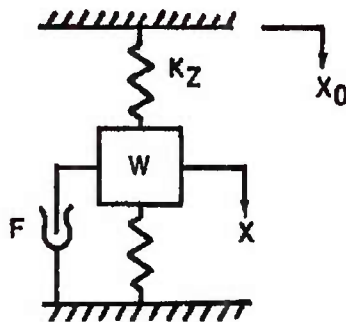
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22J90

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

ONT 6003



W = WEIGHT OF REACTOR, LBS/AXIAL INCH

K_z = EFFECTIVE TRANSVERSE SPRING CONSTANT OF LATERAL SUPPORT,
 LBS/AXIAL INCH/TRANSVERSE INCH

F = FRICTION FORCE EXERTED ON PERIPHERY OF REACTOR, LBS/AXIAL INCH

X_0 = PRESCRIBED TRANSVERSE DEFLECTION OF VEHICLE, INCHES

X = RESULTING DEFLECTION OF REACTION, INCHES

FIGURE 49. Dynamic System Model

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

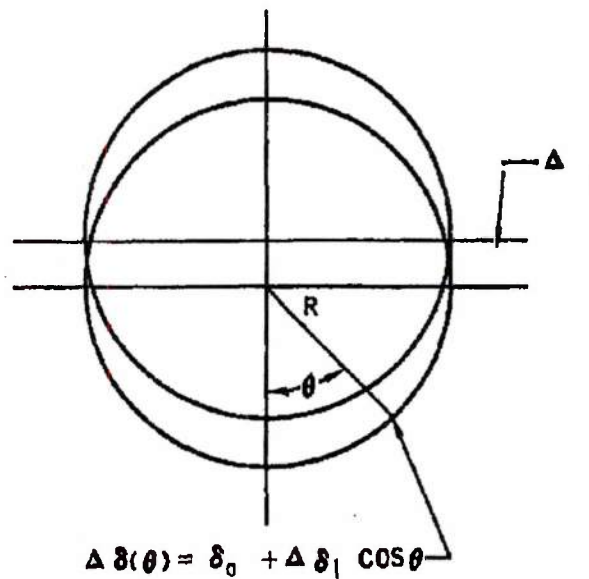


FIGURE 50. Pressure Distribution

UNCLASSIFIED

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

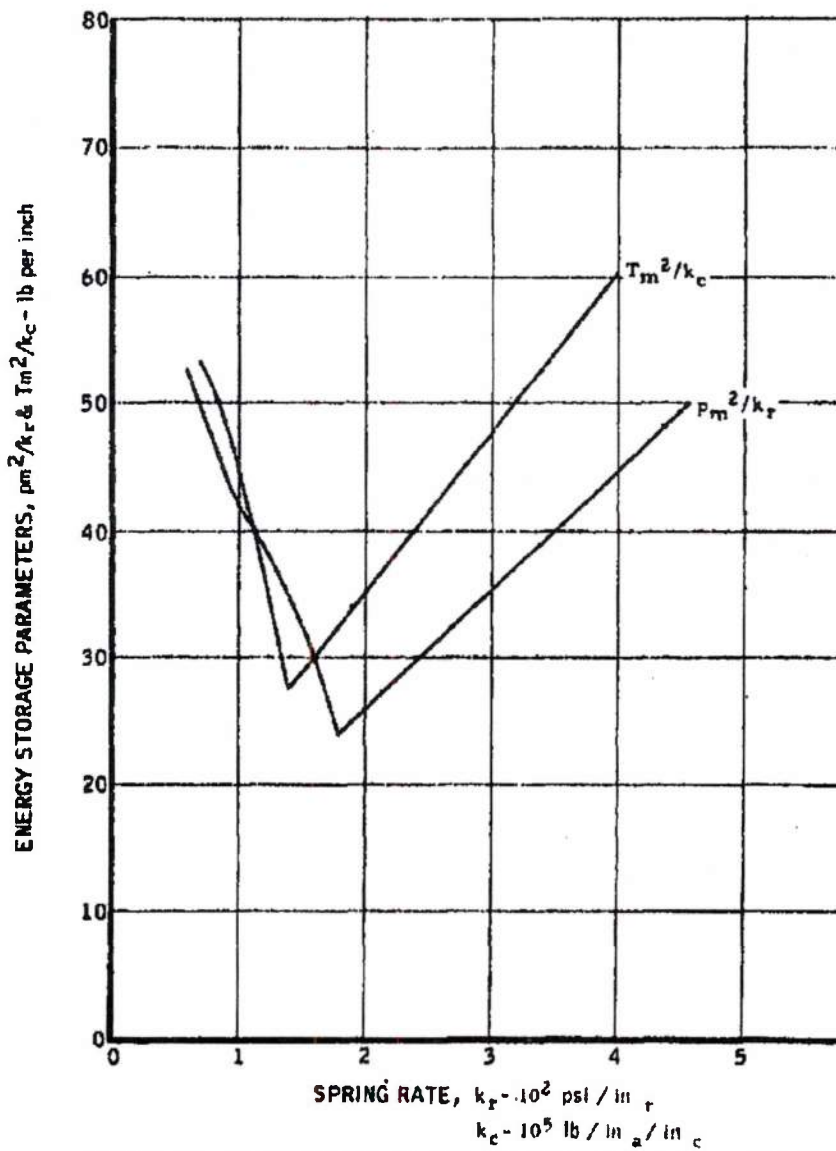


FIGURE 51. Required Energy Storage for Lateral Support System

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22B33

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

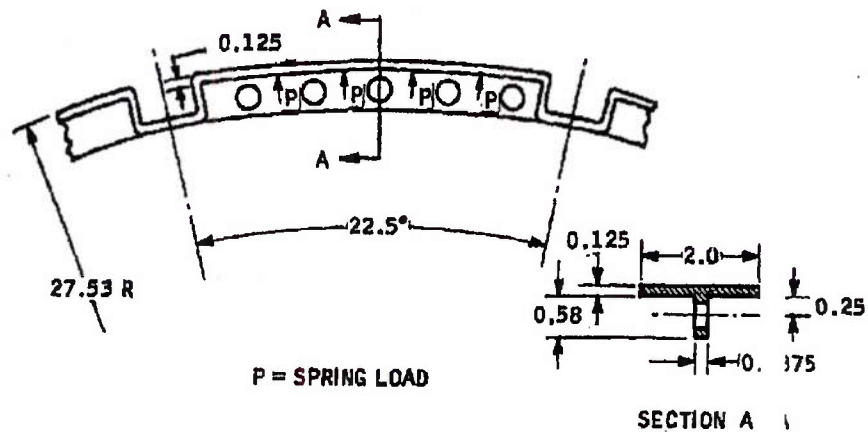


FIGURE 52. Typical Section of Web-Shell Combination

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 603

N22J91

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

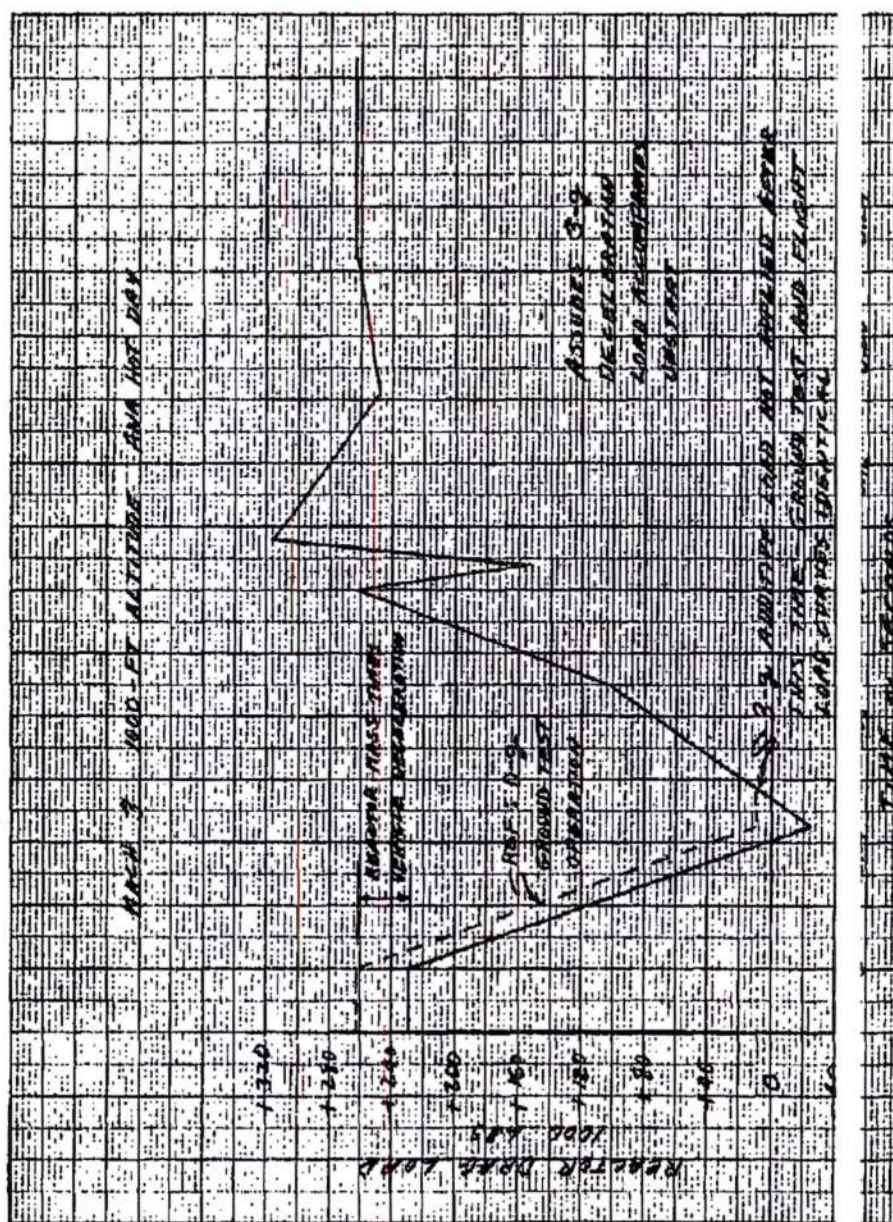


FIGURE 53. Estimated Full Scale Reactor Drag Load Variation During Unstart Transient Flight Operation

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

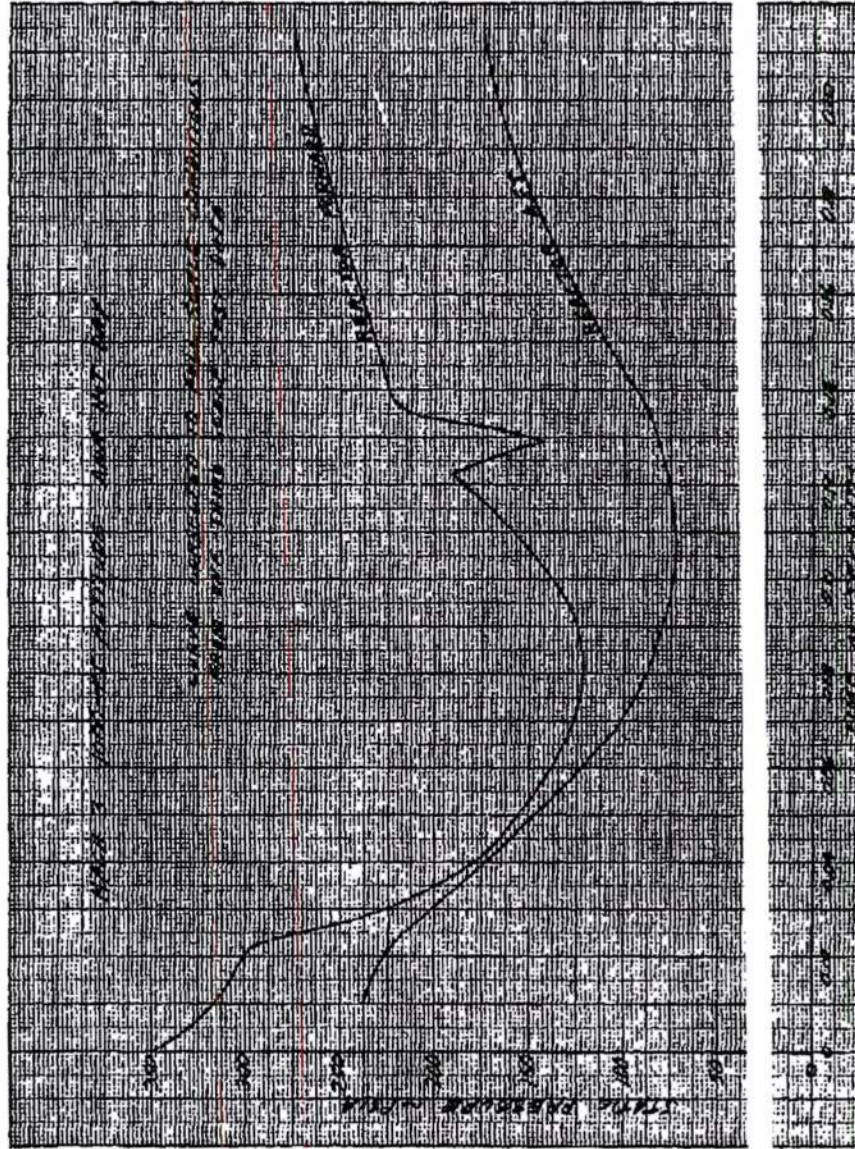


FIGURE 54. Estimated Pressure Transient on Full Scale Reactor During Unstart

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E723

- 207 -

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

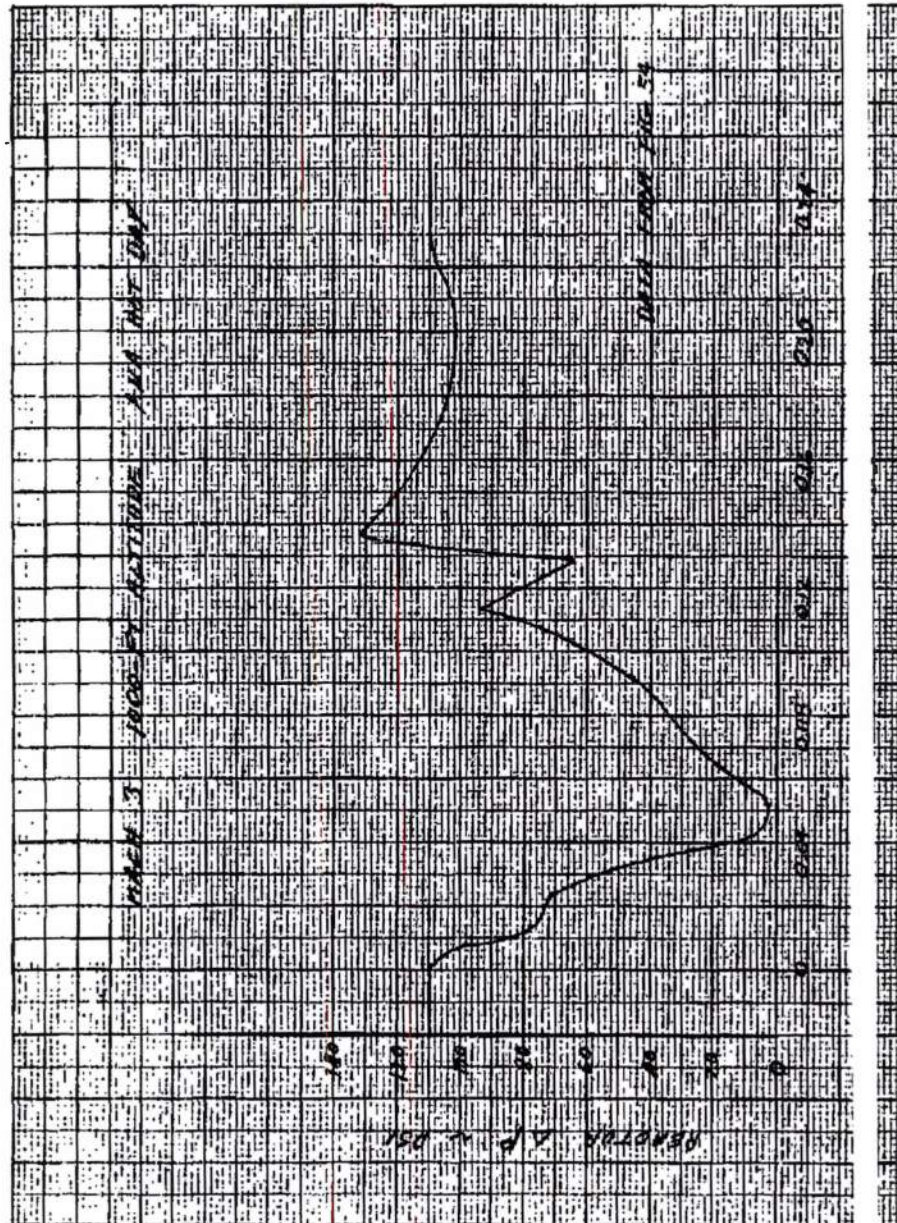


FIGURE 55. Estimated Differential Pressure Variation on Full Scale Reactor During Unstart

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 6003

N22E724

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

ASD-TDR-63-277, Vol. IV

REPORT 6003

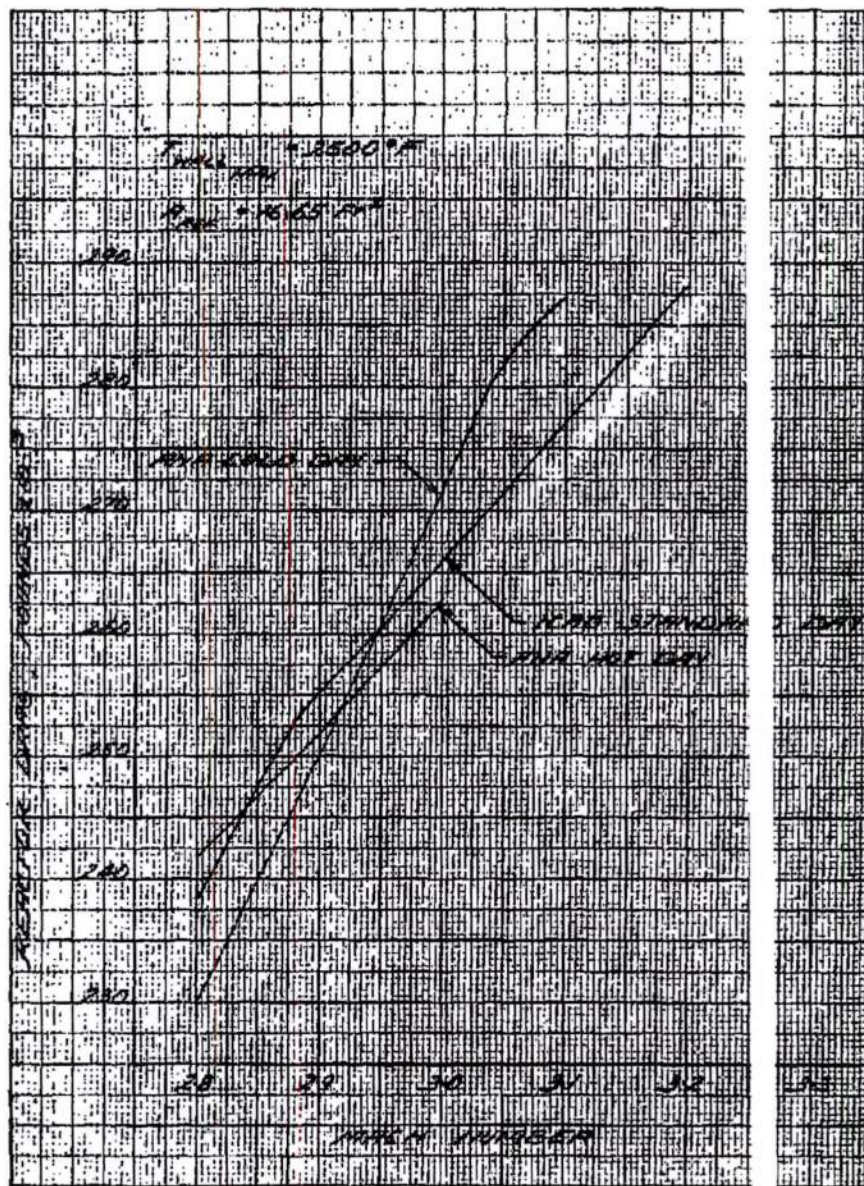


FIGURE 56. Reactor Drag, 1,000-Ft Altitude

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

N22E703

- 209 -

DECLASSIFIED IN FULL

Authority: EO 13526

Authority: EO 13526
Chief, Records & Declass Div, WHS

Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

QNT. 6003

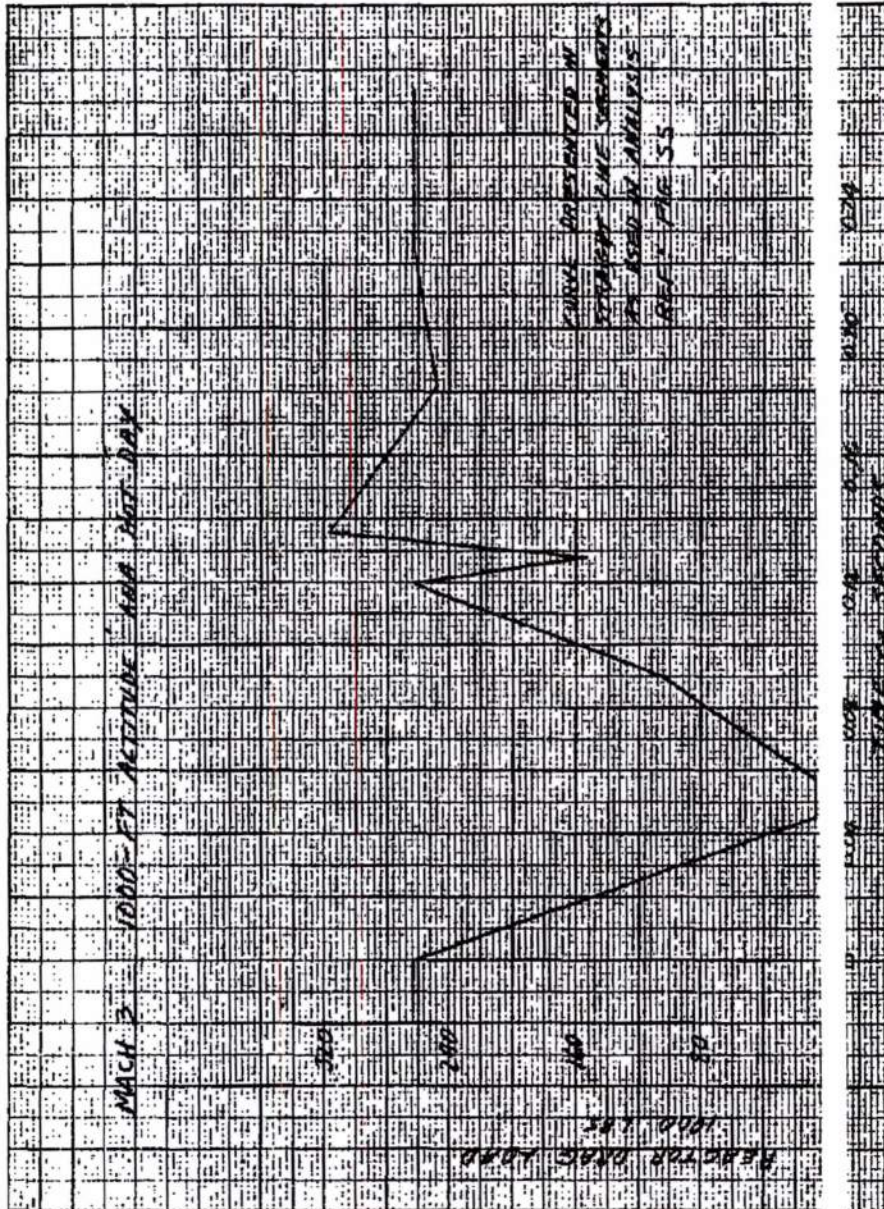


FIGURE 57. Estimated Full Scale Reactor Drag Load Variation During Unstart Transient Ground Test Only

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

ASD-TDR-63-277, Vol. IV

REPORT 6003

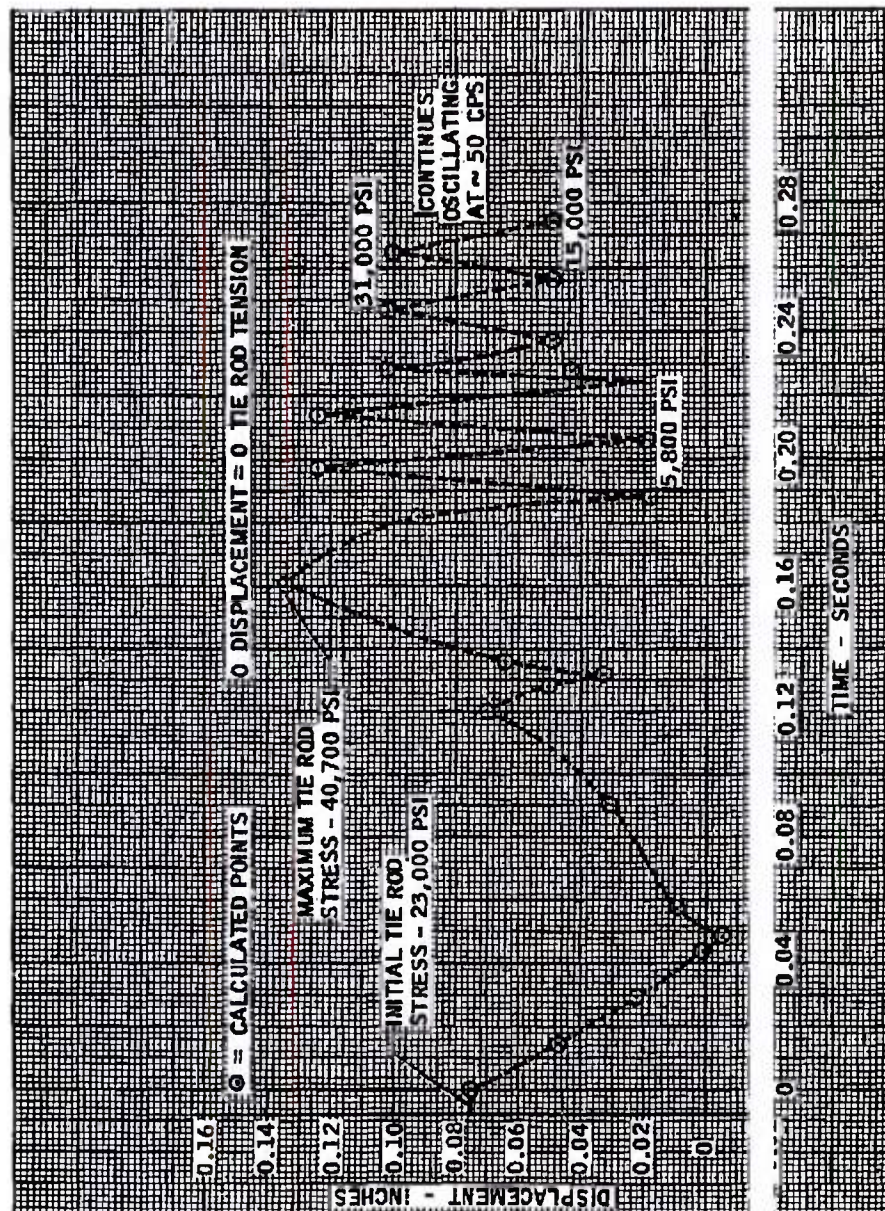


FIGURE 58. Estimated Reactor Displacement During Unstart Transient Ground Test Only--
 Mach 5, 1000-Ft Altitude, ANA Hot Day

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

N22E726

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-272, Vol. IV

RE 6003

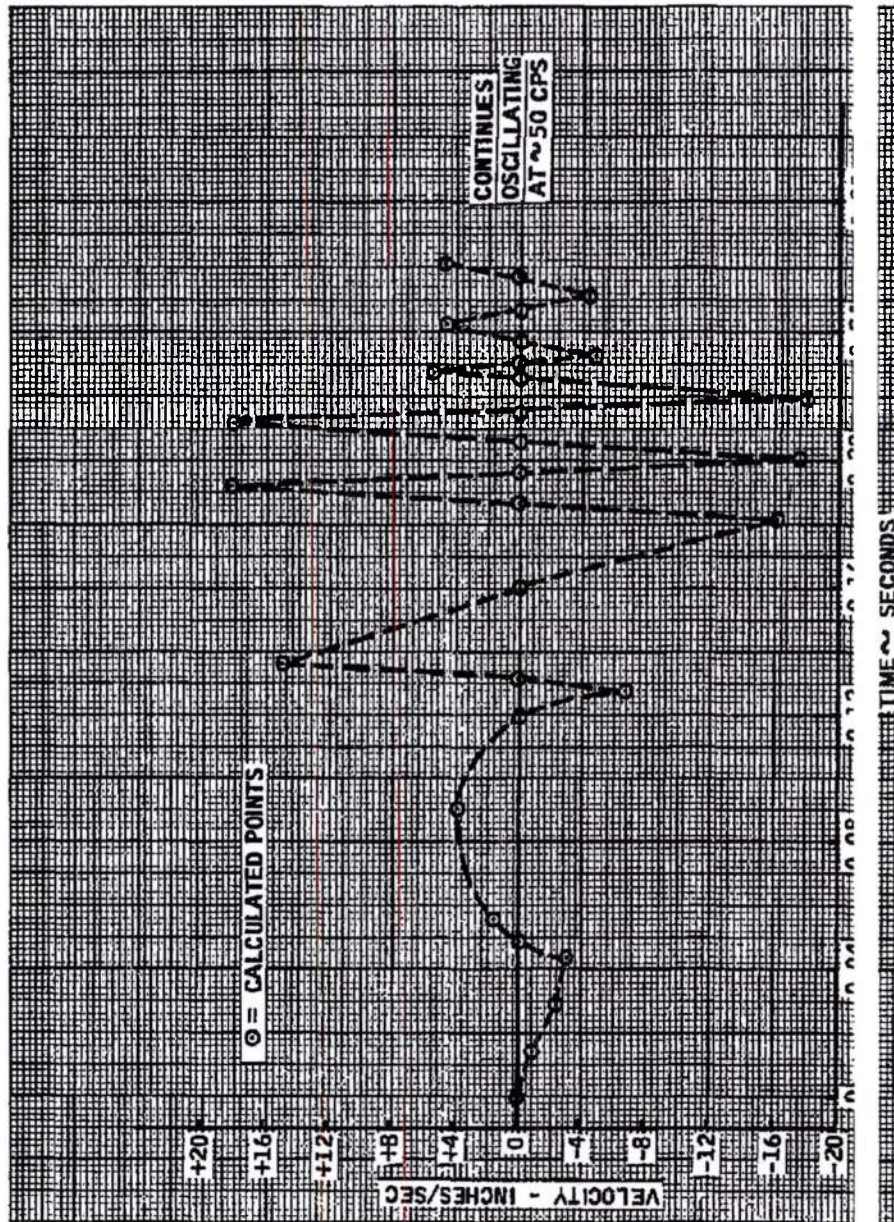


FIGURE 59. Estimated Reactor Velocity During Unstart Transient Ground Test Only--
 Mach 3, 1000-Ft Altitude, AMA Hot Day

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E727

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

ASD-TDR-63-277, Vol. IV

REPORT 6003

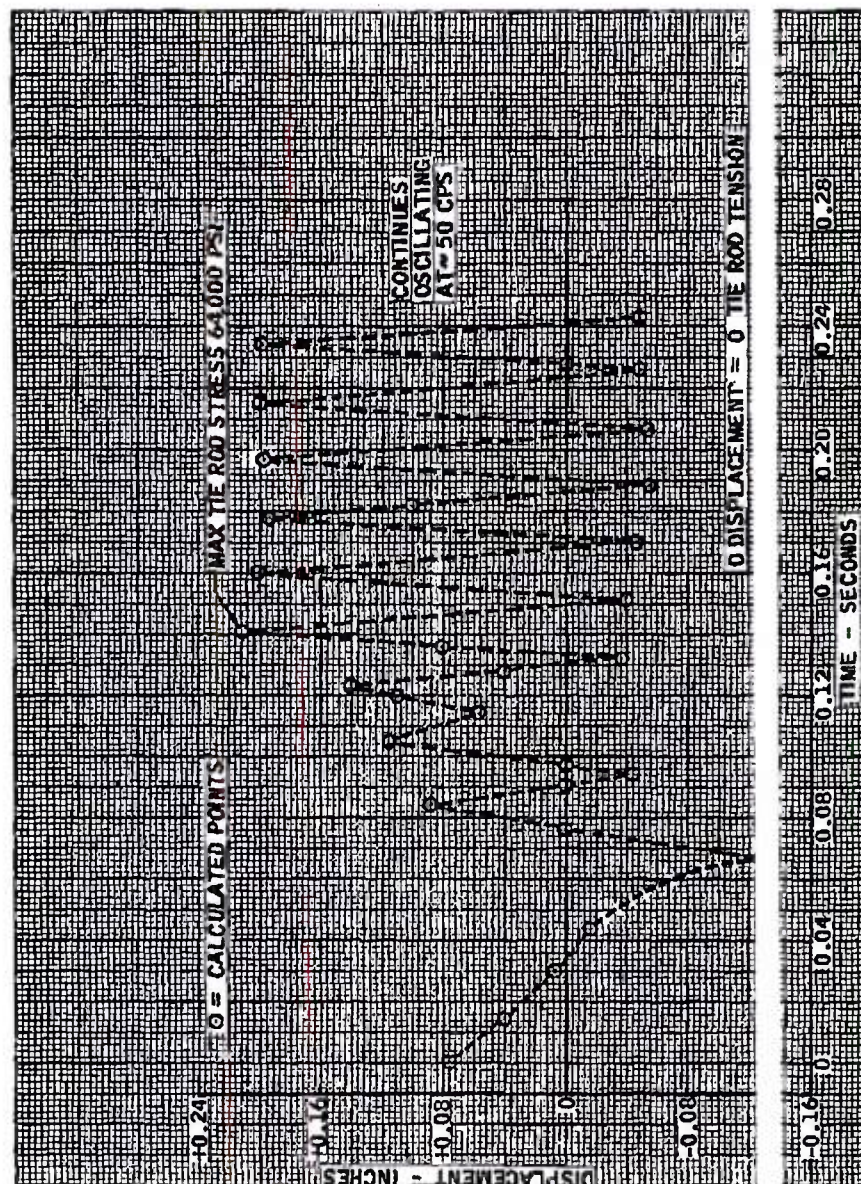


FIGURE 60. Estimated Reactor Displacement During Unstart Transient - Flight Operation
 With 3 g Vehicle Deceleration--Mach 3, 1000-Ft Altitude, ANA Hot Day

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

MAC 463

22C57

- 213 -

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

ASD-TDR-63-277, Vol. IV

6003

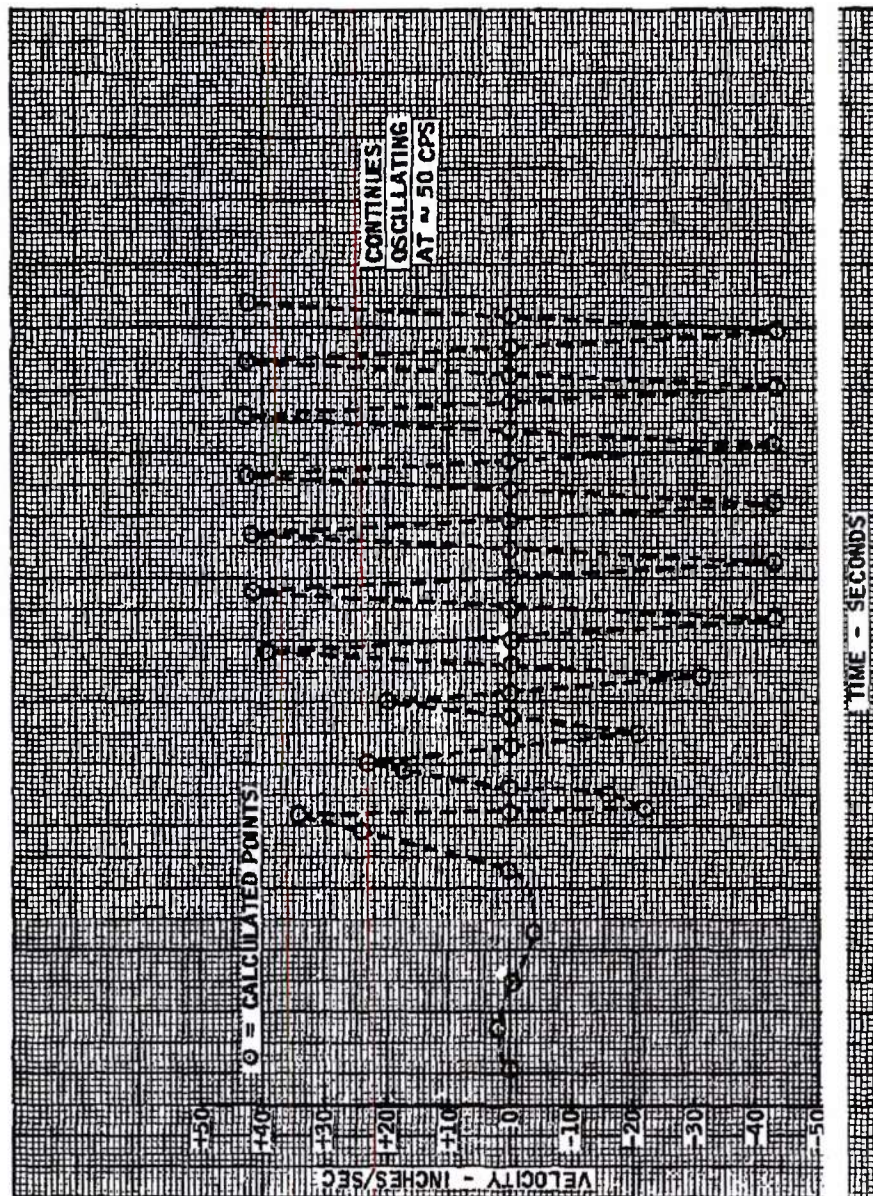


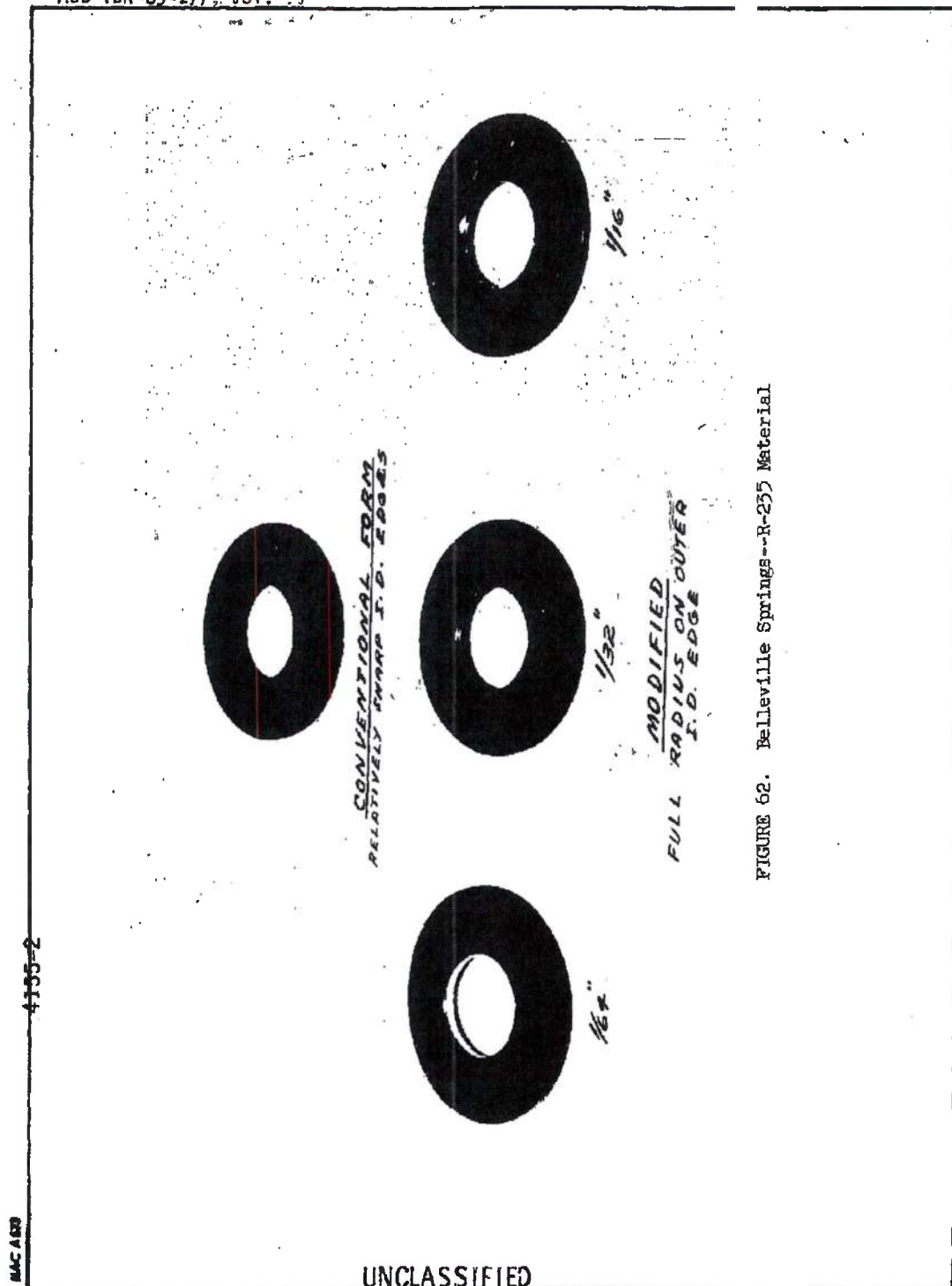
FIGURE 61. Estimated Reactor Velocity During Unstart Transient - Flight Operation
 With 3g Vehicle Deceleration - Mach 3, 10000-Ft Altitude, ANA Hot Day

~~SECRET RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

UNCLASSIFIED

ASD-TDR-63-277, Vol. 1

REPORT 6003

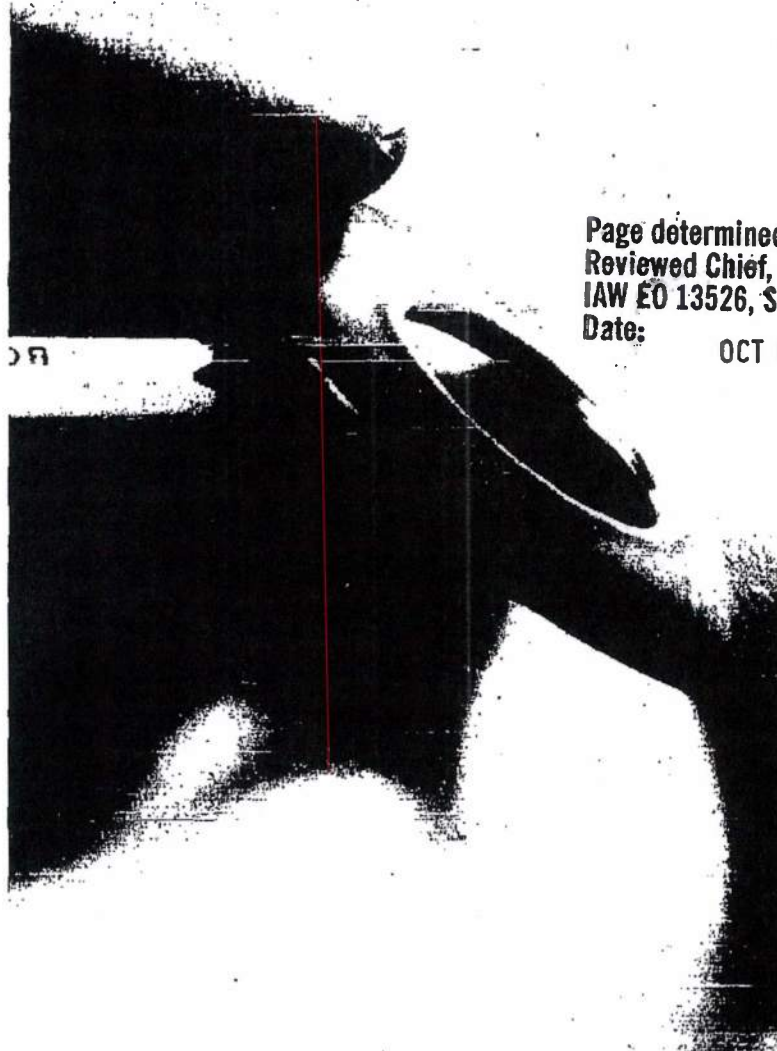


UNCLASSIFIED

UNCLASSIFIED

ASD-TPR-63-237

REPORT 6003



Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

NEG. 4155-6

MAC 422

1 SF 63 Pelletville Spring--R-235 Material
(.6 in. x .17 in. Both L.D. Edges)

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277 - Vol. 1

REPORT 6003

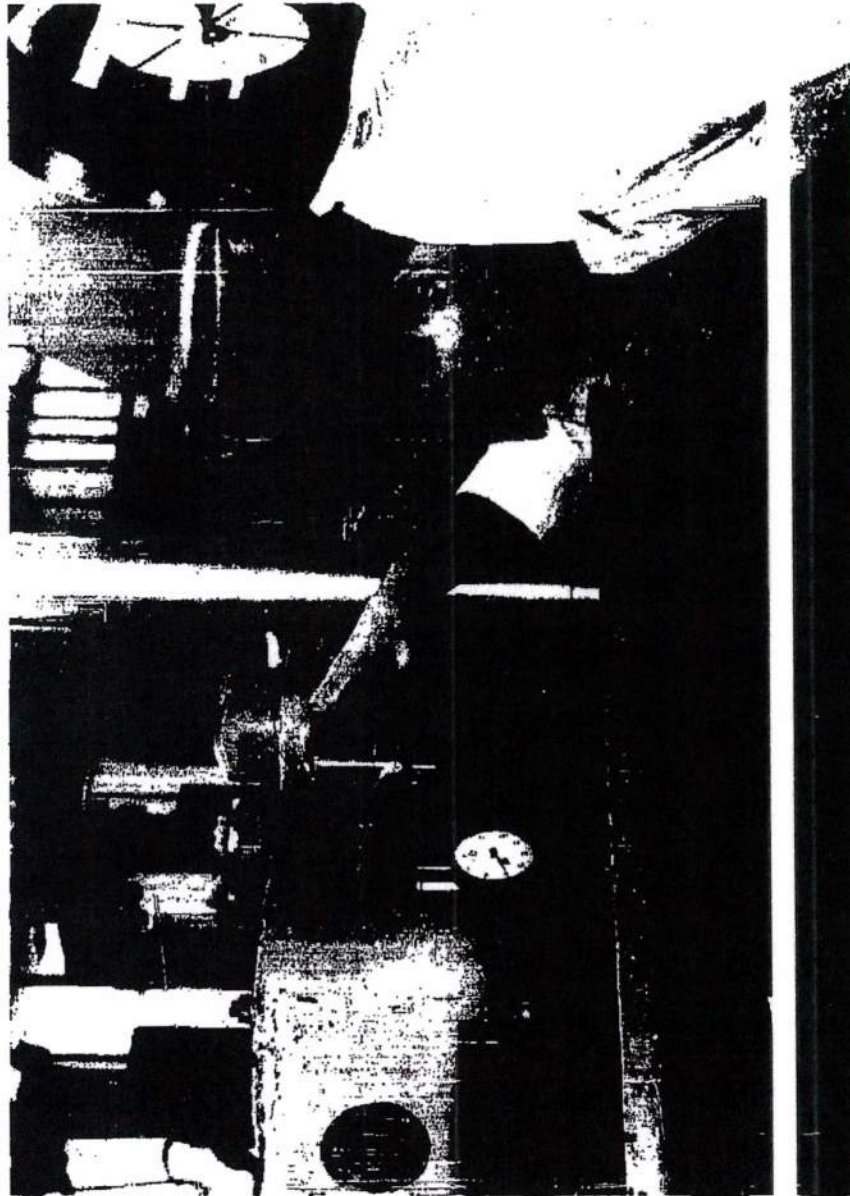


FIGURE 64. Test Setup for Single Belleville Spring--R-235 Material

4155-3

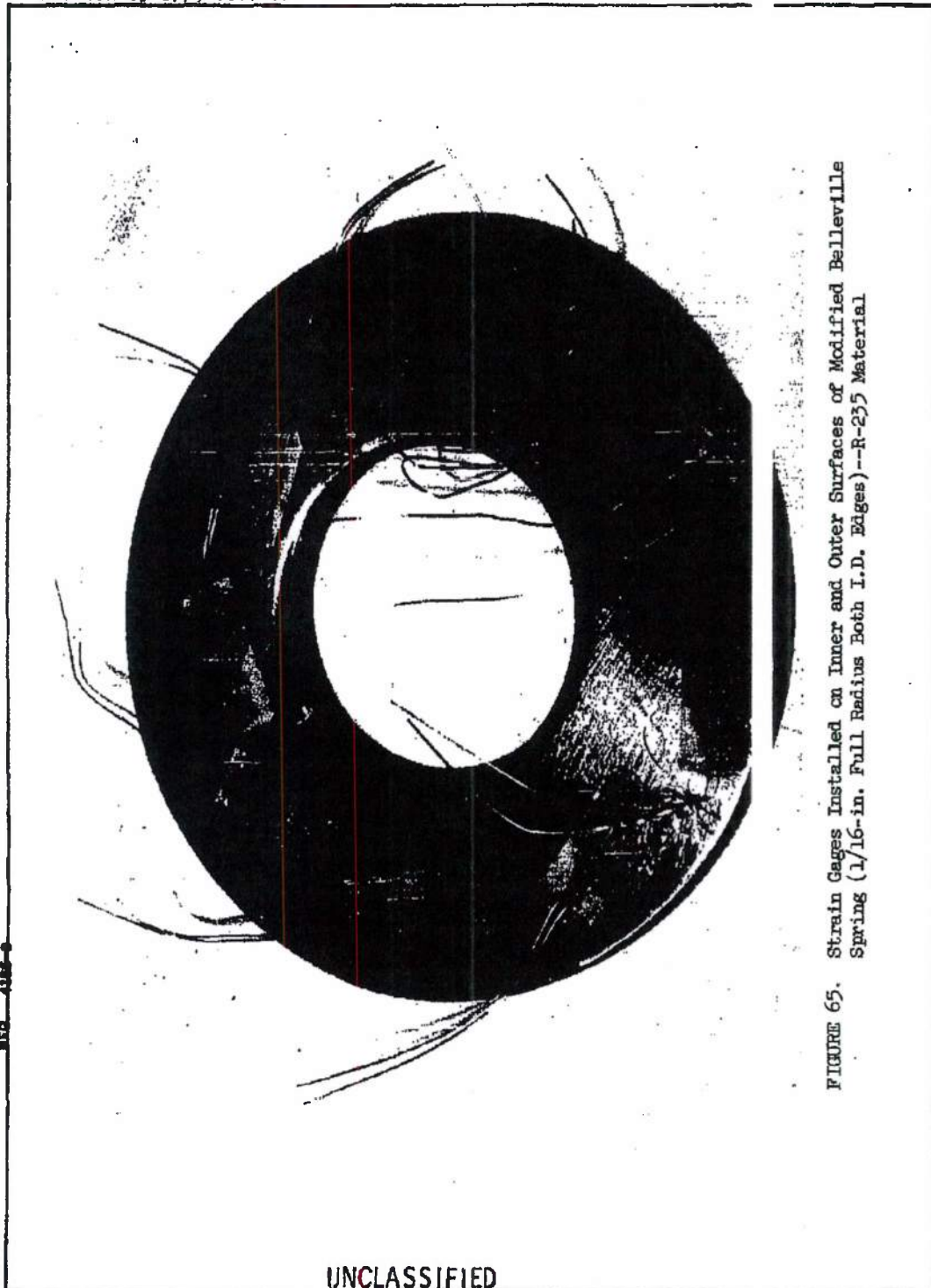
BAC 1673

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. 27

REPORT 6003



UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

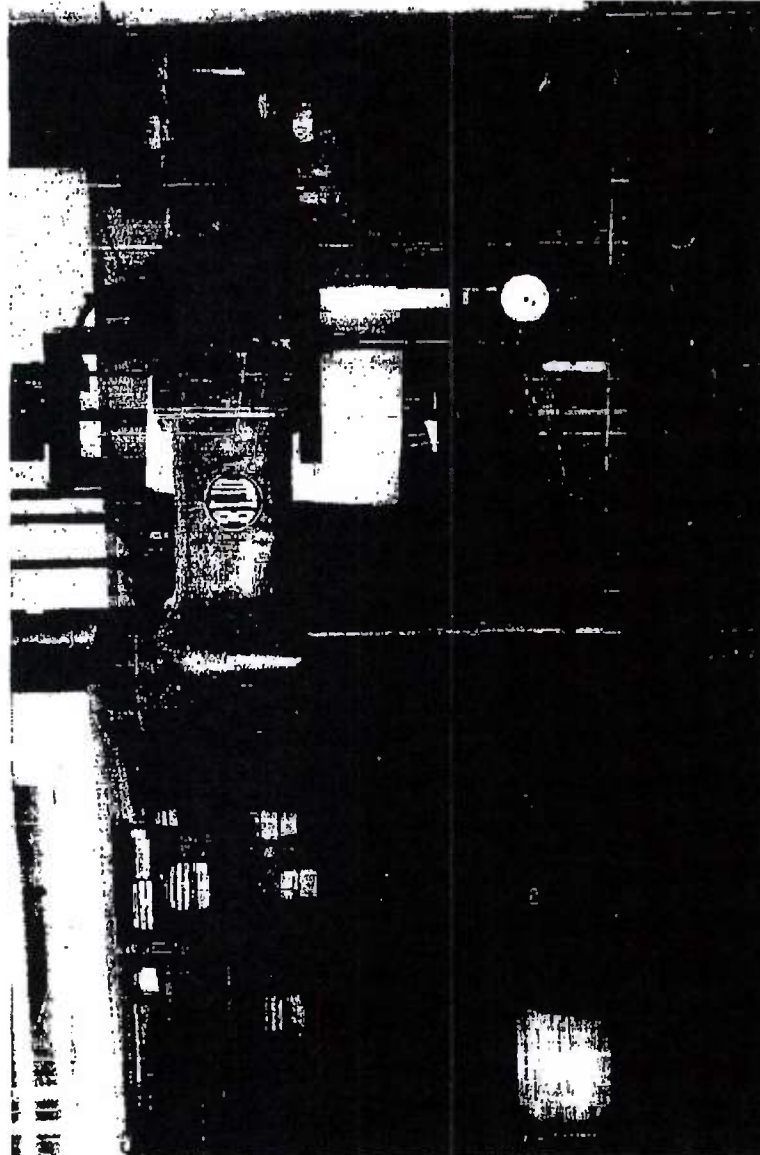


FIGURE 66. Setup for Test of Single Belleville Spring with Strain Gages Installed

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REV 6003

(SR-4, A-19 PAPER BACKED, 1/16" x 1/16" STRAIN GAGES)

AMBIENT TEMPERATURE
(EFFECTIVE FLAT PLATE LOADING)

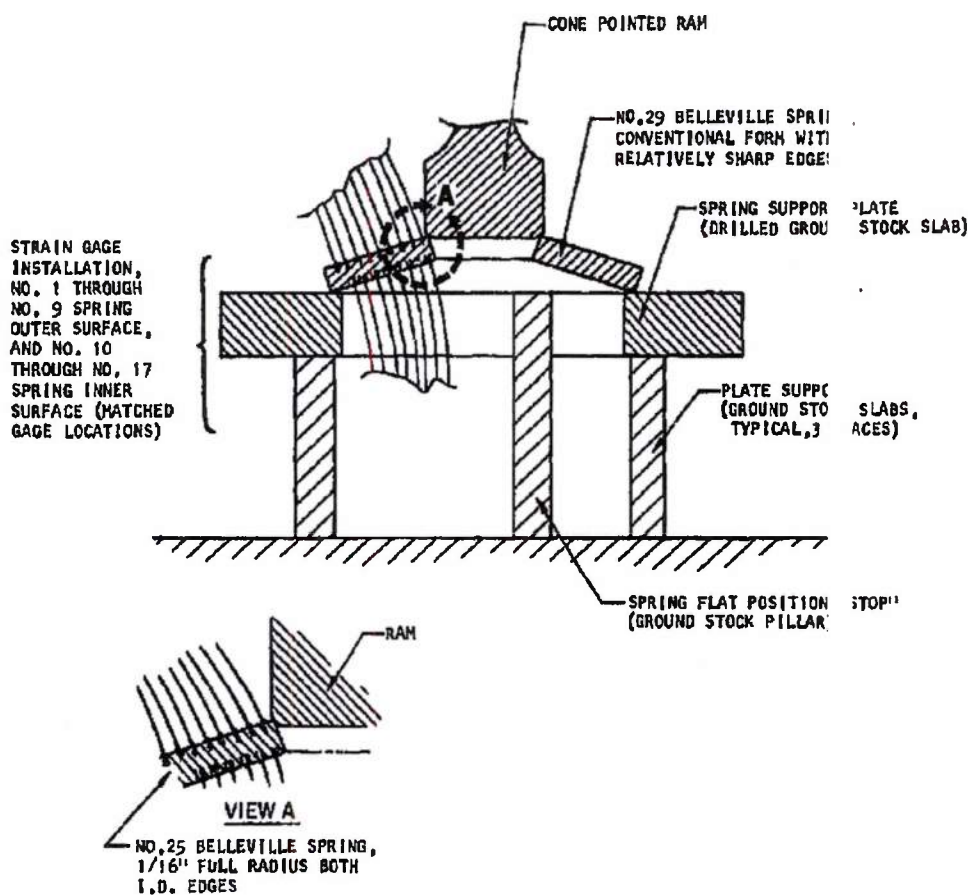


FIGURE 67. Test Setup for Strain Gaged Belleville Springs (R-2351 Serial)

UNCLASSIFIED

UNCLASSIFIED

ASD-TER-63-277, Vol. 17

REPORT 6003

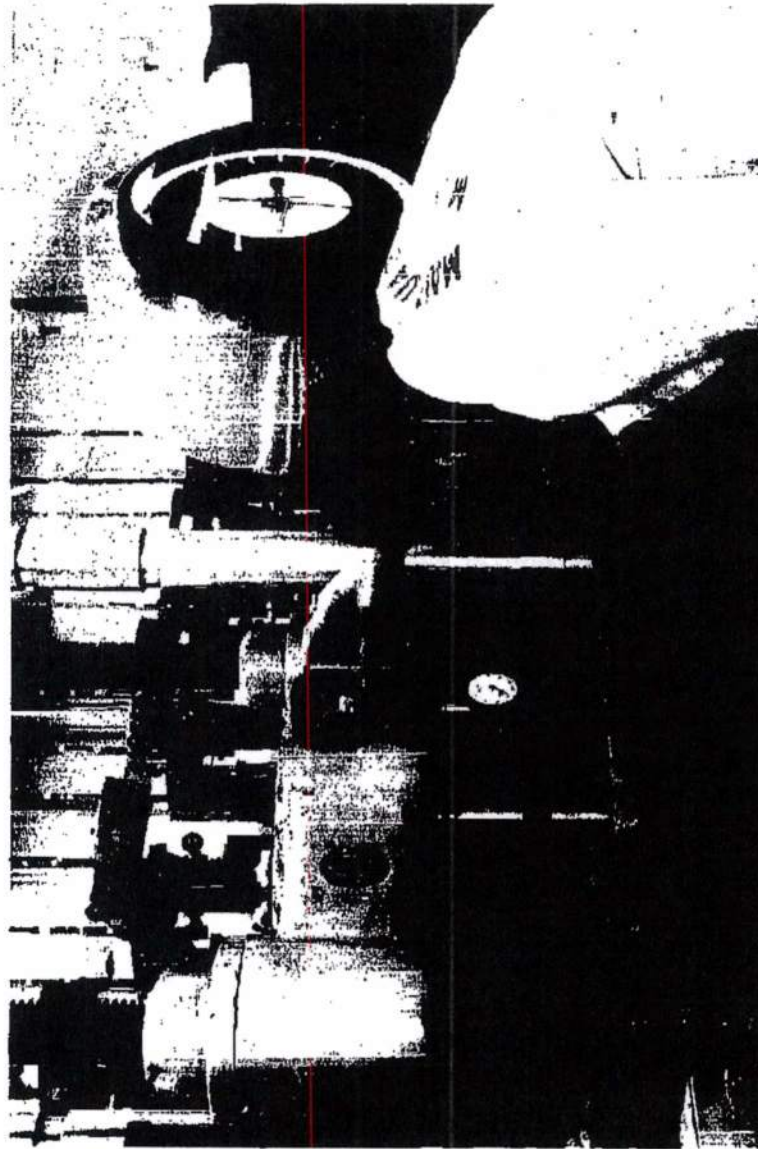


FIGURE 58. Test Setup for 10-Spring Series Stack of Belleville Springs--R-235 Material

MAC 403

4155-4

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. 17

REPORT 6003

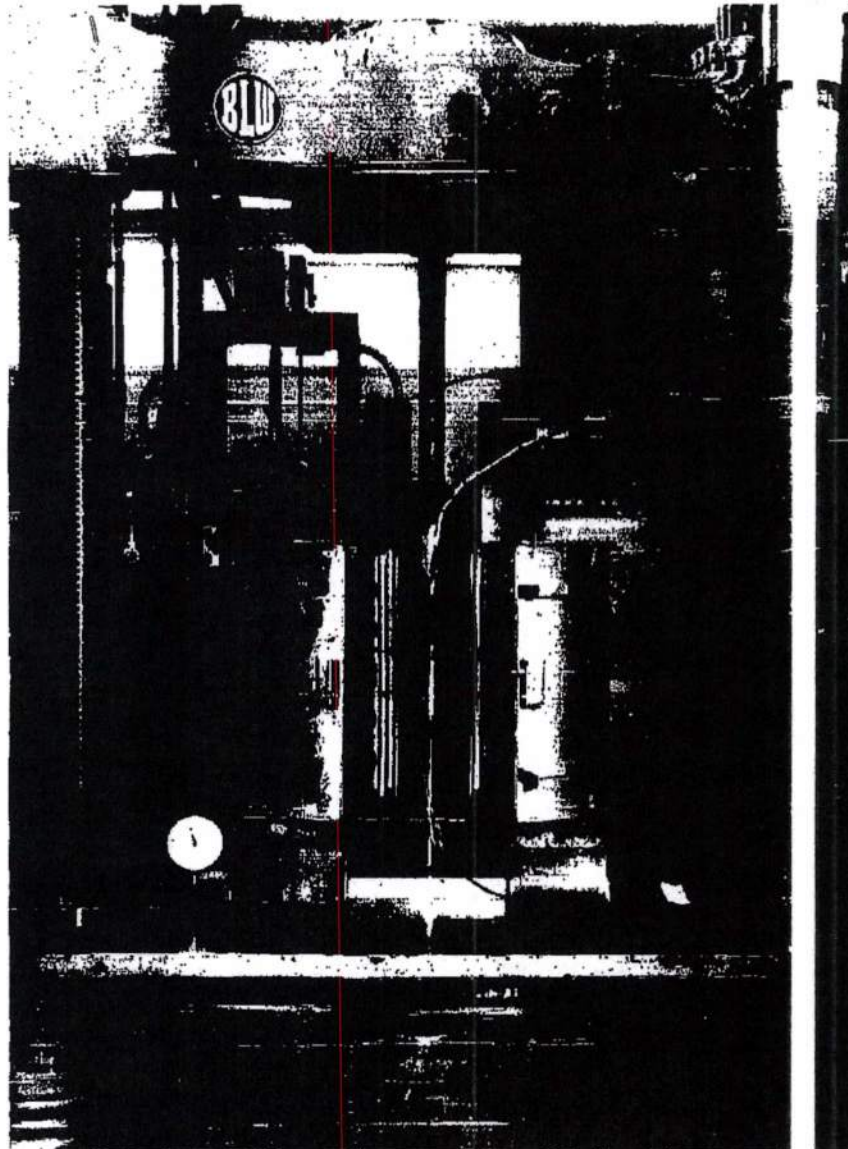


FIGURE 69. Partial Setup for 1400 P Test of 10 Spring
Series Stack of Belleville Springs--R-235 Material

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

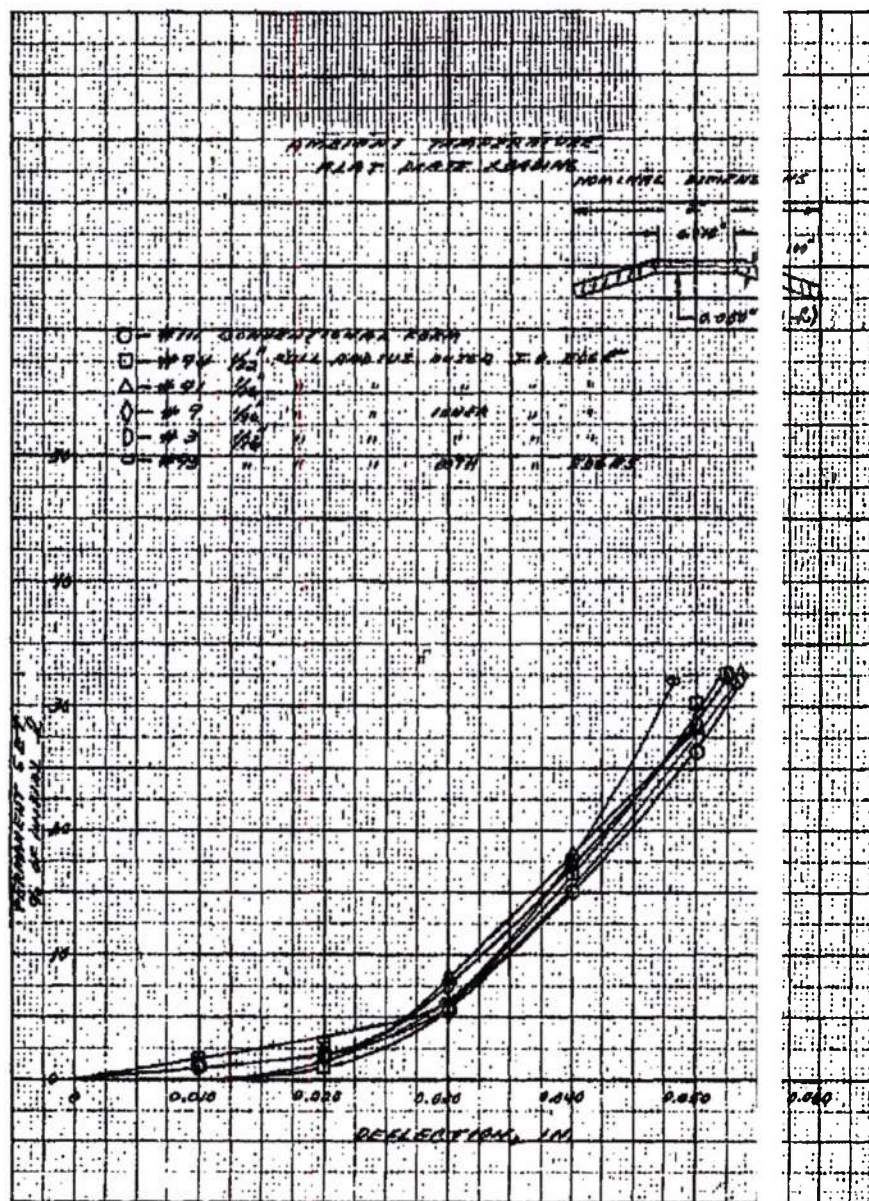


FIGURE 70. Permanent Set vs. Deflection During Presetting of Single Belleville Springs (R-255 Alloy)

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

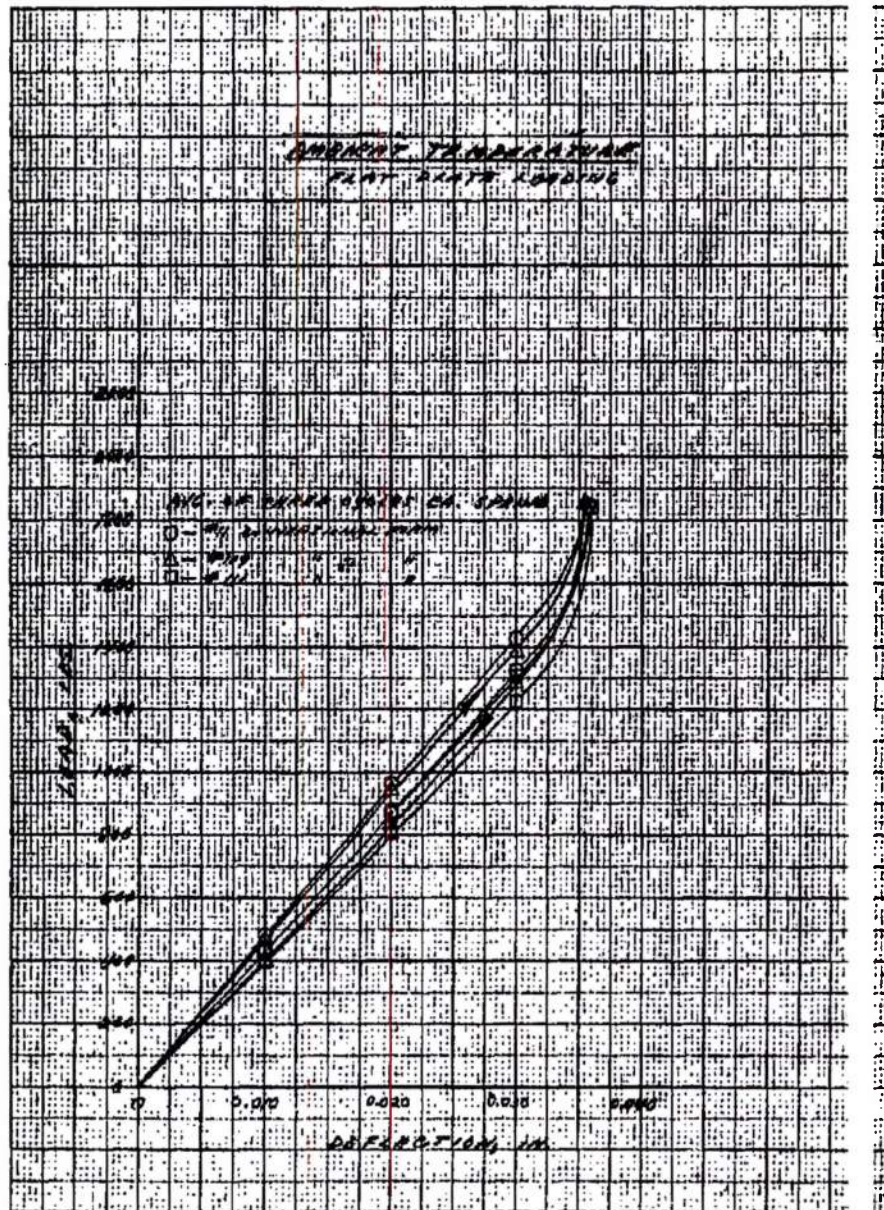


FIGURE 71. Load vs. Deflection: No. 11, No. 109, and No. 111 Single Belleville Springs (R-235 Alloy)

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

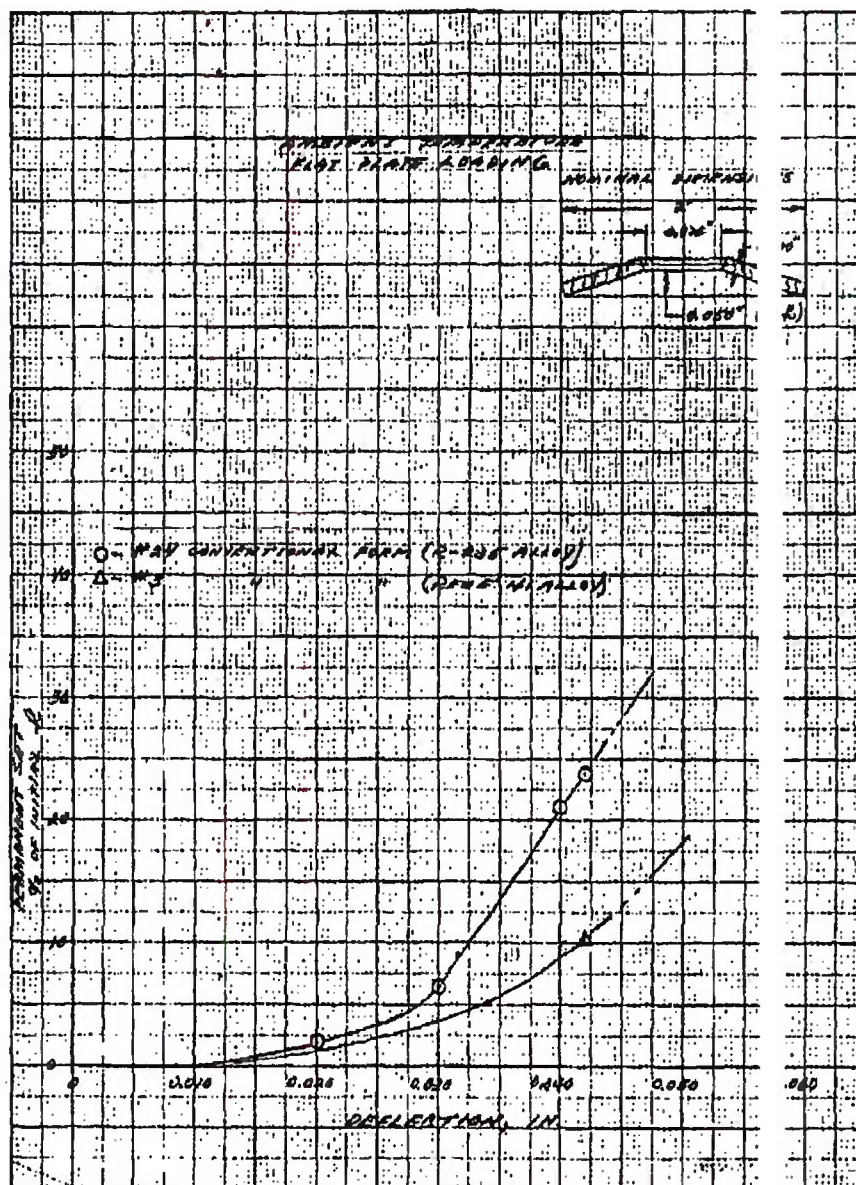


FIGURE 72. Permanent Set vs. Deflection During Presetti Single Belleville Springs

of

UNCLASSIFIED

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REP 6003

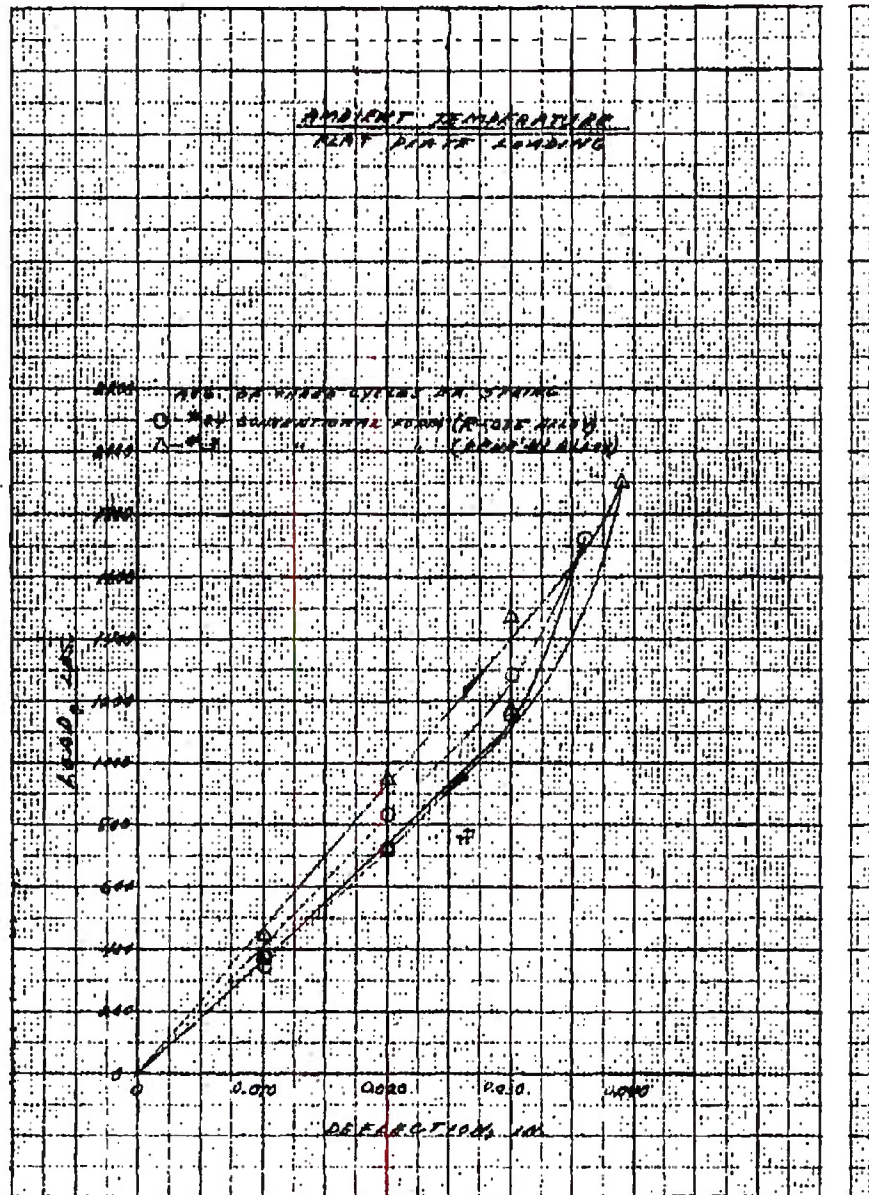


FIGURE 73. Load vs. Deflection for Single Belleville Springs (R-235 Alloy)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 1473

UNCLASSIFIED

REPORT 6003

ASD-TDR-63-277, Vol. IV

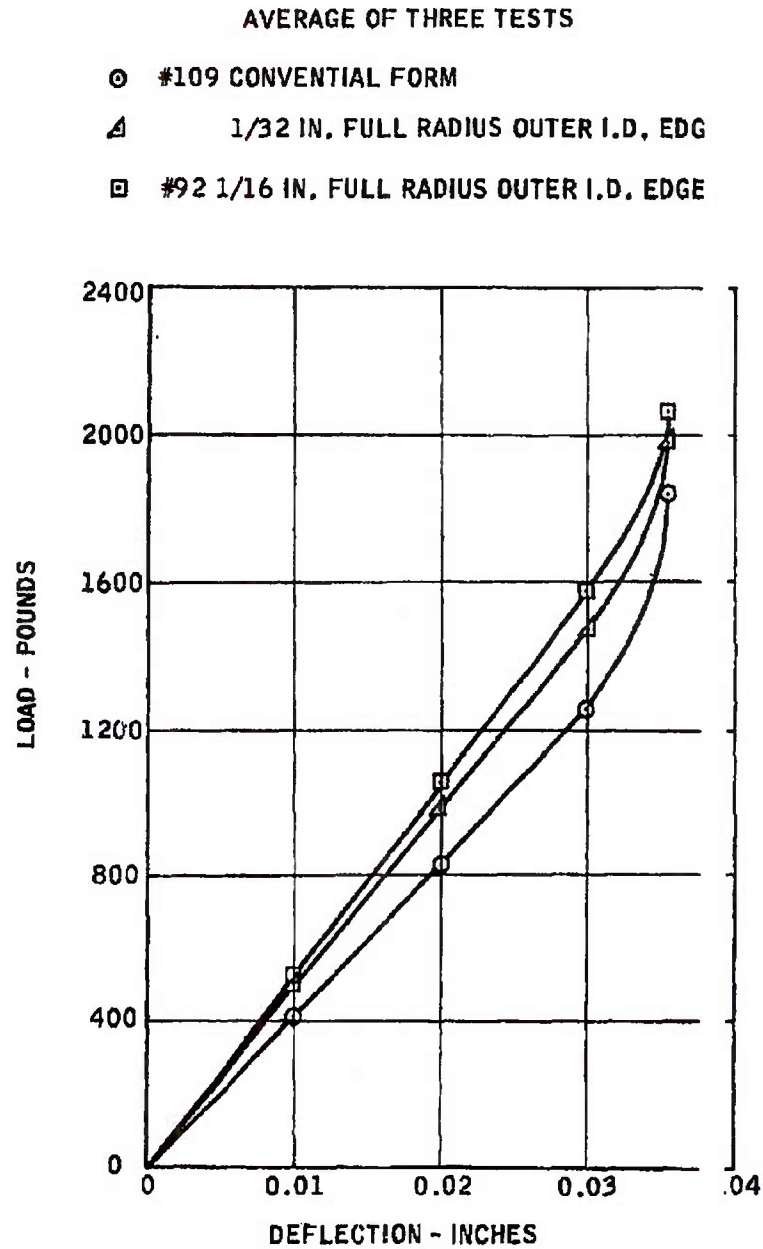


FIGURE 74. Load vs. Deflection for Single Belleville Springs (R-235 Alloy)--Ambient Temperature

UNCLASSIFIED

MAC 403

22J63

- 227 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

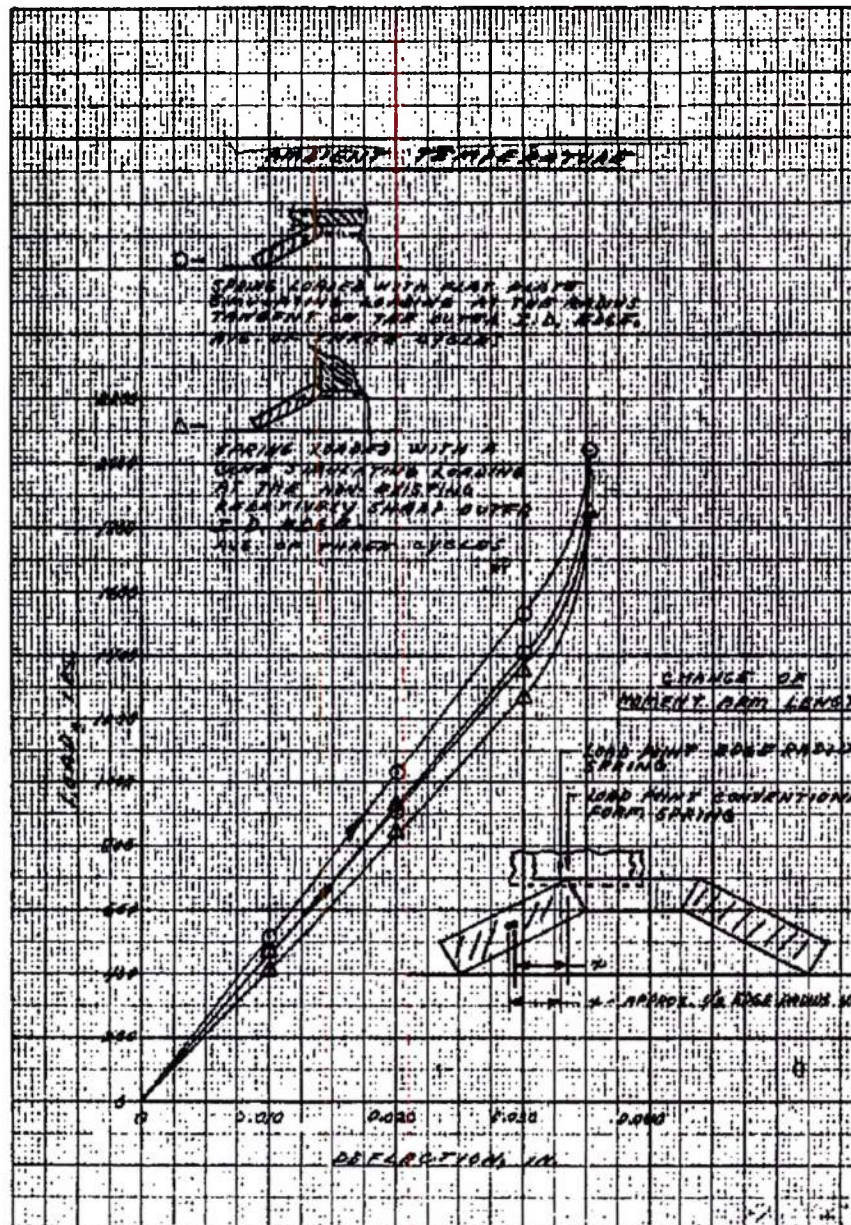


FIGURE 75. Load vs. Deflection: No. 93 Belleville Spring--
 1/16-in. Full Radius Both I.D. Edges (R-235 Alloy)

UNCLASSIFIED

OCT 02 2015

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

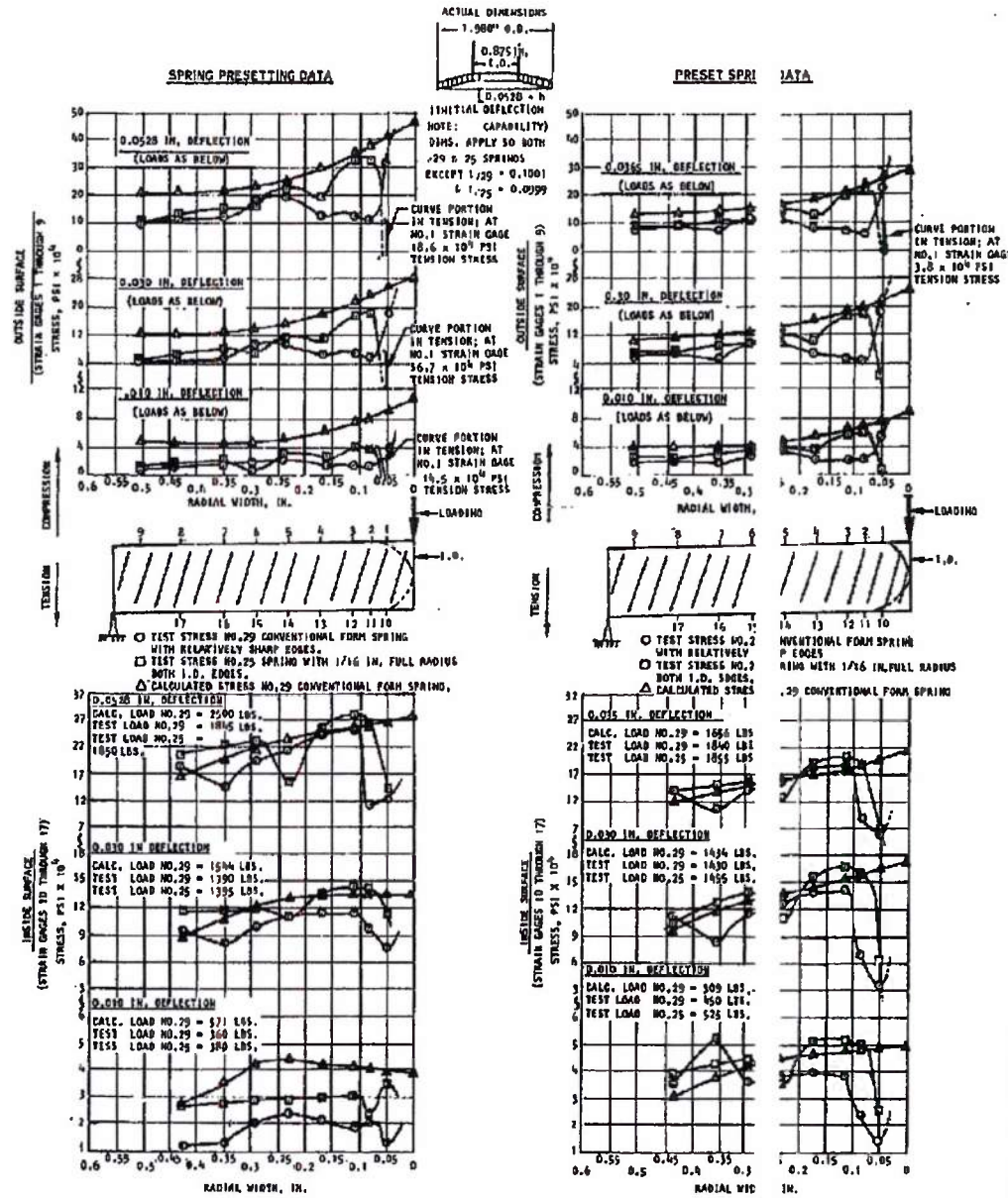
REPORT 6003

FIGURE 76. Stress vs. Spring Radial Width--Belleville Spring No. 29 and No. 25
(R-235 Material; Ambient Temperature; Flat Plate Loading)

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

on 6003

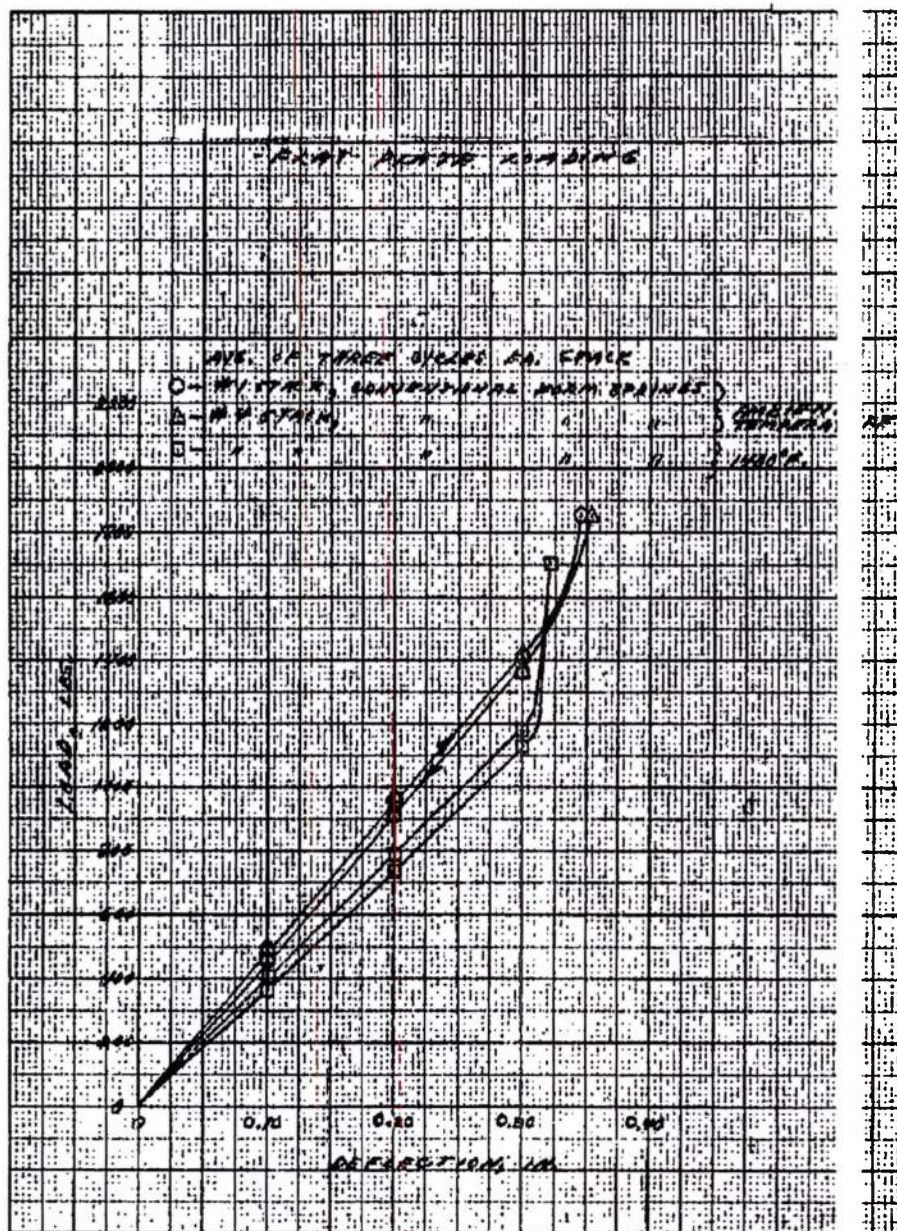


FIGURE 77. Load vs. Deflection: No. 1 and No. 4 10-Spring Series Stack of Belleville Springs (R-235 Alloy)

UNCLASSIFIED

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

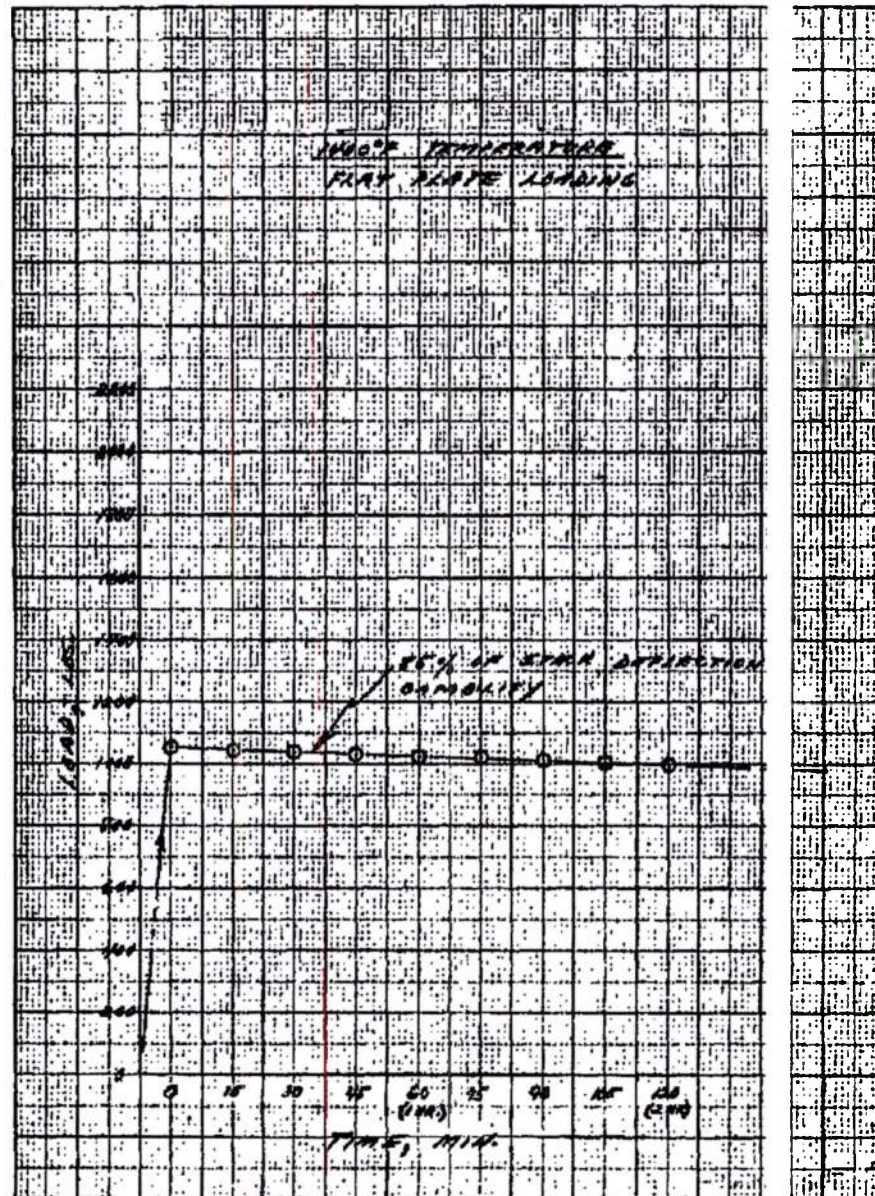


FIGURE 78. Load Loss vs. Time at Constant Spring Deflection:
10-Spring Series Stack of Belleville Springs (R-235 All)

n. 4
)

UNCLASSIFIED

~~CONFIDENTIAL~~

REPORT 6003

ASD-TDR-63-277, Vol. IV

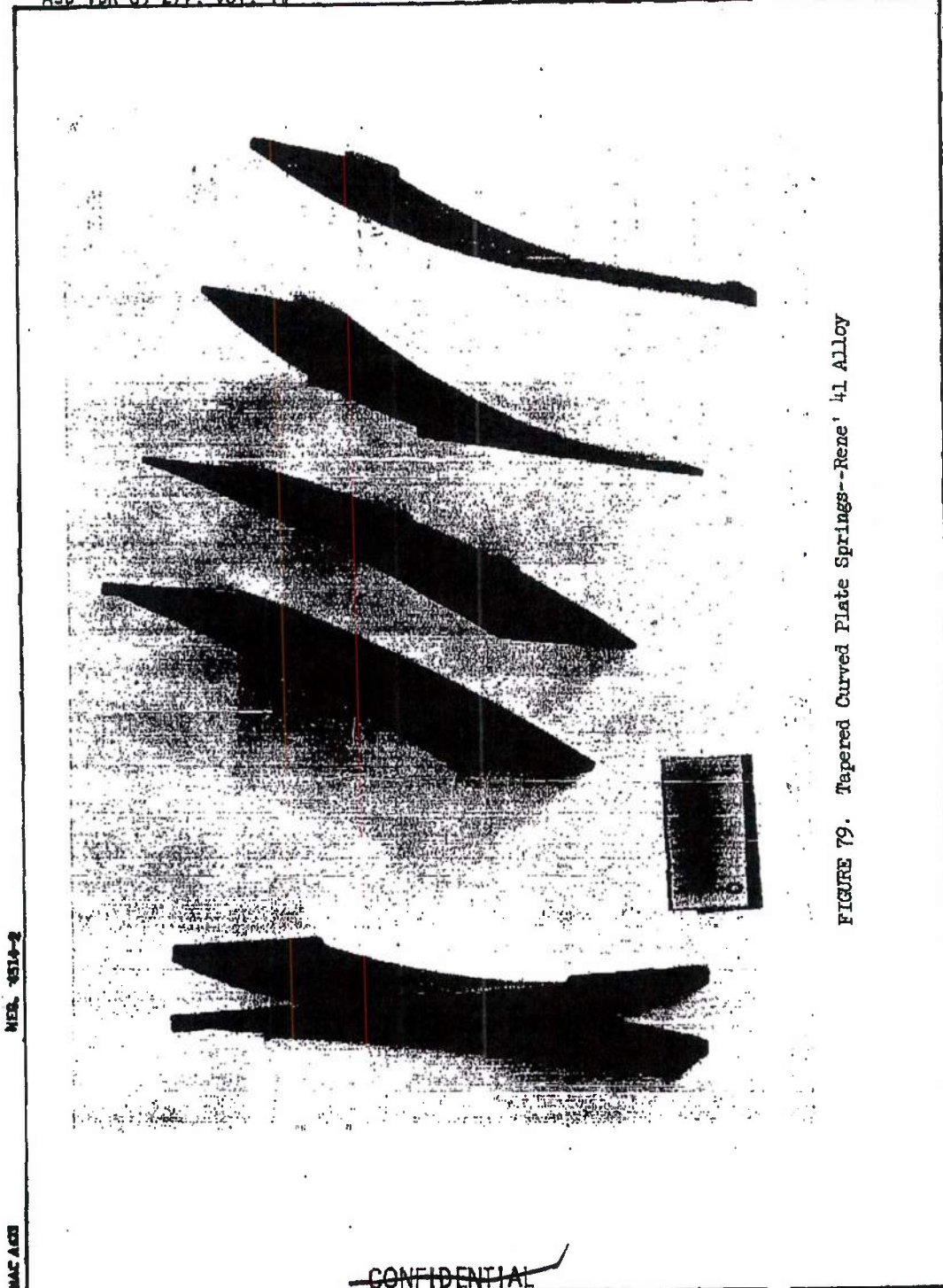


FIGURE 79. Tapered Curved Plate Springs--Rene' 41 Alloy

~~CONFIDENTIAL~~

~~CONFIDENTIAL~~

ASD-TDR-63-277 Vol. 1

REPORT 6003

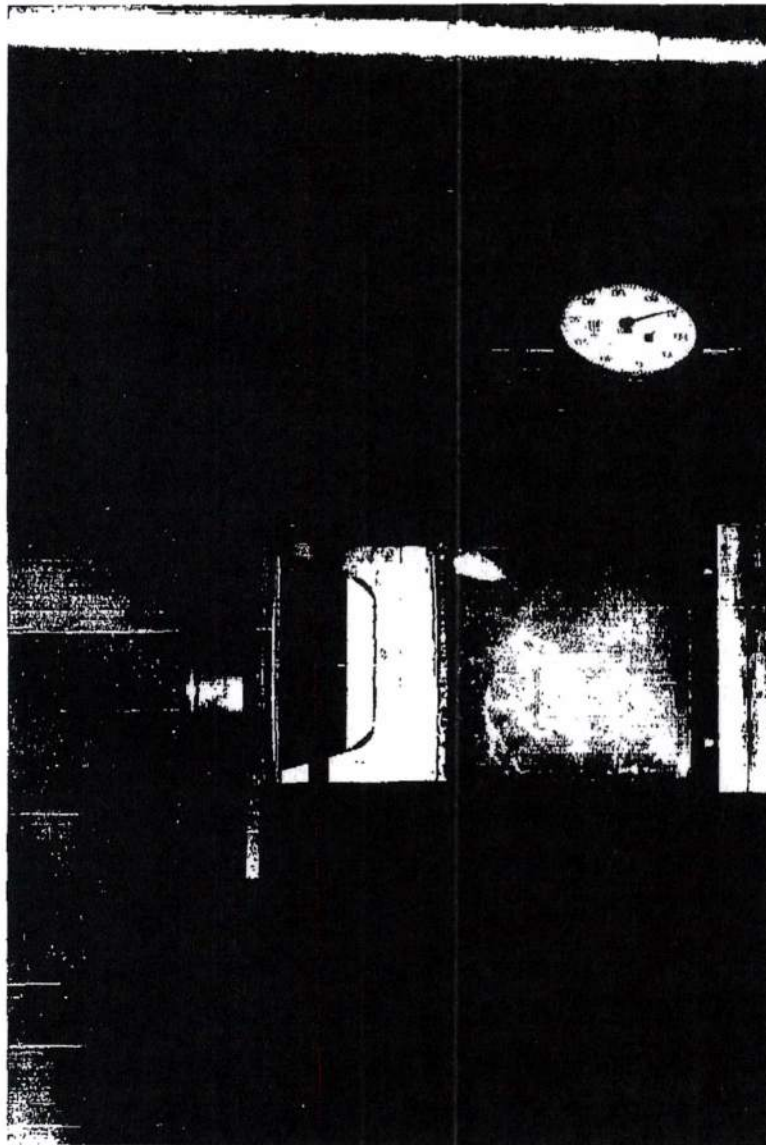


FIGURE 80. Test of Tapered Curved Plate Springs--Spring Stack Profile
at 0.370-inch Deflection--Ambient Temperature

NER, 451A-4

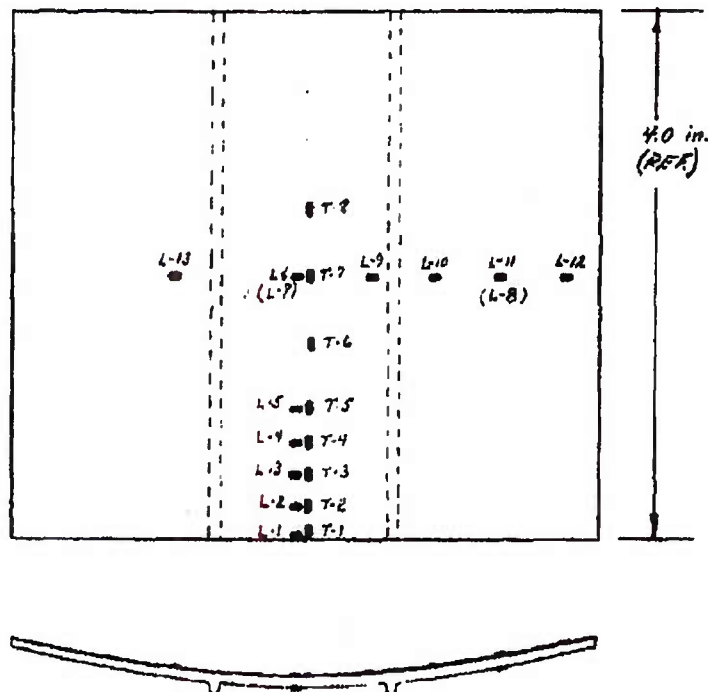
MAC A673

~~CONFIDENTIAL~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003



XB1415-503

NOTES -

1. Installed SR-4 type A-18 strain gages as shown on XB1415-503, -501 & -1.
2. L-7 & L-8 on opposite side of spring from L-6 & L-11, respectively.
3. T-5, T-6 & L-5 not used on -501. T-3, T-4, T-5, T-6, T-8, L-3, L-4 & L-5 not used on -1.
4. L- denotes longitudinal orientation
 T- " " transverse " "

FIGURE 81. Strain Gage Layout for Tapered Curved Plate Spring Test

UNCLASSIFIED

~~CONFIDENTIAL~~

ASD-7DR-63277, Vol. 1

REPORT 6003

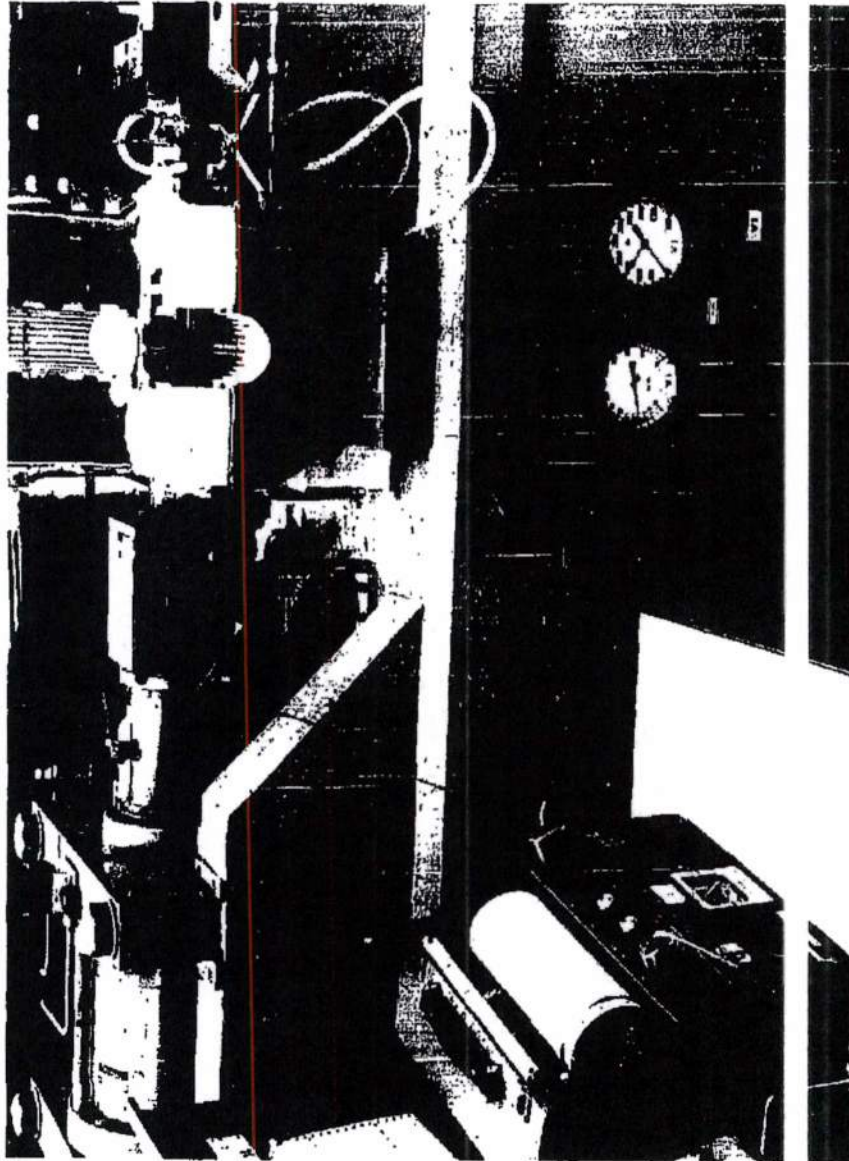


FIGURE 82. Setup for Testing of Tapered Curved Plate Springs
(1400°F Cyclic Load-Deflection)

NEG. 4514-7

MAC A63

~~CONFIDENTIAL~~

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

6003

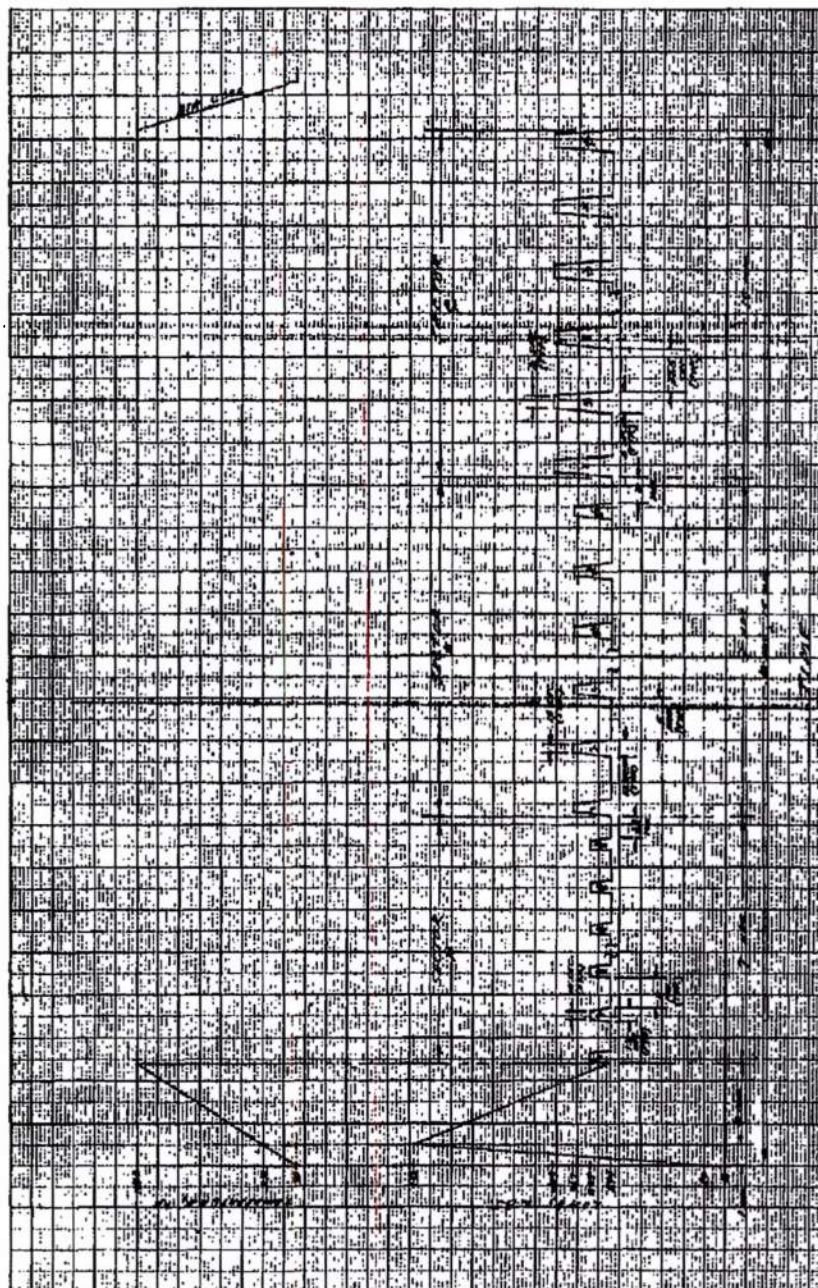


FIGURE 83. Cyclic Load-Deflection for Tapered Curved Plate Springs
(2-in. Wide Stacks, S/N 2 and 4)

~~CONFIDENTIAL~~

22J46

- 236 -

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

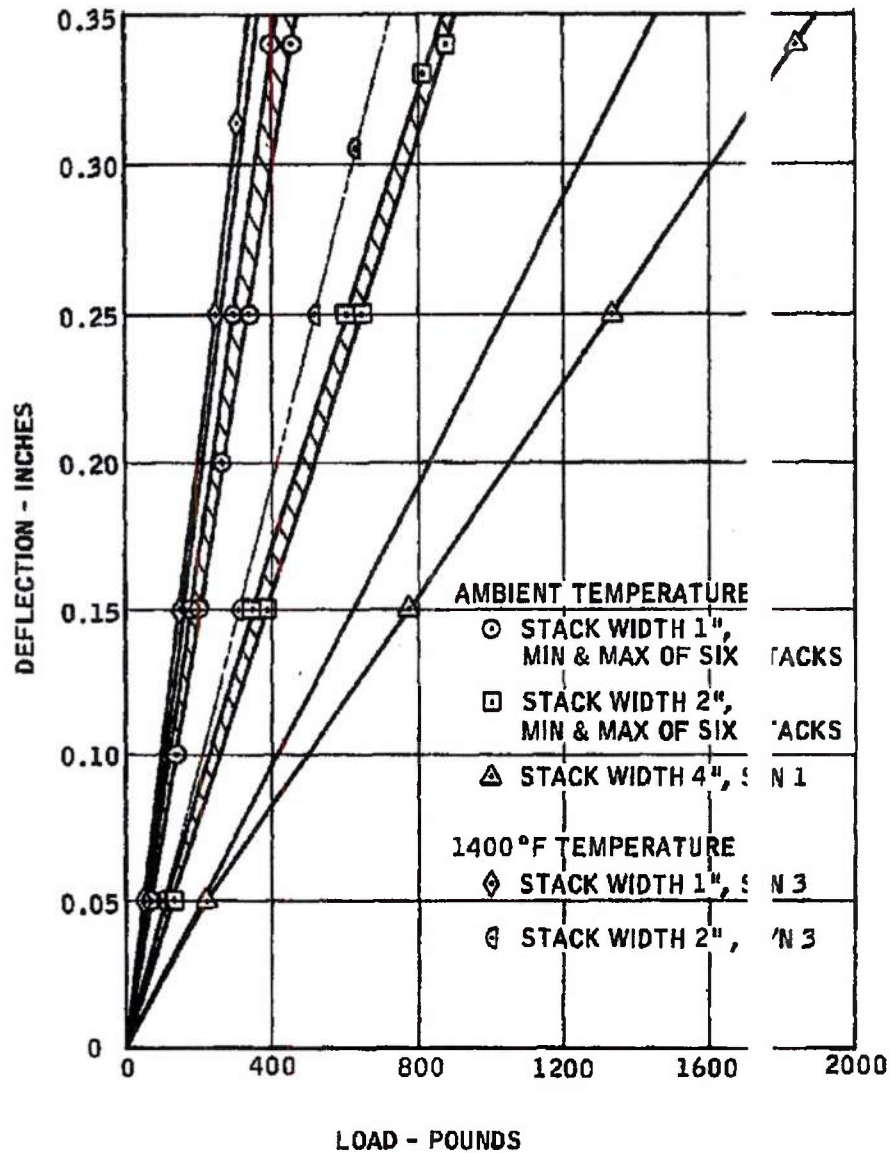
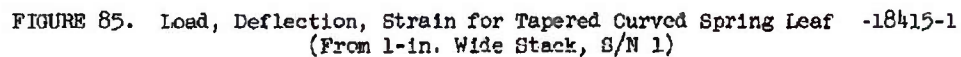


FIGURE 84. Load Deflection Characteristics for Tapered Curved Plate Spring Stack

~~CONFIDENTIAL~~

1 CNT 6003



~~CONFIDENTIAL~~

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

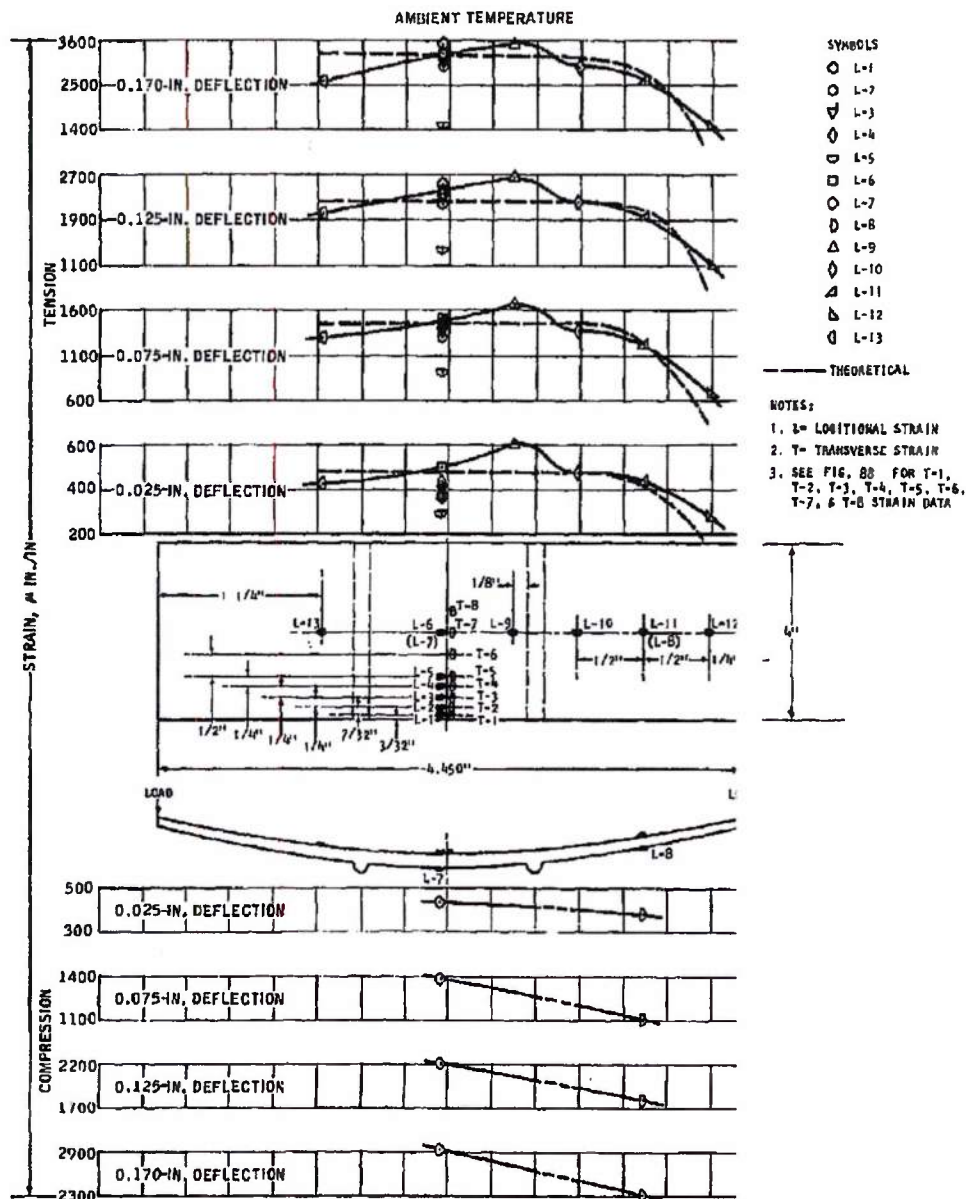


FIGURE 86. Load, Deflection, Strain for Tapered Curved Spring : af X-81415-503
 (From 4-in. Wide Stack, S/N 1)

~~CONFIDENTIAL~~

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

6003

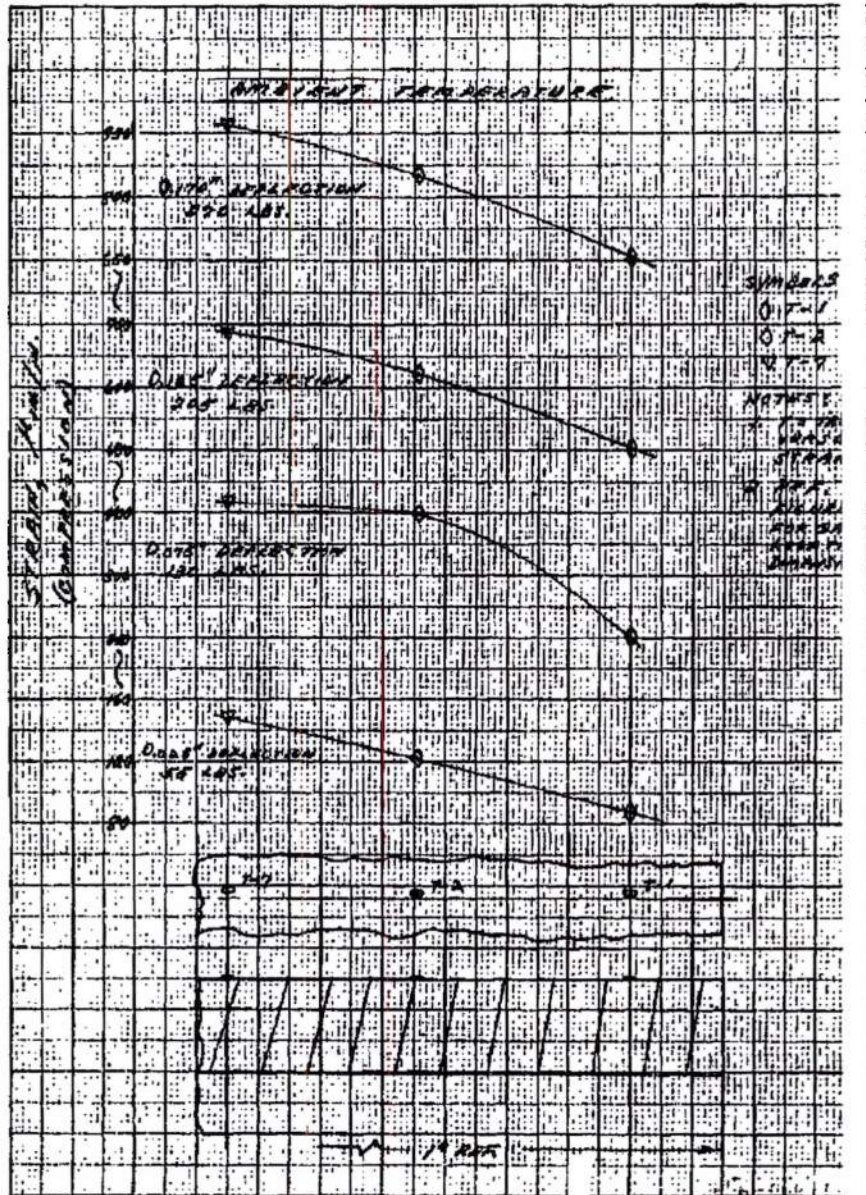


FIGURE 87. Load-Deflection-Strain Data for Tapered Curved Spring Le (X-81415-1) from 1-in. Wide Stack, S/N 1

~~CONFIDENTIAL~~

OCT 02 2015

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

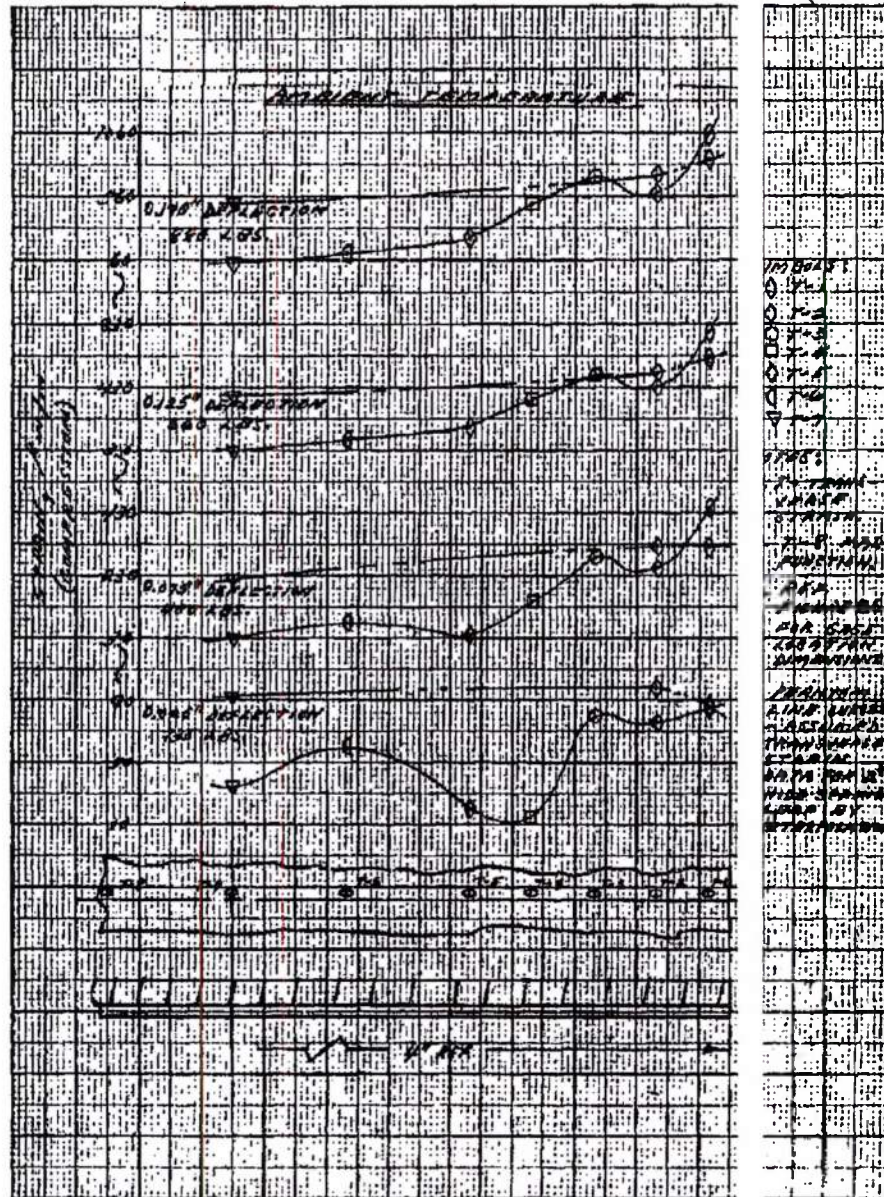


FIGURE 88. Load-Deflection-Strain Data for Tapered Curved (X-181415-503) from 4-in. Wide Stack, S/N

MAC 4603

~~CONFIDENTIAL~~

22J45

- 241 -

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPO

6003

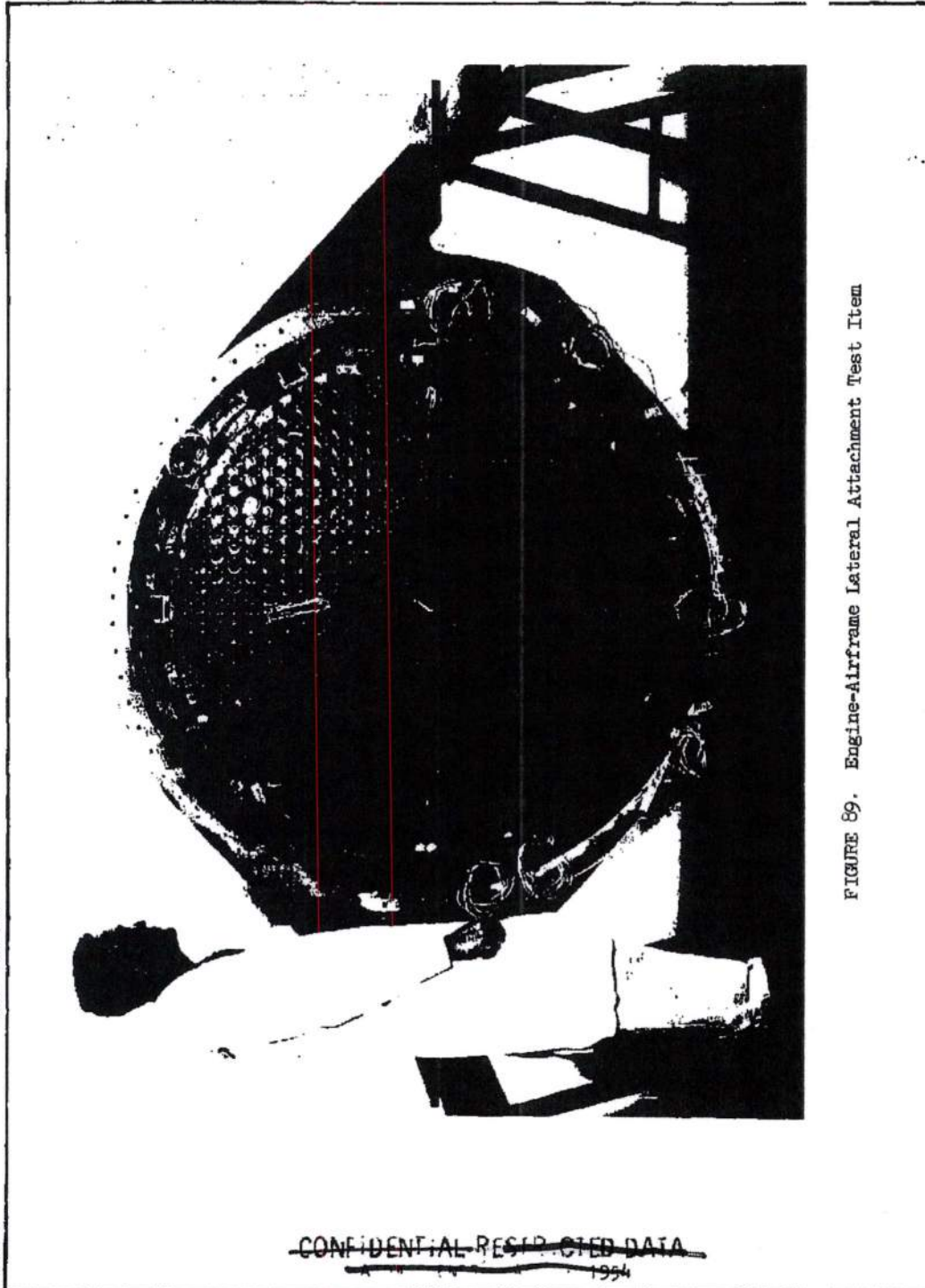


FIGURE 89. Engine-Airframe Lateral Attachment Test Item

NEG. 4142-13

MAC 1670

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~CONFIDENTIAL RESTRICTED DATA~~
~~1954-1958~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

NEG. 4142-14

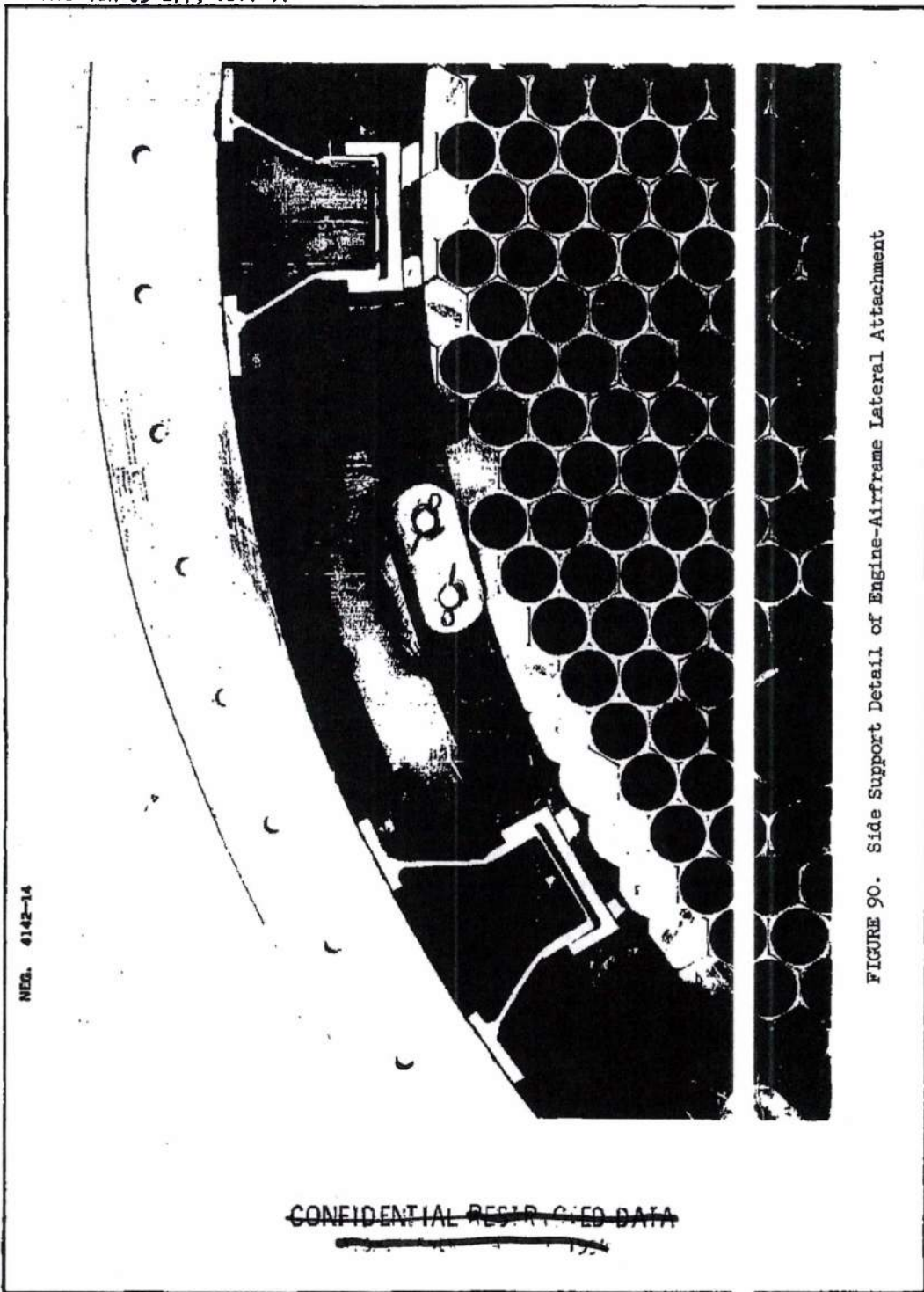


FIGURE 90. Side Support Detail of Engine-Airframe Lateral Attachment

~~CONFIDENTIAL RESTRICTED DATA~~
~~1954-1958~~

MAC A673

~~CONFIDENTIAL RESTRICTED DATA~~
~~NO FORN DISSEM 1954~~

ASD-TDR-63-272, Vol. 12

REFC 6003

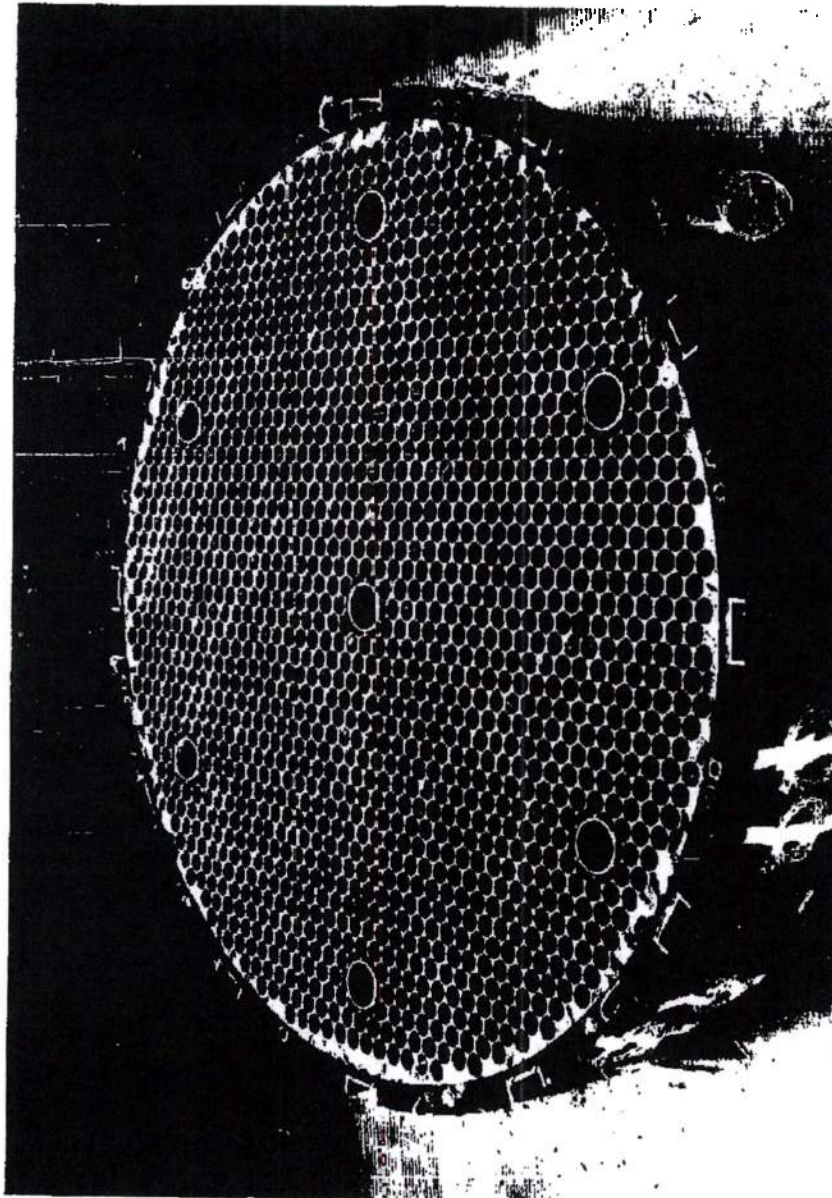


FIGURE 91. Core Matrix and Expansion Shell

4142-9

MAC 1672

~~CONFIDENTIAL RESTRICTED DATA~~
~~NO FORN DISSEM 1954~~

pl.

~~CONFIDENTIAL RESTRICTED DATA~~
1954

ASD-TDR-63-277, Vol. 17

REPORT 6003

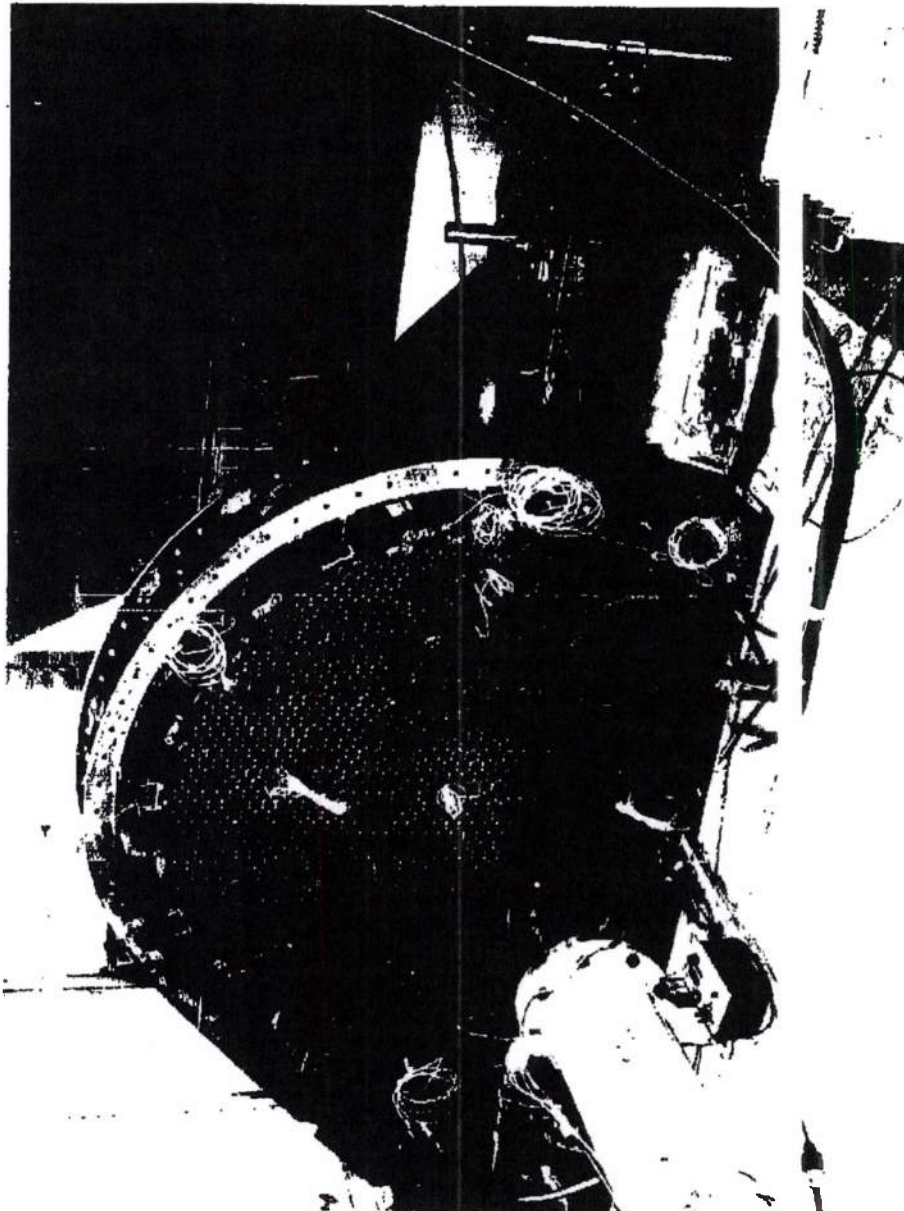


FIGURE 92. Engine-Airframe Lateral Attachment Test Item in Shaker Facility

NEG. 4342-17

MAC 672

~~CONFIDENTIAL RESTRICTED DATA~~
1954

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

NT 6003

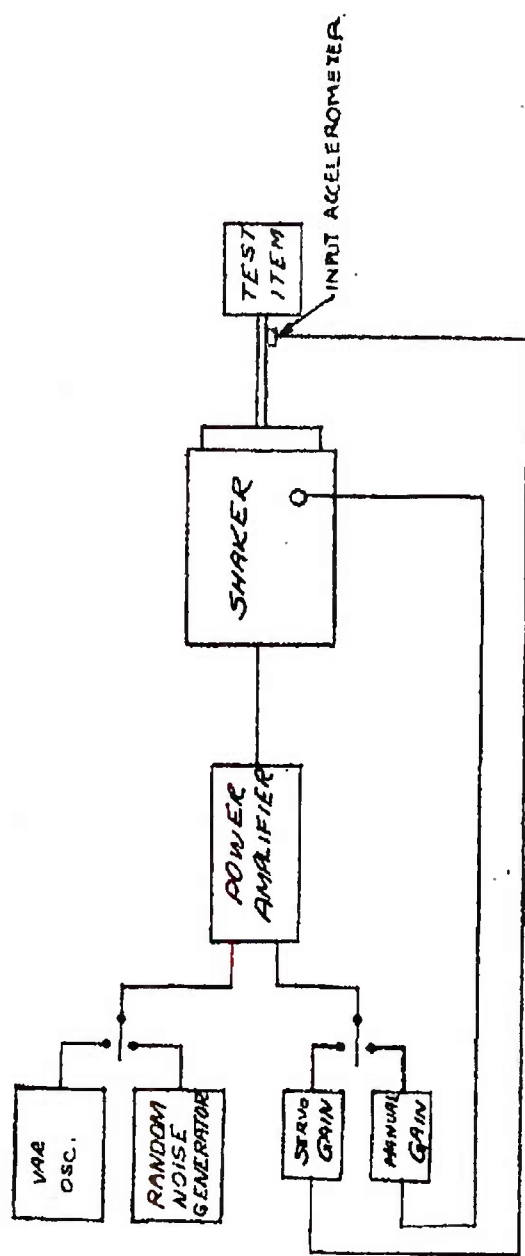


FIGURE 93. Block Diagram of Shaker Control System

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC AG2

N22K25

UNCLASSIFIED

ASD-TDR-63-277, Vol. 1

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

MAC 6027

NEG-4142-18



FIGURE 94. Engine-Airframe Lateral Attachment Test Item in Furnace

UNCLASSIFIED

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

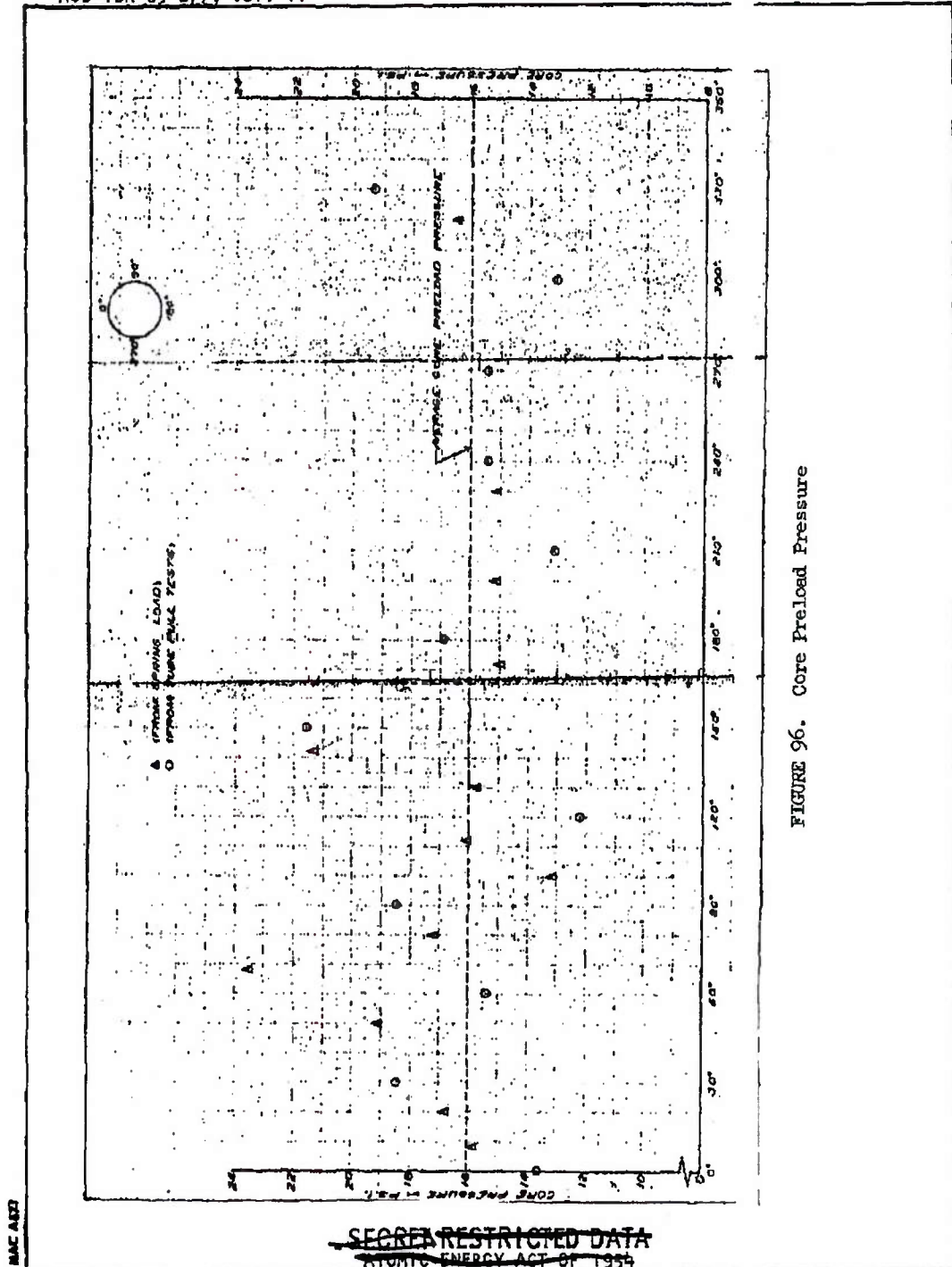


FIGURE 96. Core Preload Pressure

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E842

- 249 -

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

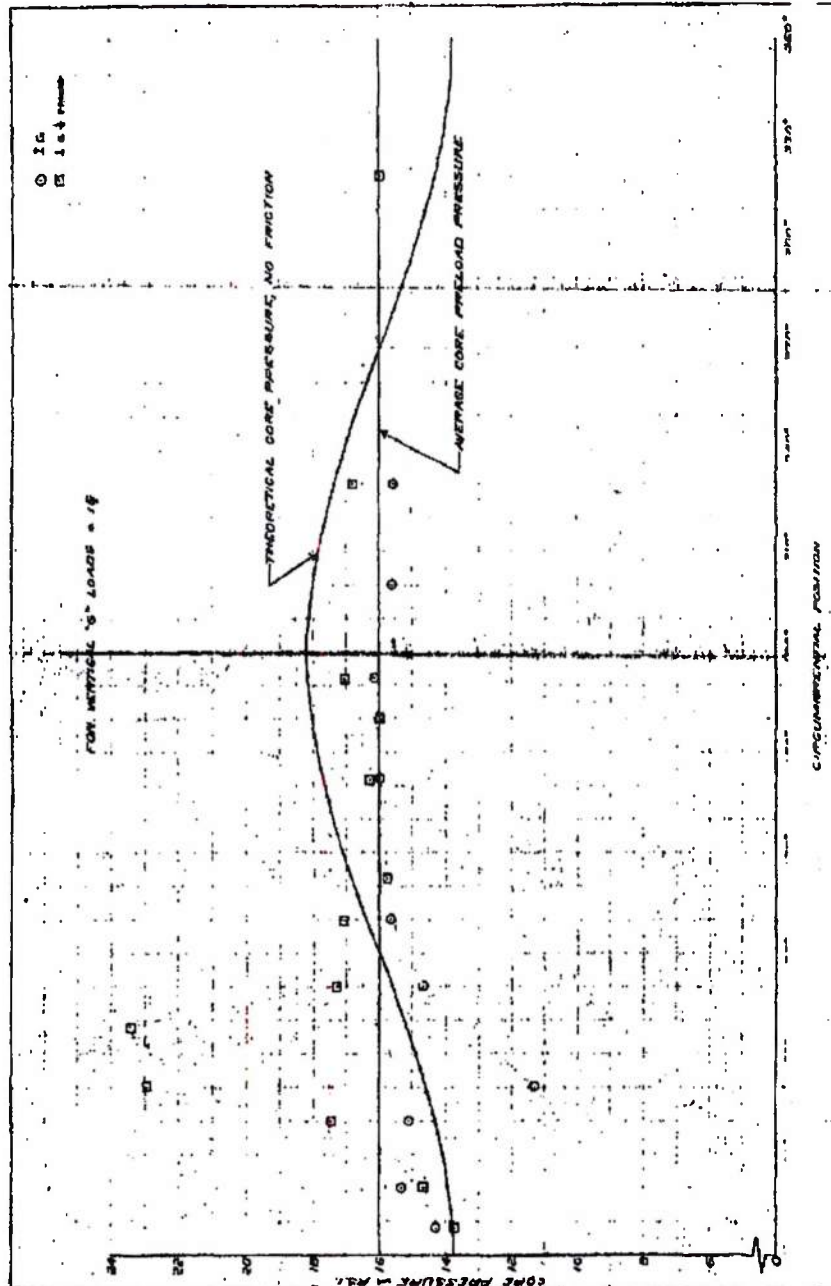


FIGURE 97. Core Pressure (From Spring Loads)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E841

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

REPORT 6003

ASD-TDR-63-277, Vol. IV

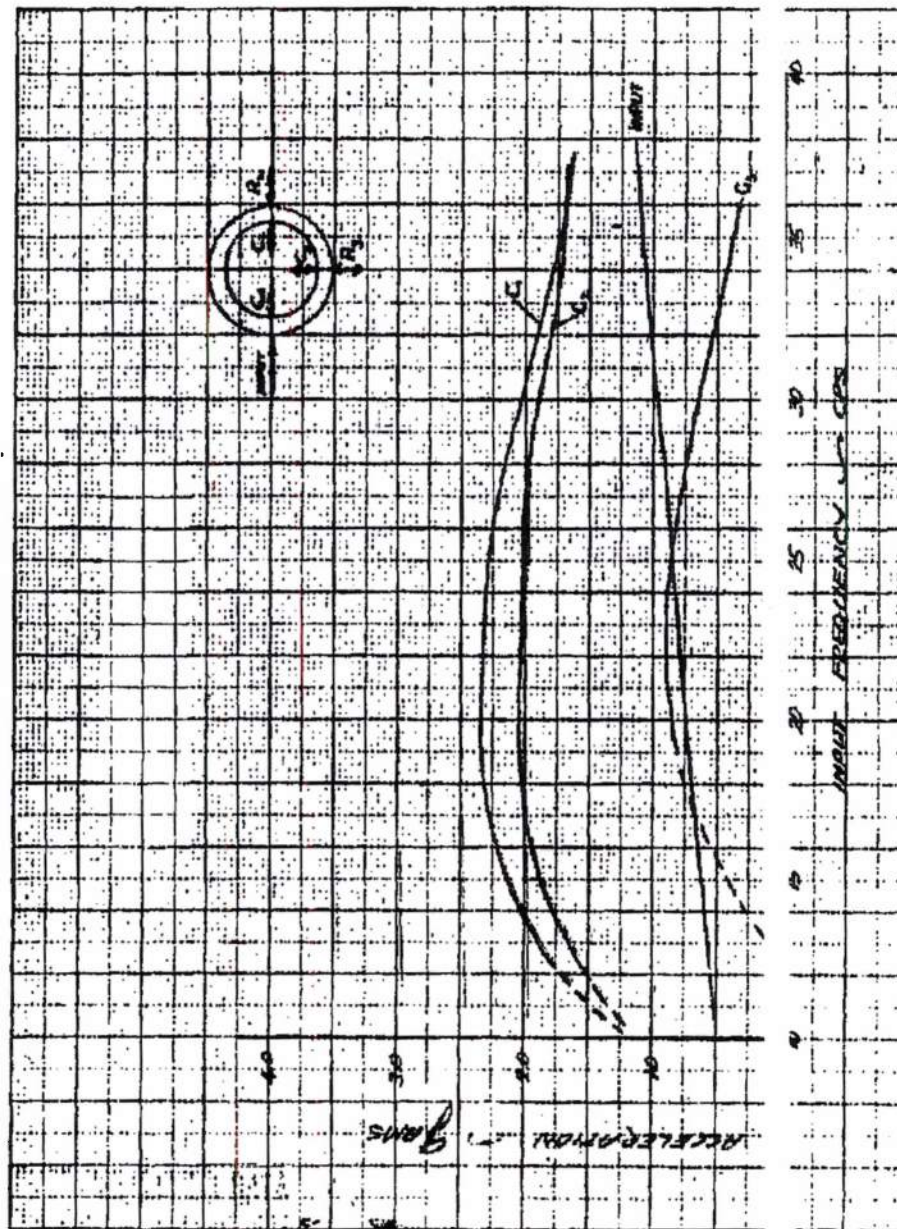


FIGURE 98. Core Dynamic Response (Run No. 4)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E957

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

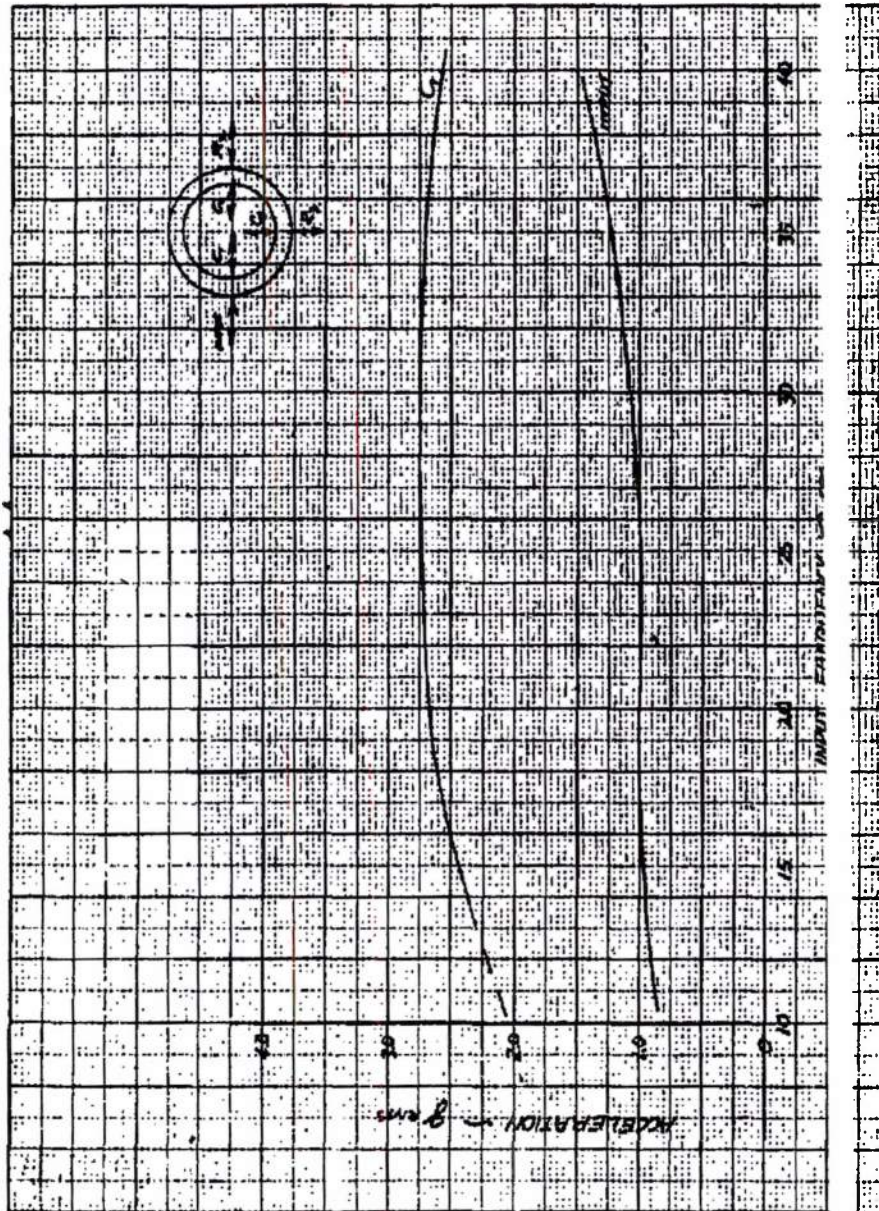


FIGURE 99. Core Dynamic Response (Run No. 10)

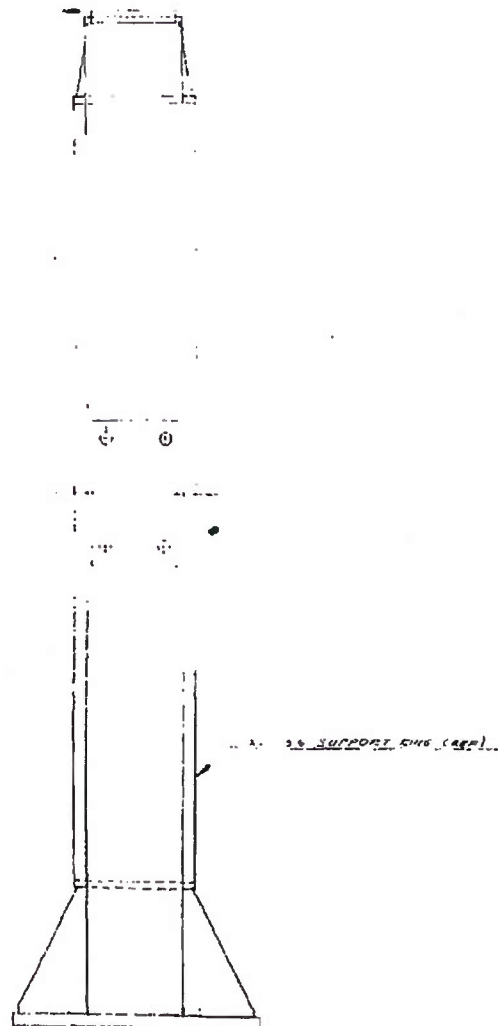
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

N22E956

ASD-TDR-63-277, Vol. IV
~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

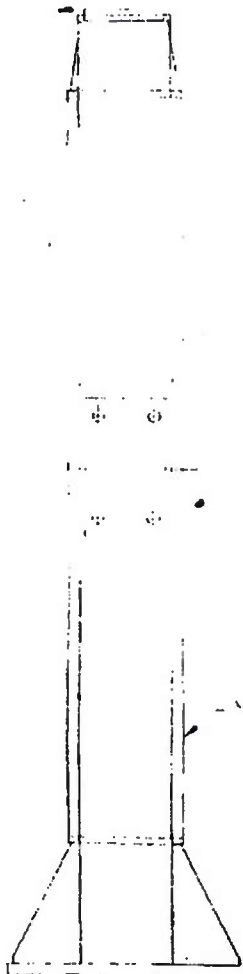
1



~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

2

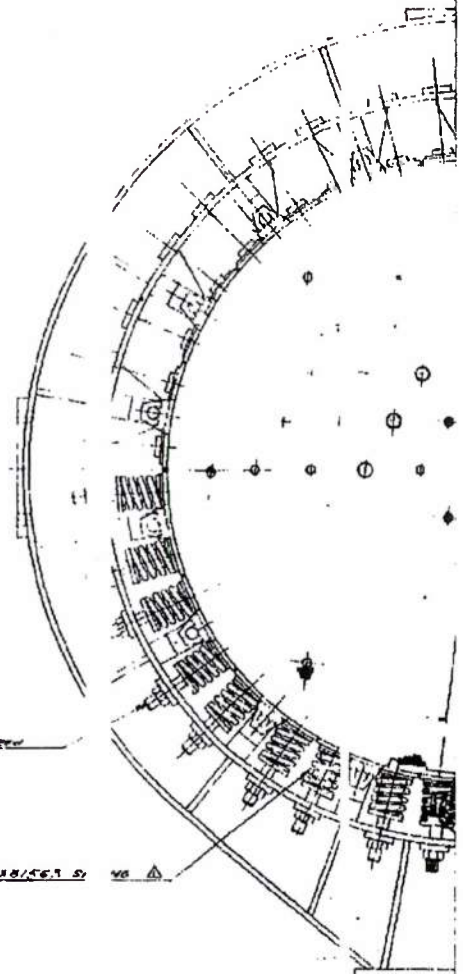
Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015



1/4" X 1/2" SUPPORT SING (SEE 1)

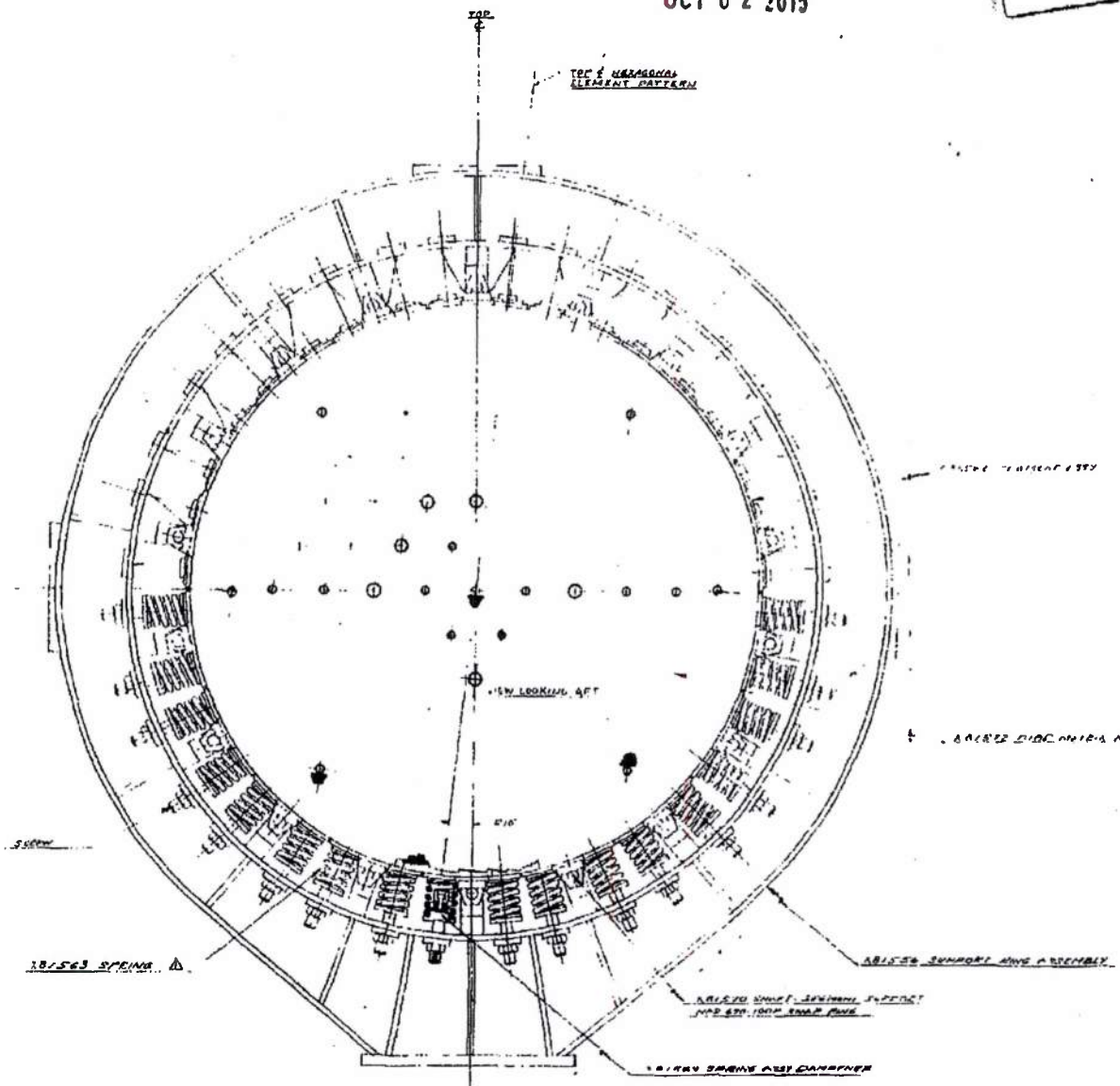
1,000-16.0A, -9.0V HIGH-SOCKET TYPE OFT SCREW
LOAD 1/2" X 1/2" AREA 1/2" X 1/2"

1/2" X 1/2" S



OCT 02 2015

3





DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 100. La rai Attach Assembly
Engl: -Airframe

Δ SPRING MUST BE INDIVIDUALLY CALIBRATED ON
PRESSURE IN FINAL ASSEMBLY AT 1400 PSI.
NOTES: UNLESS OTHERWISE SPECIFIED

REPO

ARMED DANGER ACT OF 92
6003

RECEIVED
DATE
TIME

5

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 100. Lateral Attach Assembly,
Engine-Airframe

⚠ SPRING MUST BE INDIVIDUALLY CALIBRATED AND
RE-TESTED IN FINAL ASSEMBLY AT 114 POUNDS.
NOTES: UNLESS OTHERWISE SPECIFIED

ITEM	QTY	DESCRIPTION
1	1	SPRING
2	1	WASHER
3	1	NUT
4	1	SCREW
5	1	WASHER
6	1	NUT
7	1	SCREW
8	1	WASHER
9	1	NUT
10	1	SCREW

11	1	SPRING
12	1	WASHER
13	1	NUT
14	1	SCREW
15	1	WASHER
16	1	NUT
17	1	SCREW
18	1	WASHER
19	1	NUT
20	1	SCREW

21	1	SPRING
22	1	WASHER
23	1	NUT
24	1	SCREW
25	1	WASHER
26	1	NUT
27	1	SCREW
28	1	WASHER
29	1	NUT
30	1	SCREW

CONFIDENTIAL DETERMINED DATA
ARMED DANGER ACT OF 1994

31	1	SPRING
32	1	WASHER
33	1	NUT
34	1	SCREW
35	1	WASHER
36	1	NUT
37	1	SCREW
38	1	WASHER
39	1	NUT
40	1	SCREW

CONFIDENTIAL
X01535
A

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

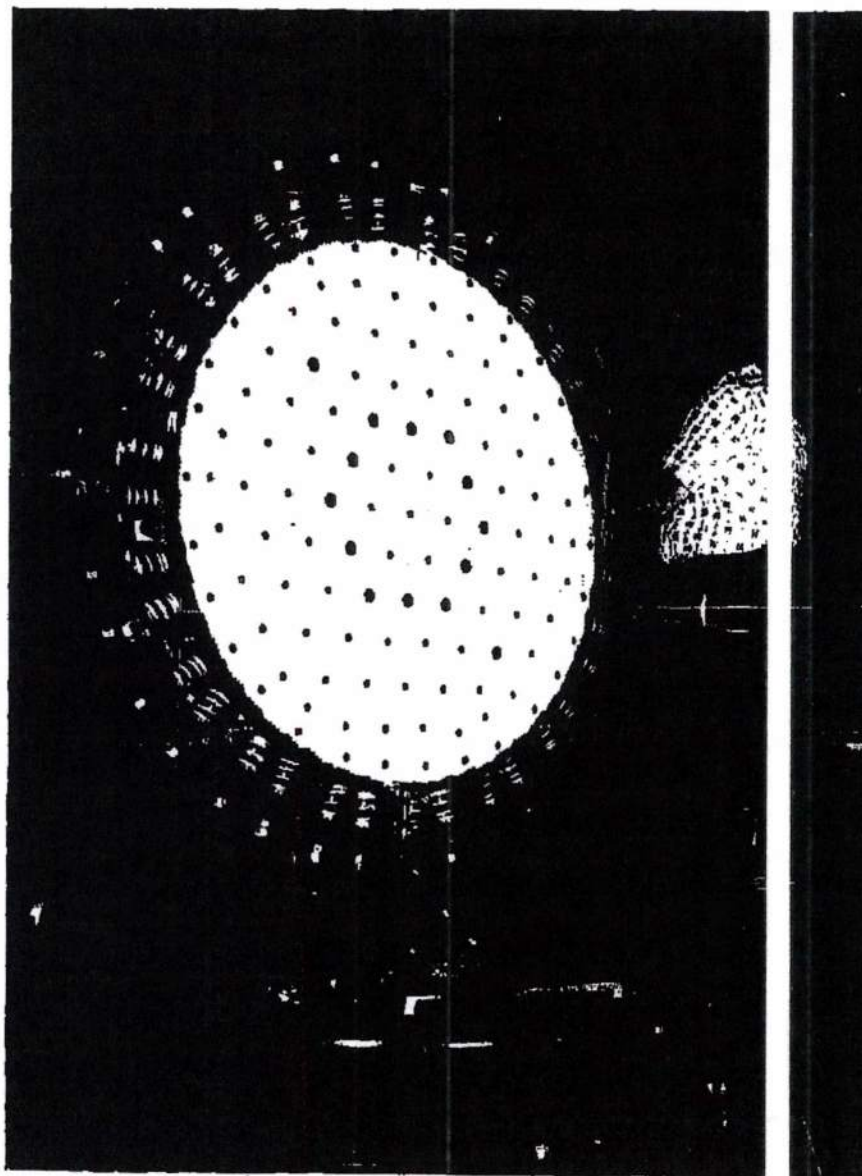


FIGURE 101. Test Item for Engine-Airframe Lateral Attachment Test
(Phase II)

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

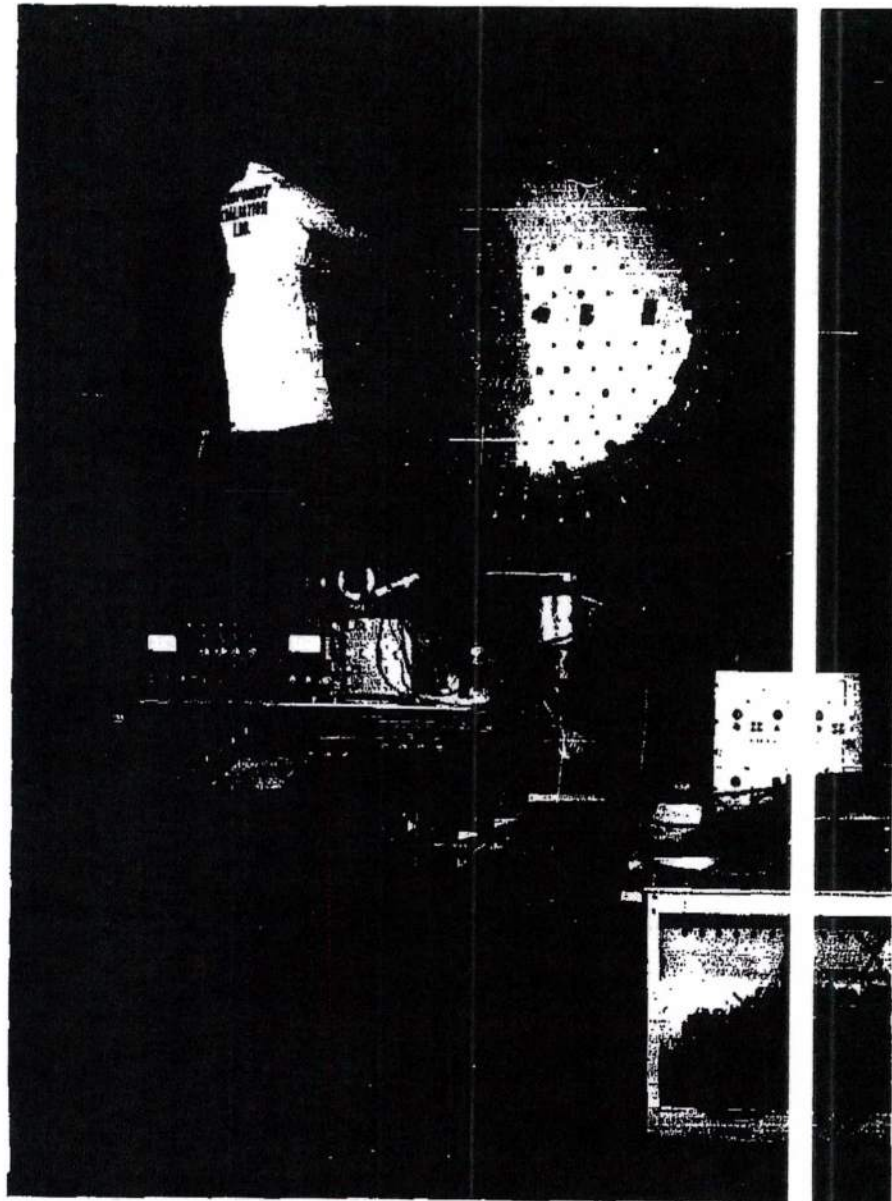


FIGURE 102. Vibration Test Setup for Engine-Airframe
Lateral Attachment Test (Phase I.)

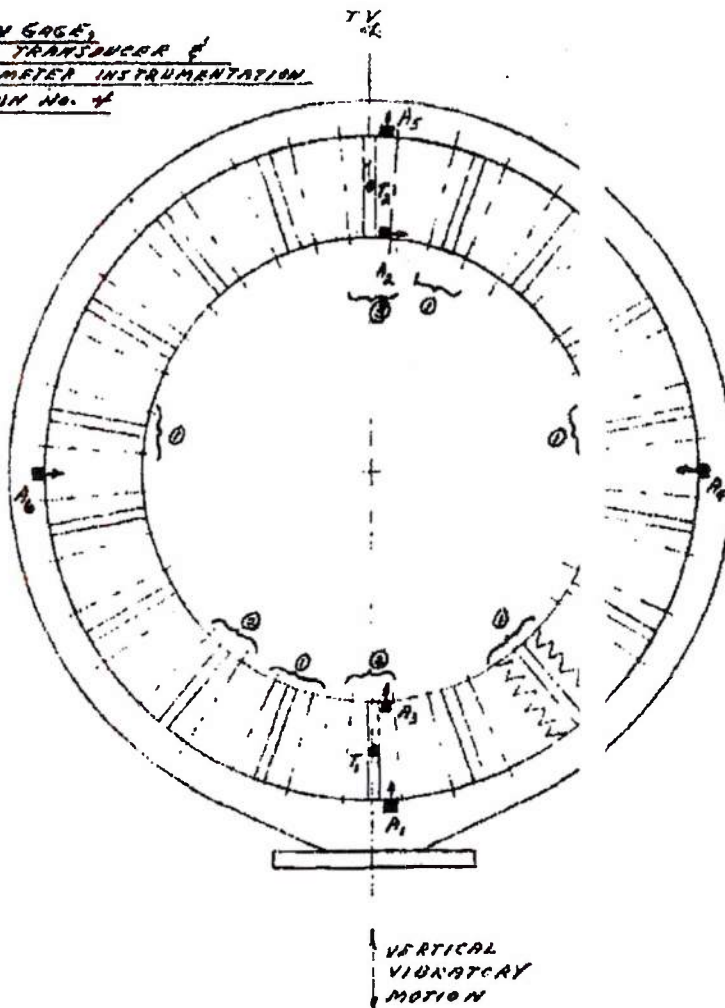
~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

STRAIN GAGE,
MOTION TRANSDUCER &
ACCELEROMETER INSTRUMENTATION
FOR RUN NO. 4



NOTES:

1. $\bullet$ — MOTION TRANSDUCERS (DISPLACEMENT), T_1 & T_2 , ACROSS SPRING BAR.
2. A — ACCELEROMETERS, A_1 THROUGH A_6 .
3. O — LOCATION & NUMBER OF STRAIN GAGED SPRINGS MONITORED.

FIGURE 103. Instrumentation Requirements for Engine-Airframe Lateral Attachment Test (Phase II)

UNCLASSIFIED

22J56

- 257 -

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

ASD-TDR-63-277, Vol. IV

IIPOI

6003

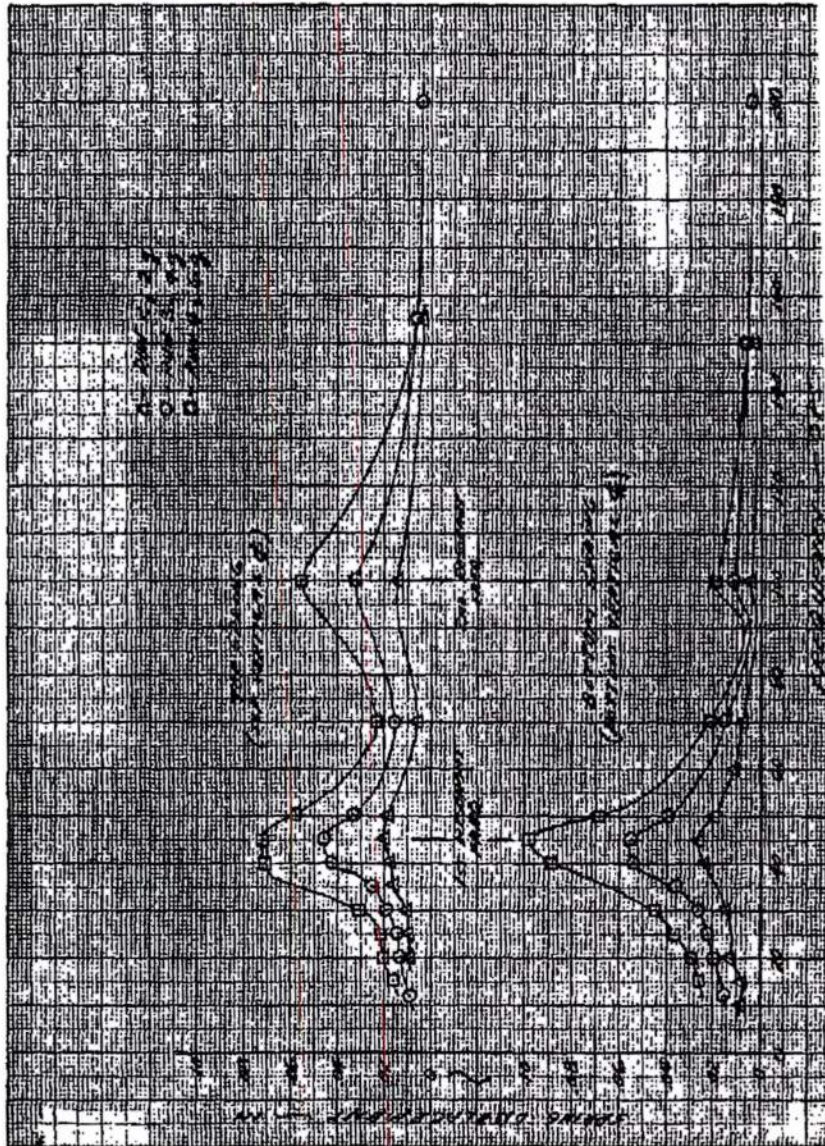


FIGURE 10L. Spring Displacement vs. Frequency (15 psi Core Preload)

~~SECRET RESTRICTED DATA~~

MAC 6023

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

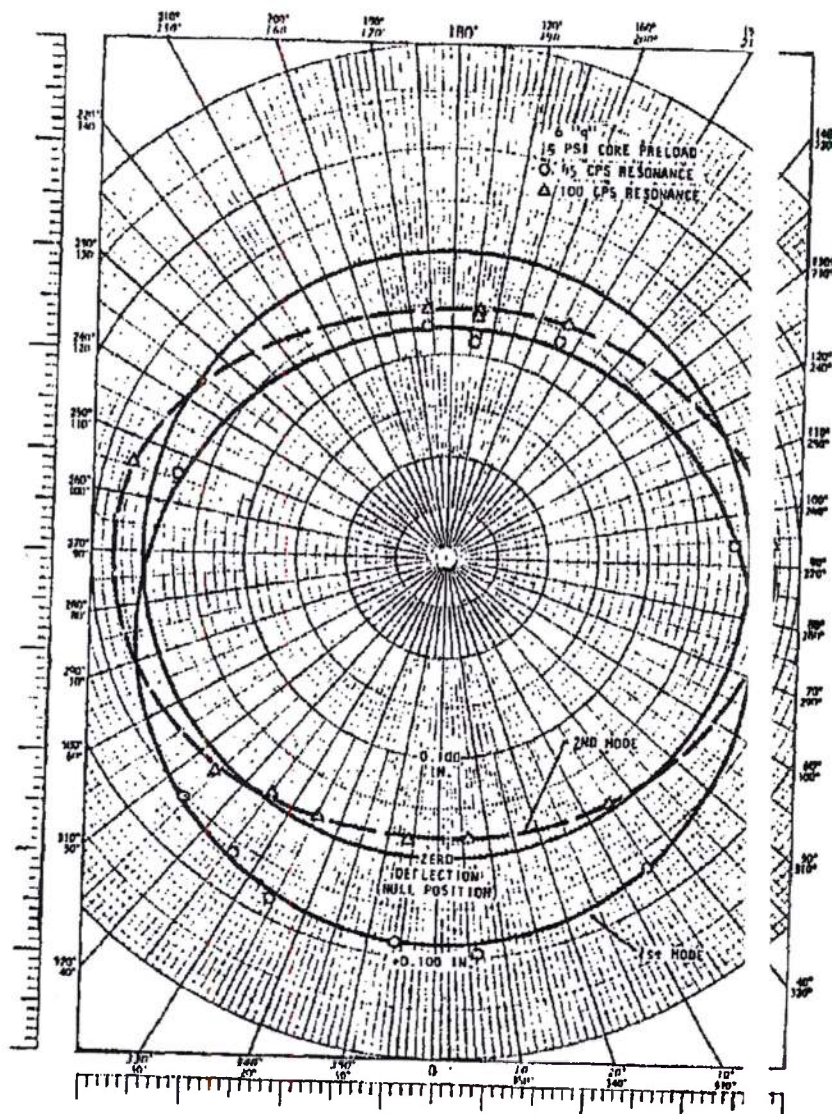


FIGURE 105. Core Deflection Pattern, Run 4

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E782

~~CONFIDENTIAL RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

ASD-TCR-63-277, Vol. 17

REPORT

6003

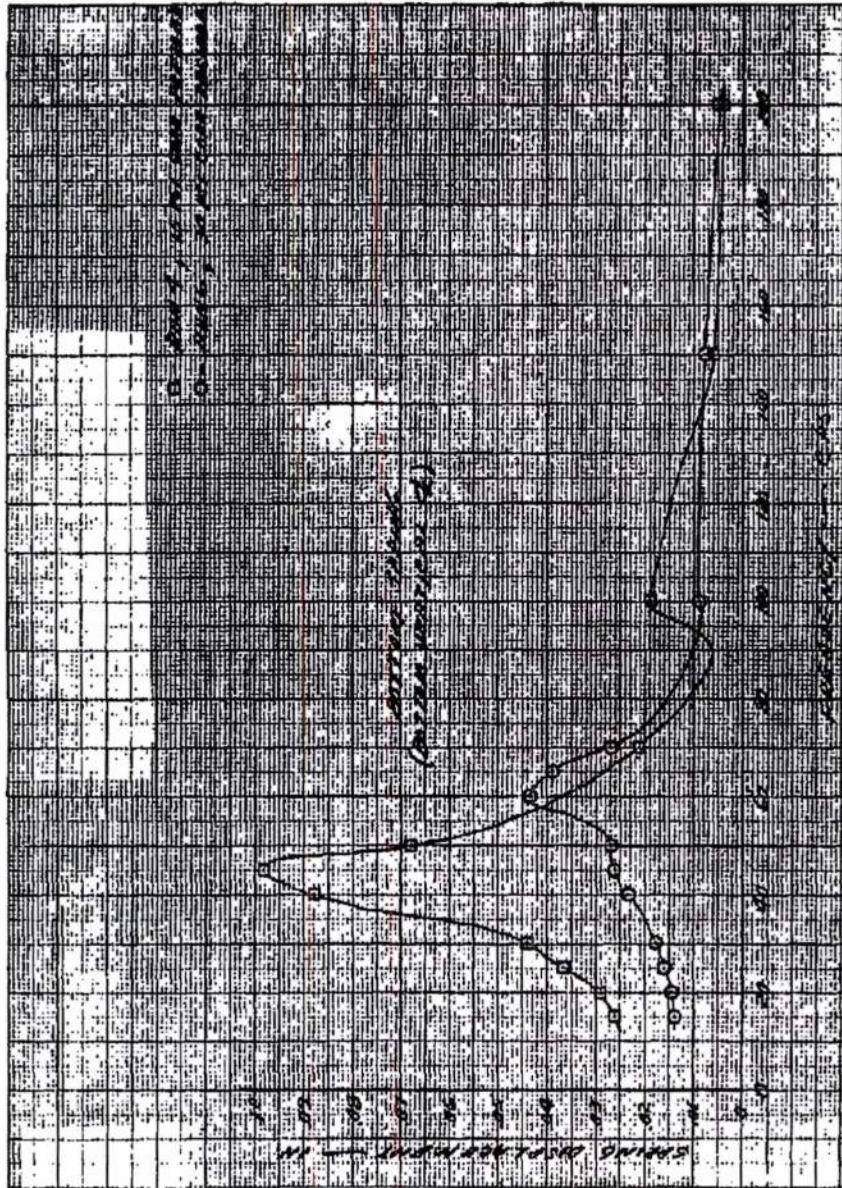


FIGURE 106. Spring Displacement vs. Frequency (6g Force Input)

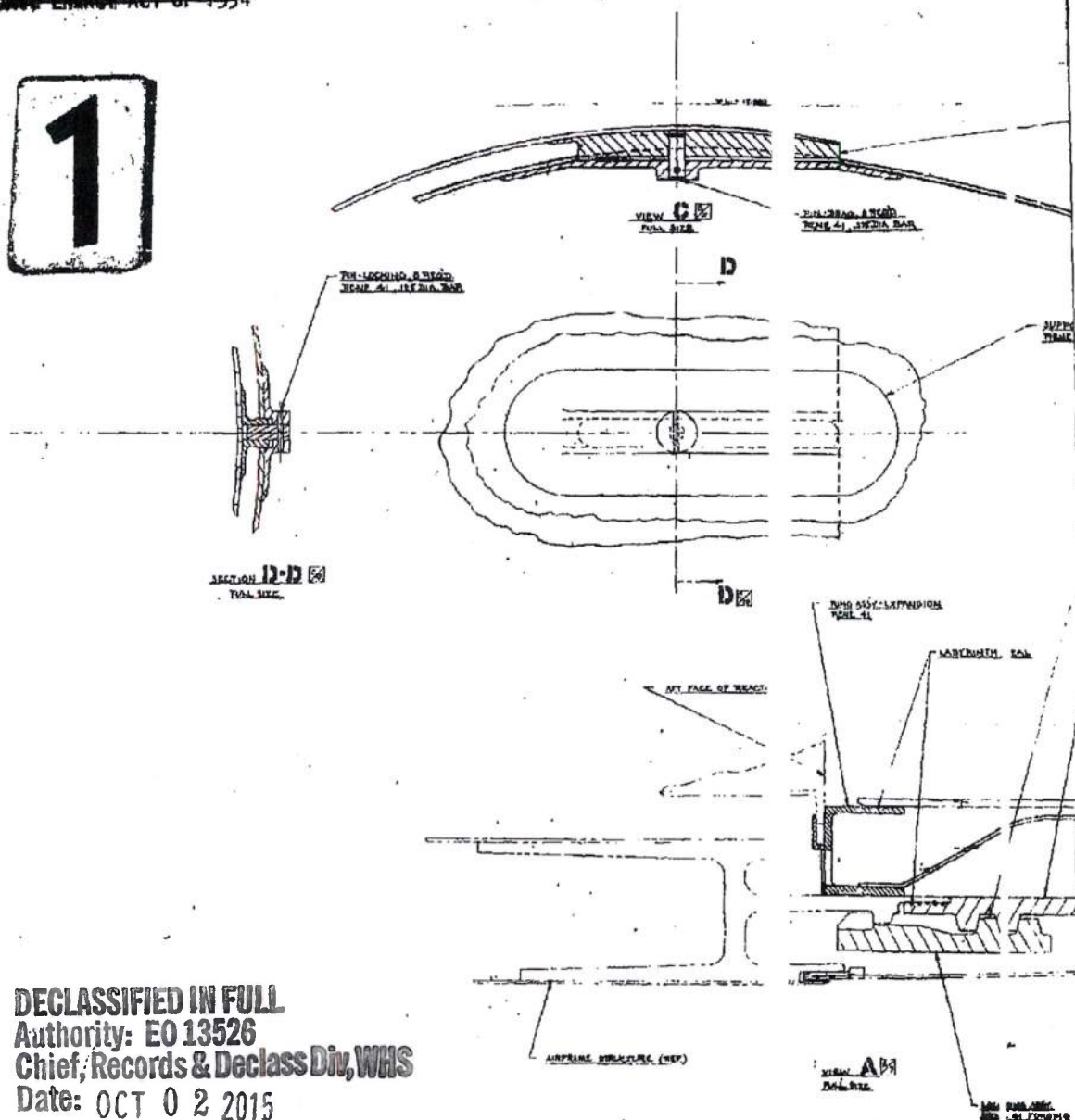
~~CONFIDENTIAL RESTRICTED DATA~~
 ATOMIC ENERGY ACT OF 1954

MAC 100

22J57

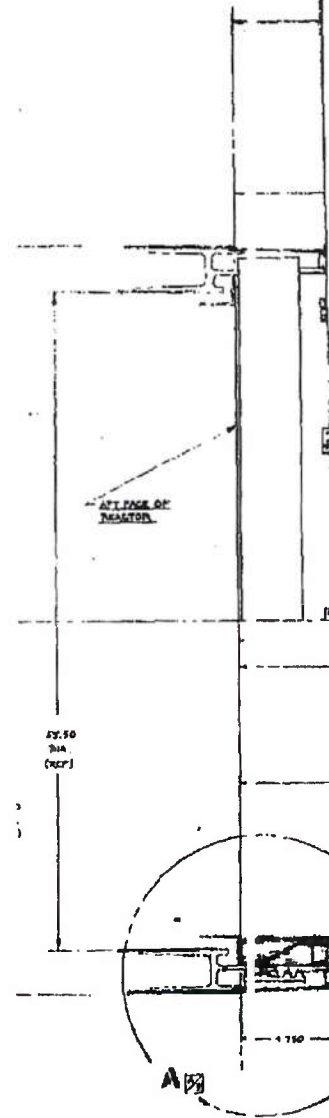
- 260 -

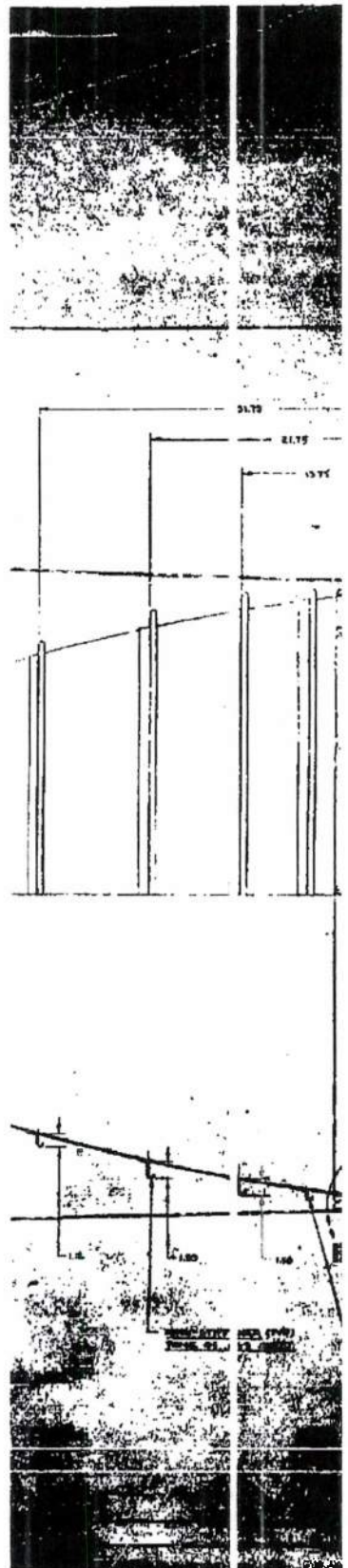
1



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WWS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~





Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

5

- ① STRAIGHT LINE, SLOPE = 0
 ② 2.75 RADIUS
 ③ STRAIGHT LINE, SLOPE = 0
 ④ 2.75 RADIUS, SLOPE = 0
 ⑤ STRAIGHT LINE, SLOPE = 0
 ⑥ 2.75 RADIUS

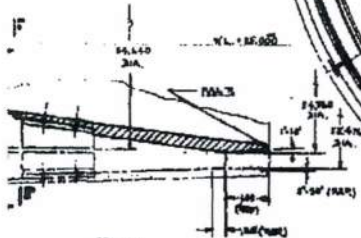
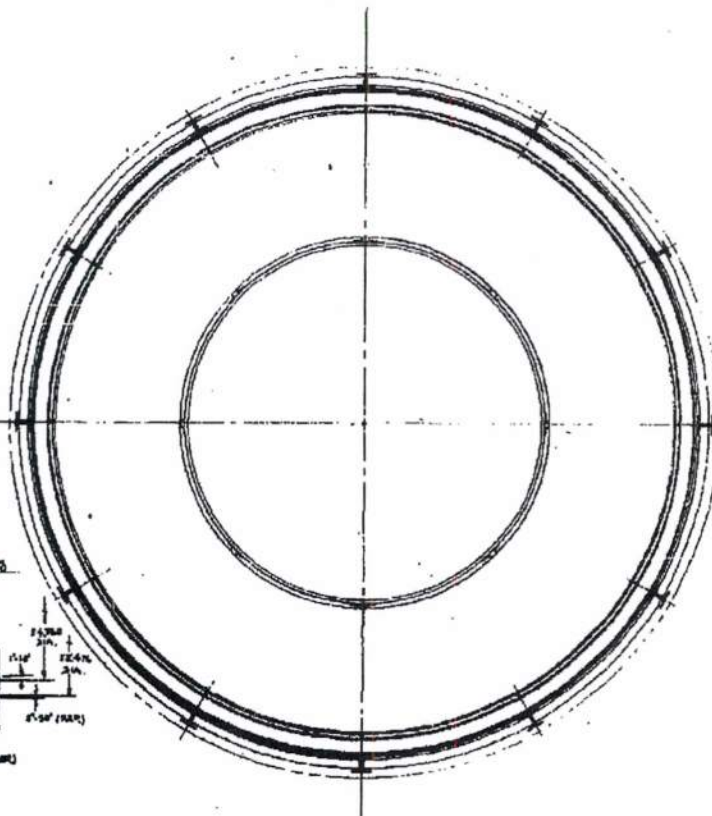


FIGURE 107 Design Layout
 Propellant System Exit Nozzle

UNCLASSIFIED DATA FURNISHING AUTHORITY ACTS
 DIRECTIVE SHALL BE OBSERVED. THE
 INFORMATION CONTAINED HEREIN IS UNCLASSIFIED
 EXCEPT WHERE SHOWN OTHERWISE. IT IS THE POLICY OF THE
 DEPARTMENT OF DEFENSE TO RELEASE THIS INFORMATION TO THE
 MAXIMUM EXTENT POSSIBLE.

UNCLASSIFIED DATA FURNISHING AUTHORITY ACTS
 DIRECTIVE SHALL BE OBSERVED. THE
 INFORMATION CONTAINED HEREIN IS UNCLASSIFIED
 EXCEPT WHERE SHOWN OTHERWISE. IT IS THE POLICY OF THE
 DEPARTMENT OF DEFENSE TO RELEASE THIS INFORMATION TO THE
 MAXIMUM EXTENT POSSIBLE.

NO.	DESCRIPTION	DATE	BY
1	DESIGN LAYOUT	1964	WHS
2	PROPELLANT SYSTEM	1964	WHS
3	EXIT NOZZLE	1964	WHS

NO.	DESCRIPTION	DATE	BY
1	DESIGN LAYOUT	1964	WHS
2	PROPELLANT SYSTEM	1964	WHS
3	EXIT NOZZLE	1964	WHS

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

MAC 1632

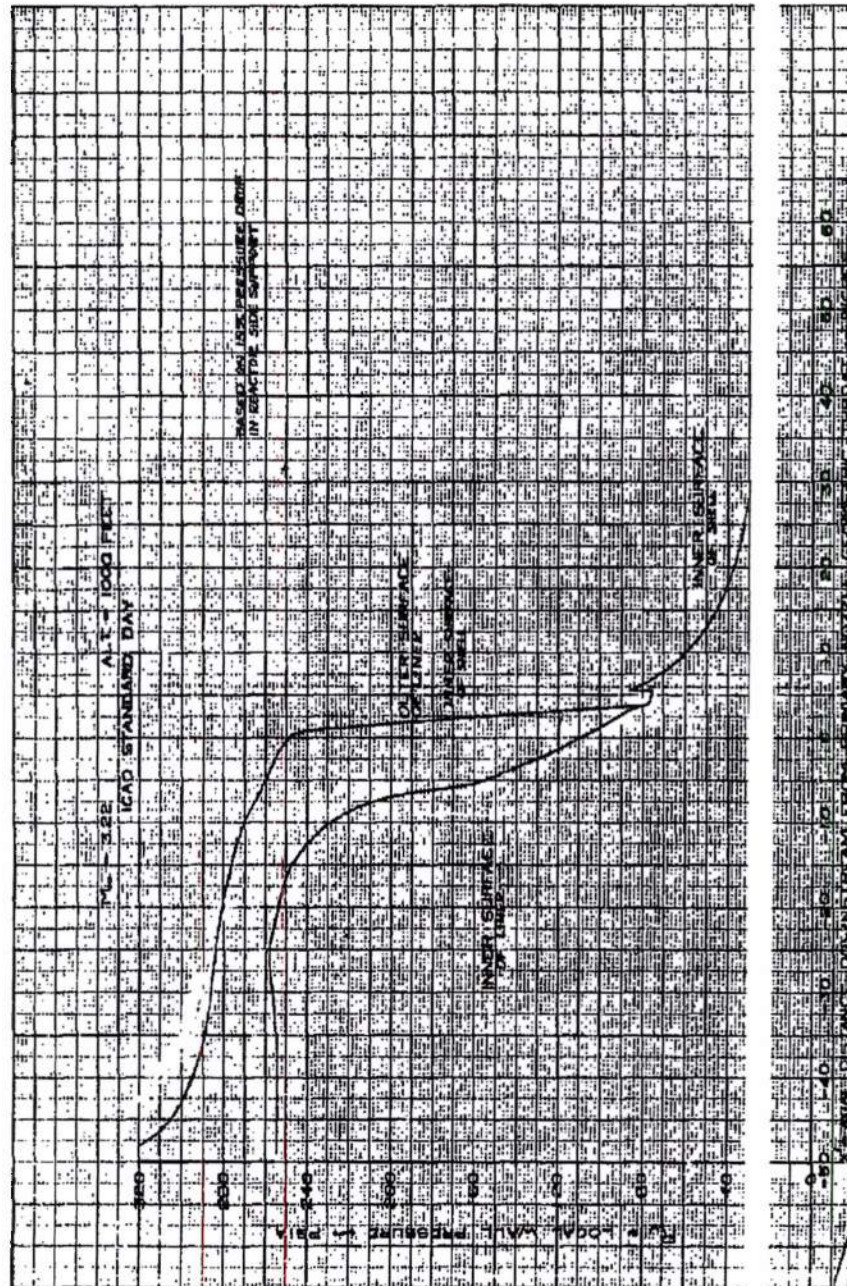


FIGURE 108. Wall Pressure Distribution for MA50-KCB Ejector Nozzle (Altitude = 10,000 Feet)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E774

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

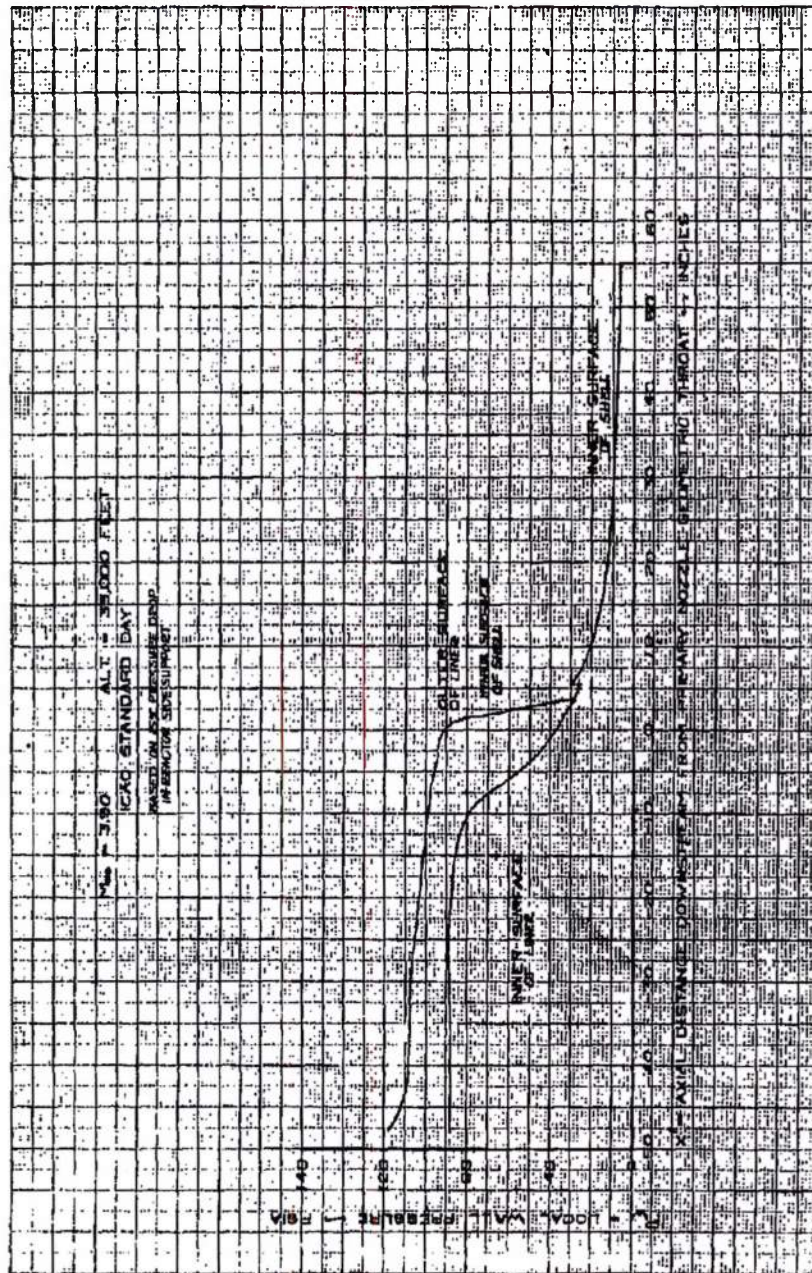


FIGURE 109. Wall Pressure Distribution for MA50-XCB Ejector Nozzle (Altitude = 35,000 Feet)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E772

- 26 -

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

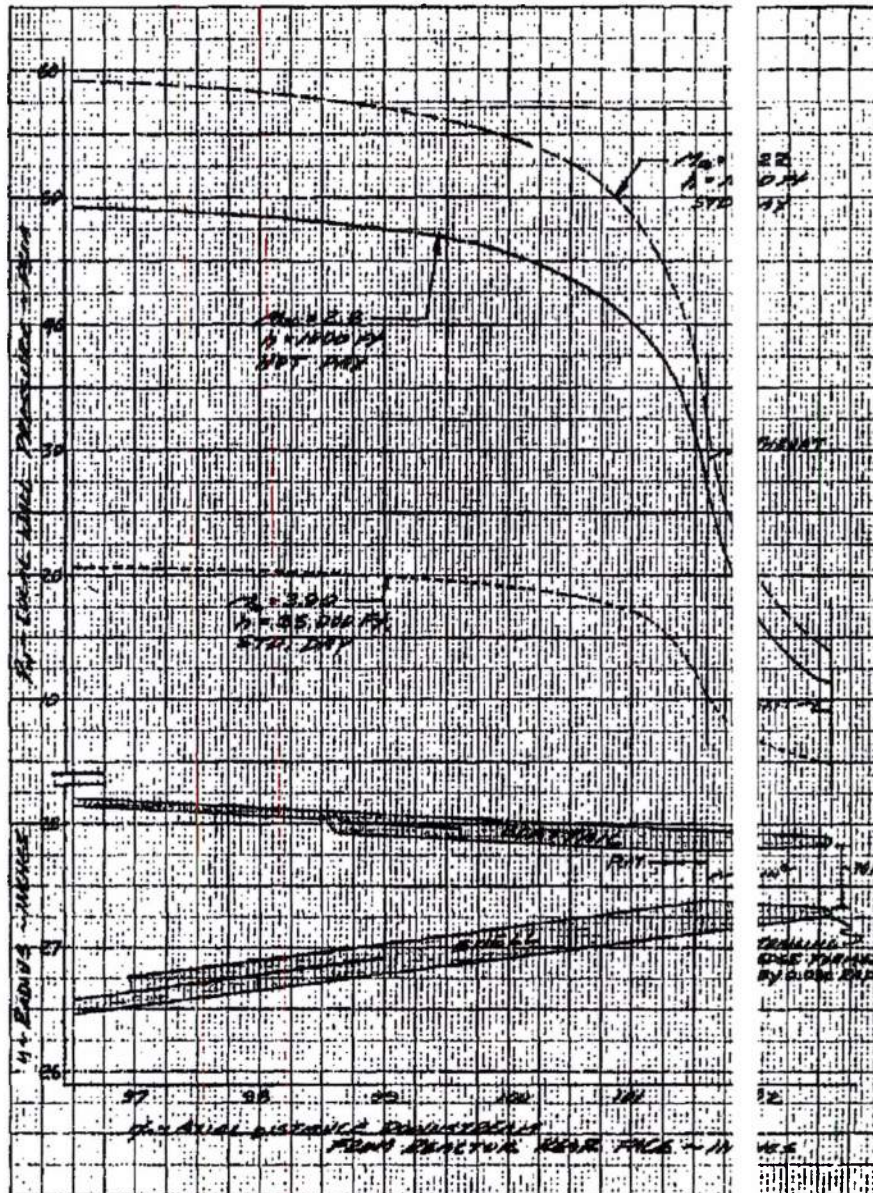


FIGURE 110. Local Wall Pressure Distribution in Exhaust I
 Airframe Cooling Flow: MA50-XCB Ejector Nozzle

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E770

- 265 -

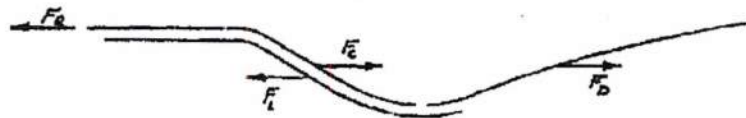
DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

RD 6003

MA50-XCB EJECTOR NOZZLE
 AXIAL LOADS (DRAG)



F_L = AXIAL LOAD ON NOZZLE LINER (LB)
 F_C = AXIAL LOAD ON SHELL CONVERGENT SECTION (LB)
 F_D = AXIAL LOAD ON SHELL DIVERGENT SECTION (LB)
 F_R = AXIAL FORCE REQUIRED TO RETAIN NOZZLE ASSEMBLY IN
 POSITION ON SIDE SUPPORT SHELL (LB)
 M_∞ = FLIGHT MACH NUMBER
 h = ALTITUDE (FT)

M_∞	h	DAY	F_L	F_C	F_D	F_R
3.22	1000	STD.	59,961	393,749	33,650	367,431
3.90	35,000	STD.	30,931	144,988	13,712	127,761

FIGURE 111. MA50-XCB Ejector Nozzle Axial Loads (Drag)

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
 Authority: EO 13526
 Chief, Records & Declass Div, WHS
 Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ASD-TDR-63-277, Vol. IV~~

REPORT 6003

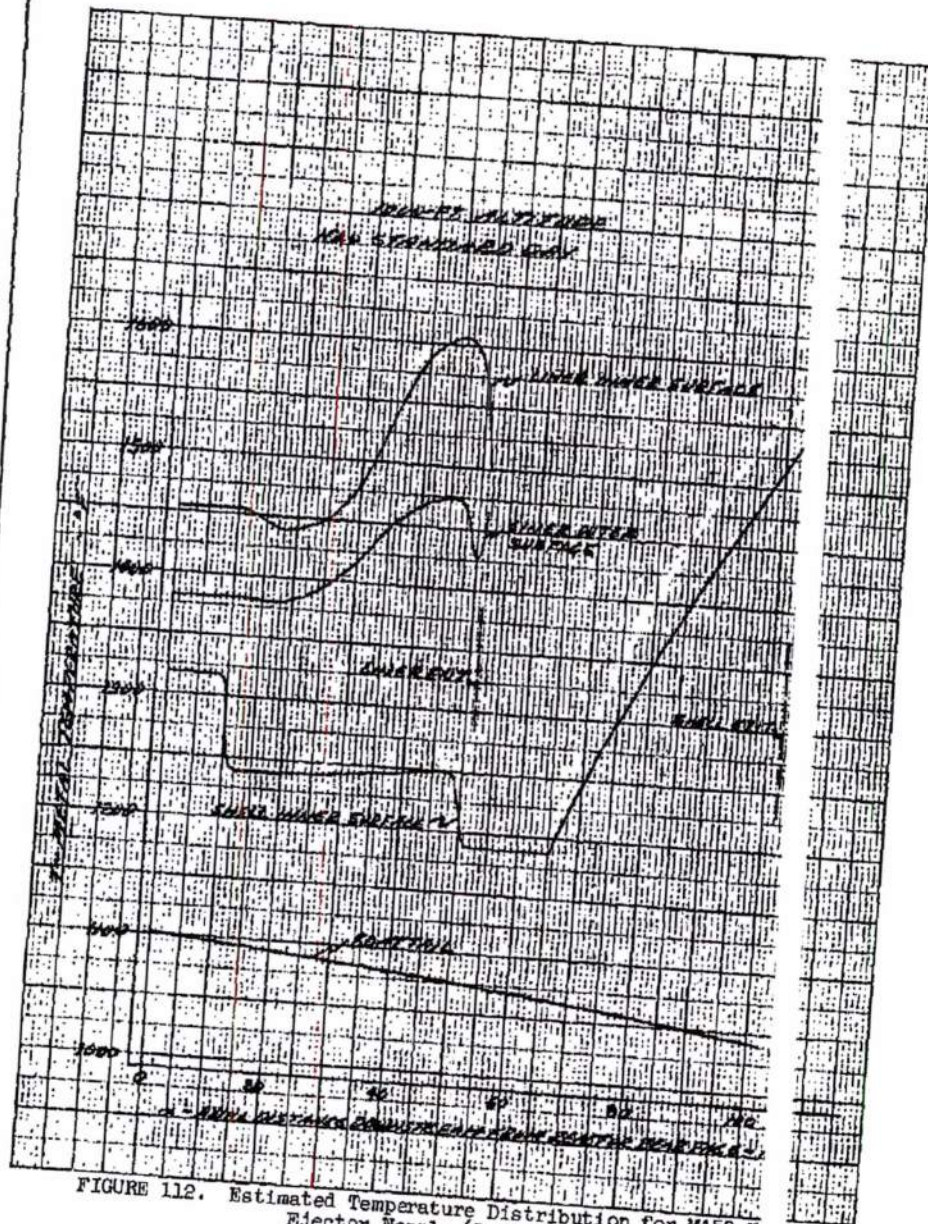


FIGURE 112. Estimated Temperature Distribution for MA50-X Ejector Nozzle (3.22 Mop)

~~SECRET RESTRICTED DATA~~
~~ASD-TDR-63-277, Vol. IV~~

22E776

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

6003

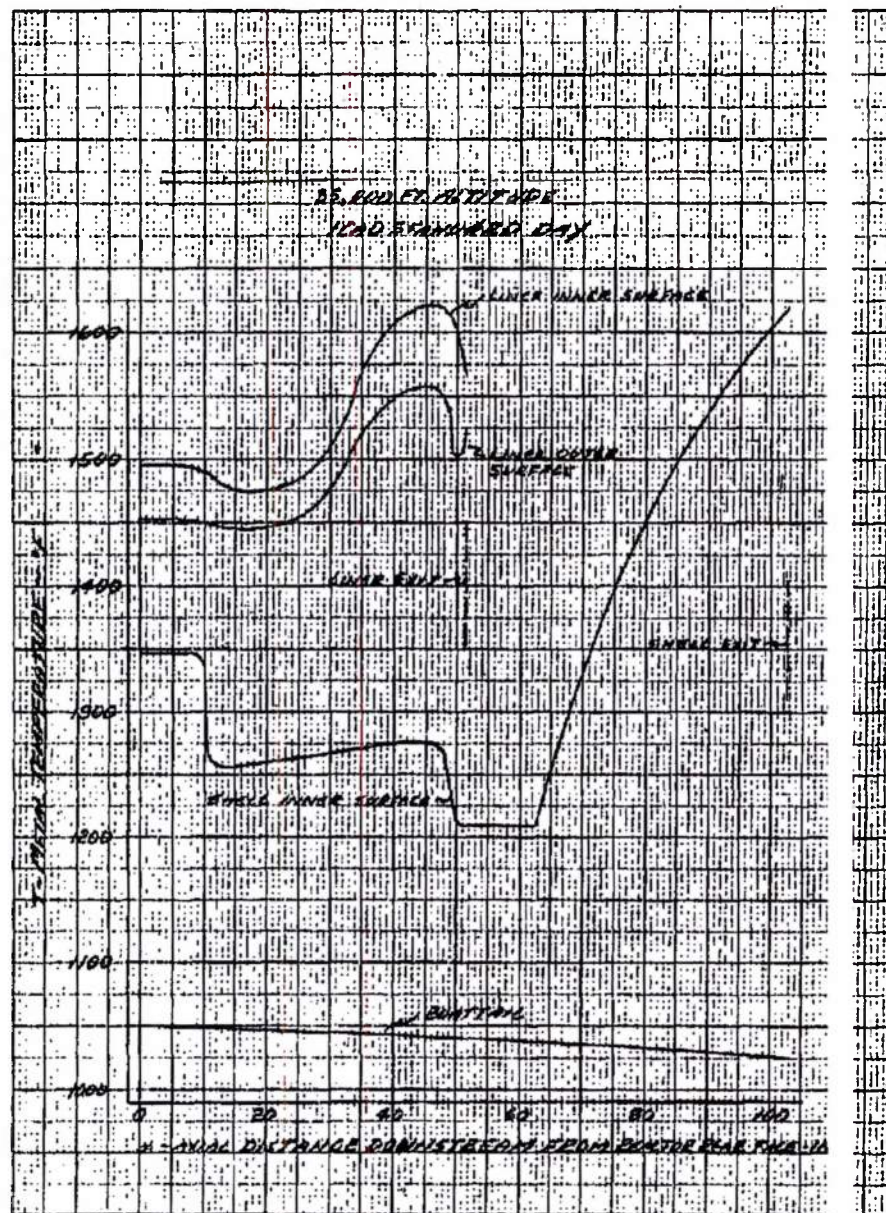


FIGURE 113. Estimated Temperature Distribution for MA50-3 Ejector Nozzle (3.90 M₀)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 403

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

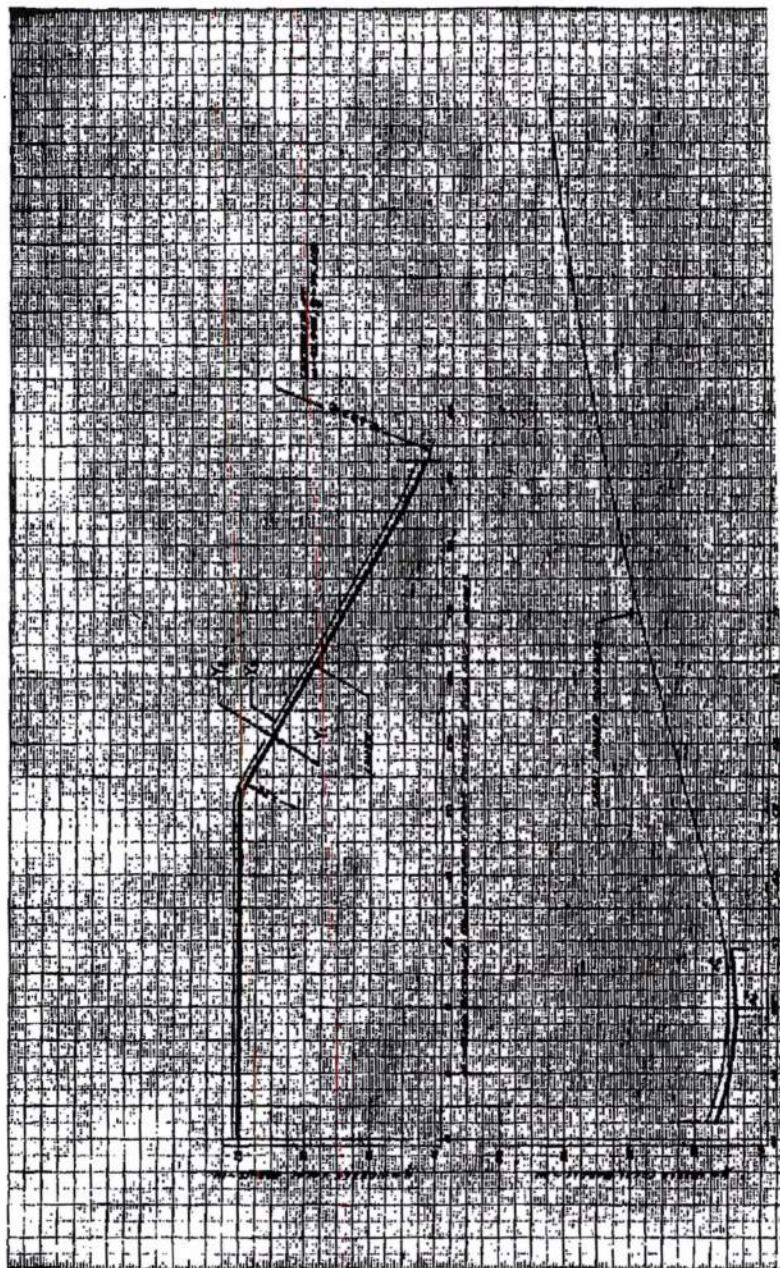


FIGURE 114. NA50-XCB Ejector Nozzle Contours (Based on 0.125 in. Nozzle Liner Thickness)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

22E773

- 269 -

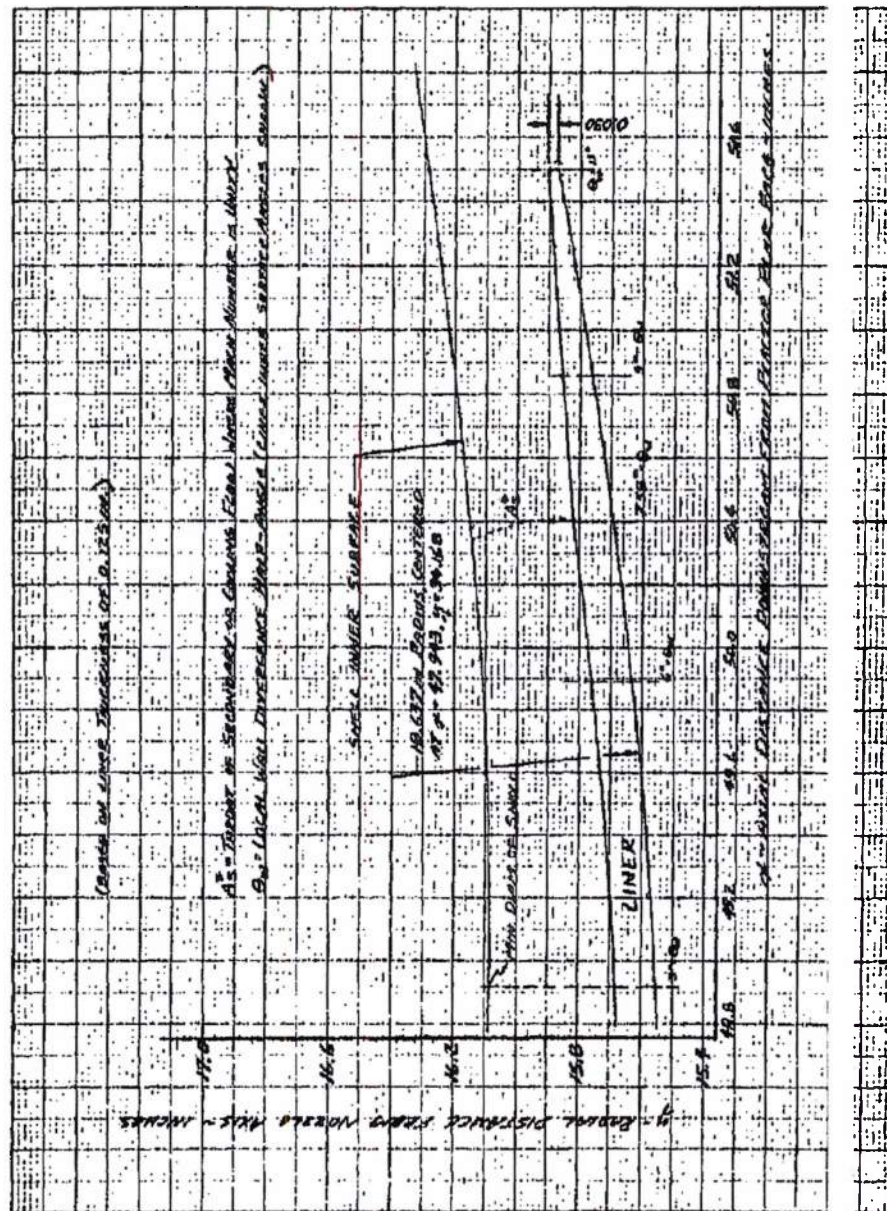


FIGURE 115. MA50-XCB Ejector Nozzle Liner Exit Details

MAC 100

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~CONFIDENTIAL~~

REPORT 6003

ASD-TDR-63-277, Vol. IV

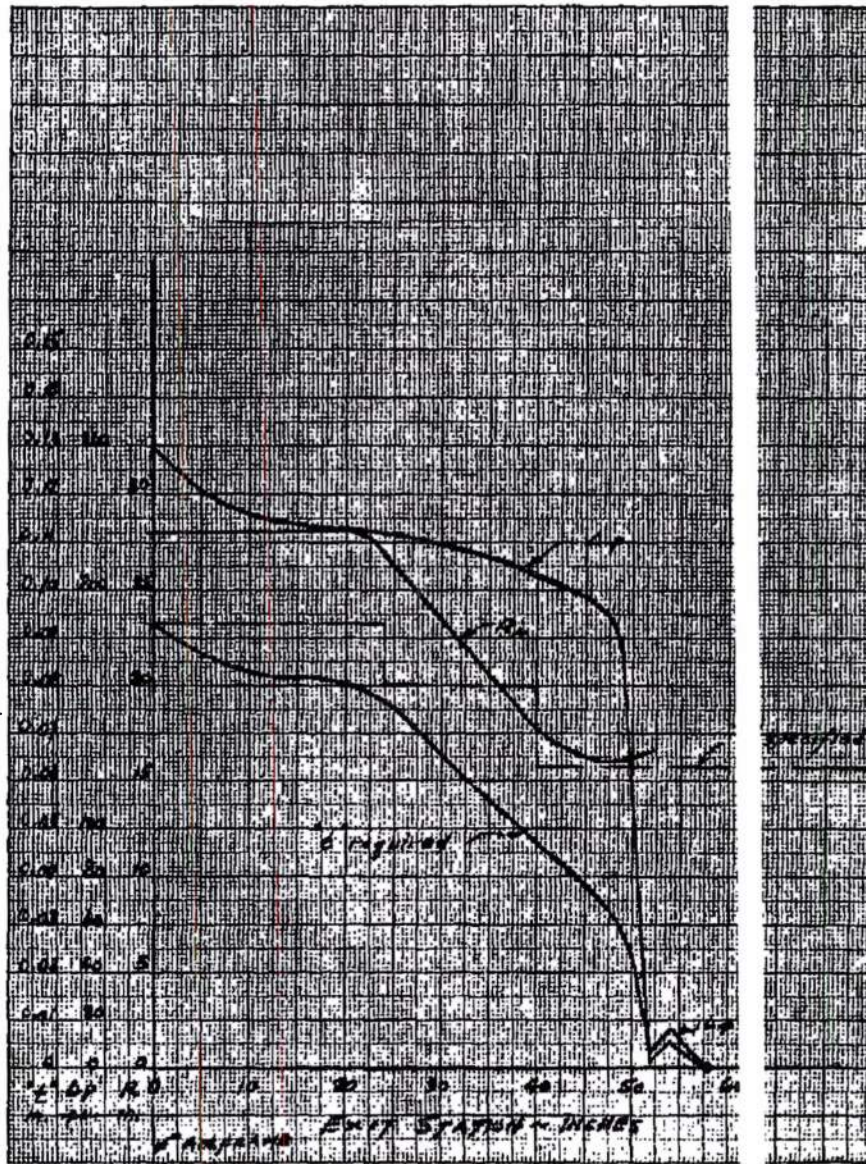


FIGURE 116. Nozzle Load Parameters

~~CONFIDENTIAL~~

MAC A 63

22E778

- 271 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

Page determined to be Unclassified
Reviewed Chief, RDD, WWS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

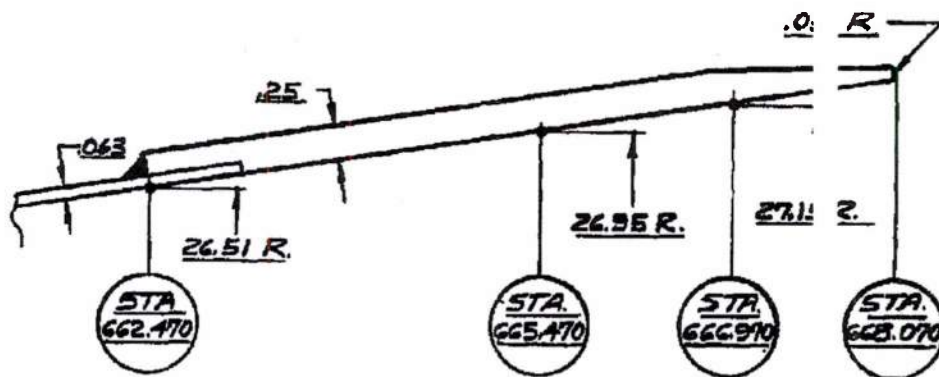


FIGURE 117. Doubler Ring

UNCLASSIFIED

22J84

- 272 -

CONFIDENTIAL

ASD-TDR-63-277, Vol. IV

REPORT 6003

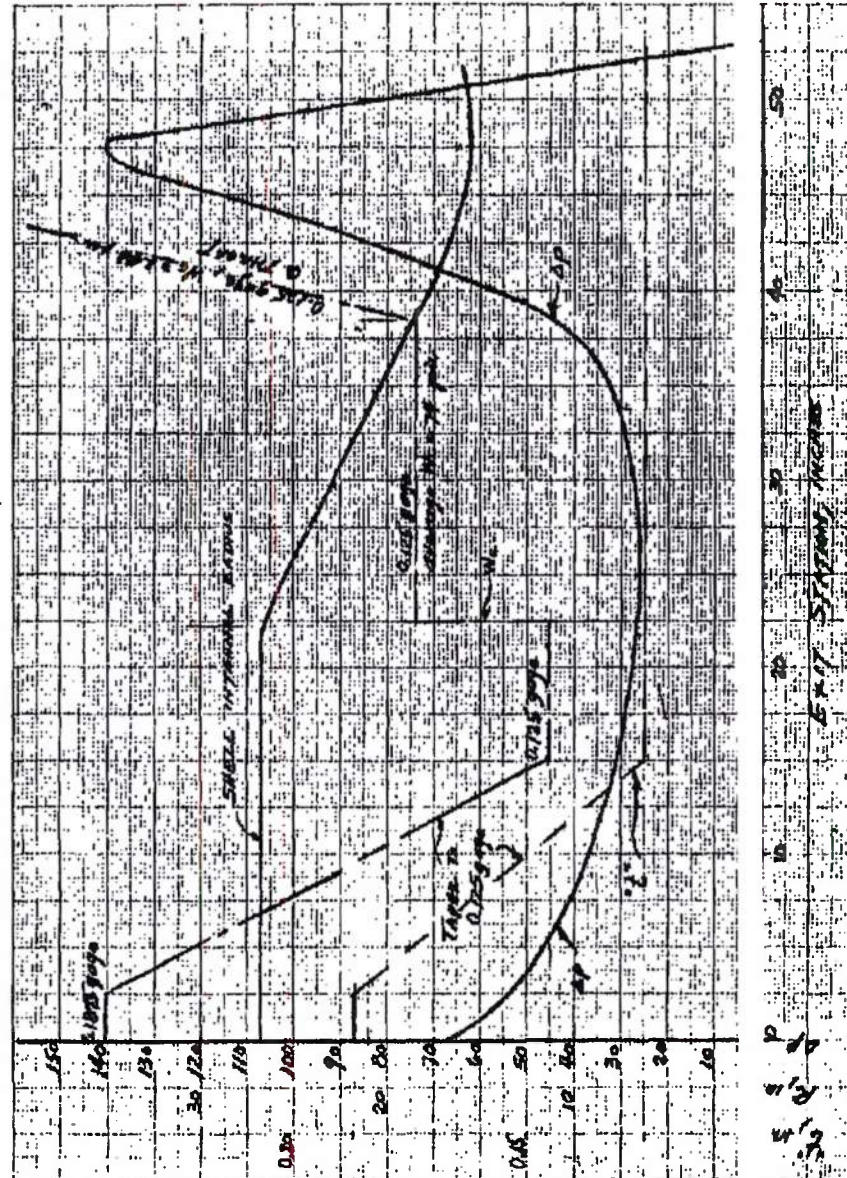


FIGURE 118. Liner Load Parameters

CONFIDENTIAL

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

ORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

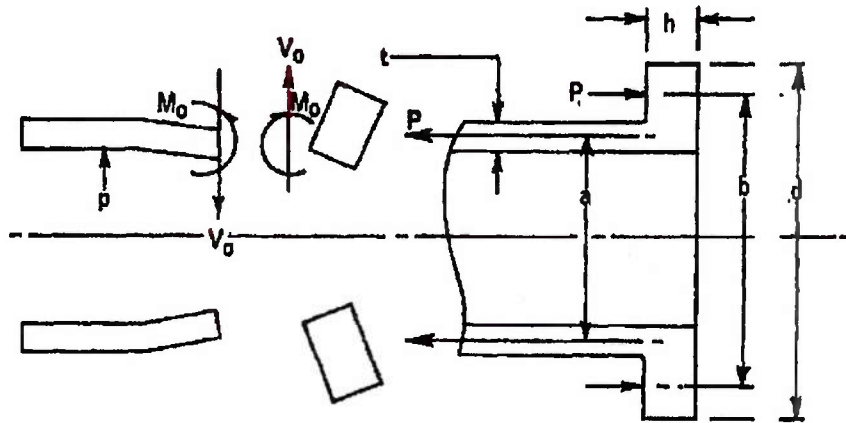


FIGURE 119. Assumed Coupling Geometry

UNCLASSIFIED

22J85

- 274 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

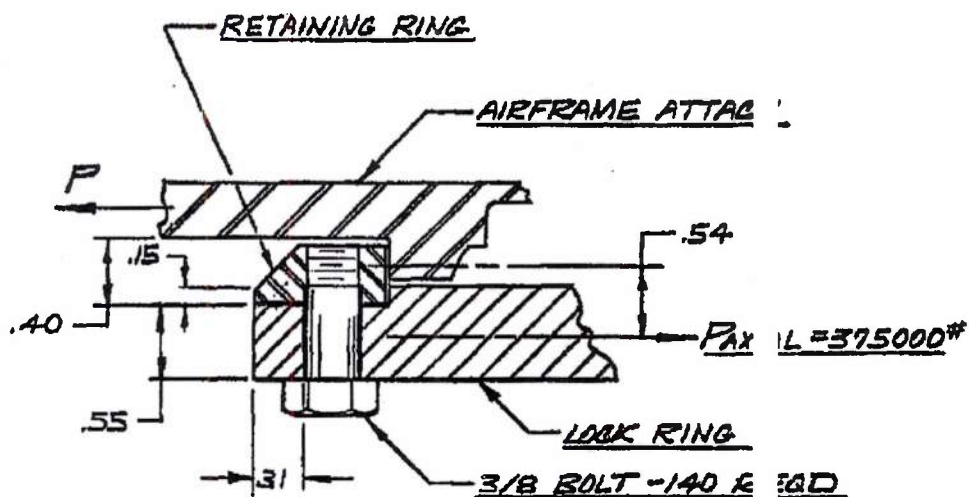


FIGURE 120. Retaining Ring Geometry

UNCLASSIFIED

22J86

- 275 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

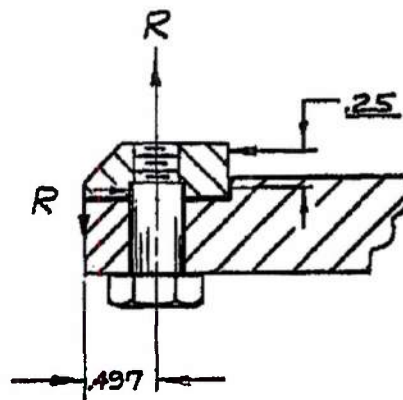


FIGURE 121. Airframe Fitting Shell

UNCLASSIFIED

22J87

- 276 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

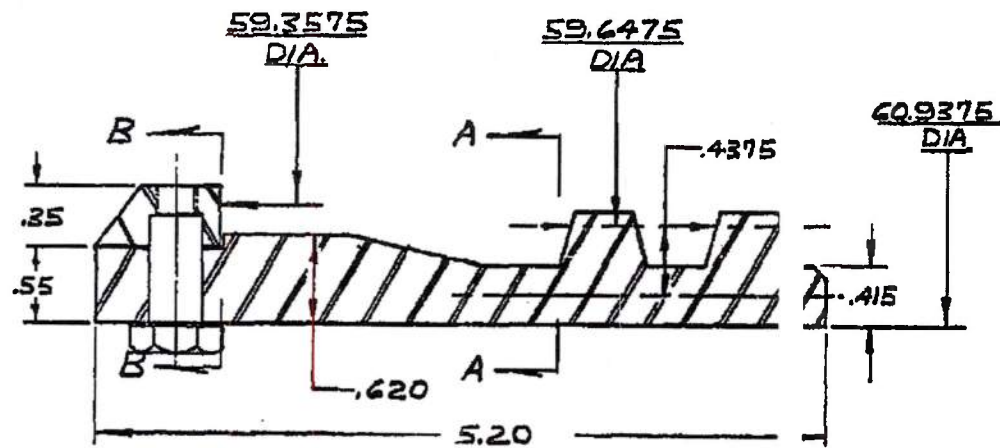


FIGURE 122. Lock Ring

UNCLASSIFIED

22J88

- 277 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

NI 6003

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

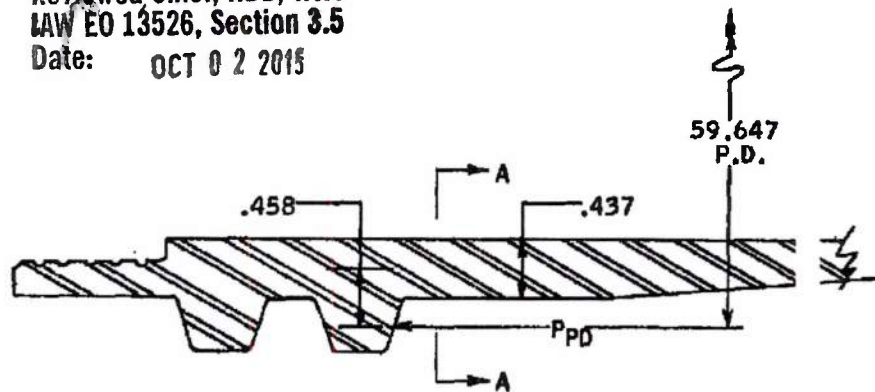


FIGURE 123. Exit Nozzle Attach Ring

MAC A430

UNCLASSIFIED

22J89

- 278 -

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

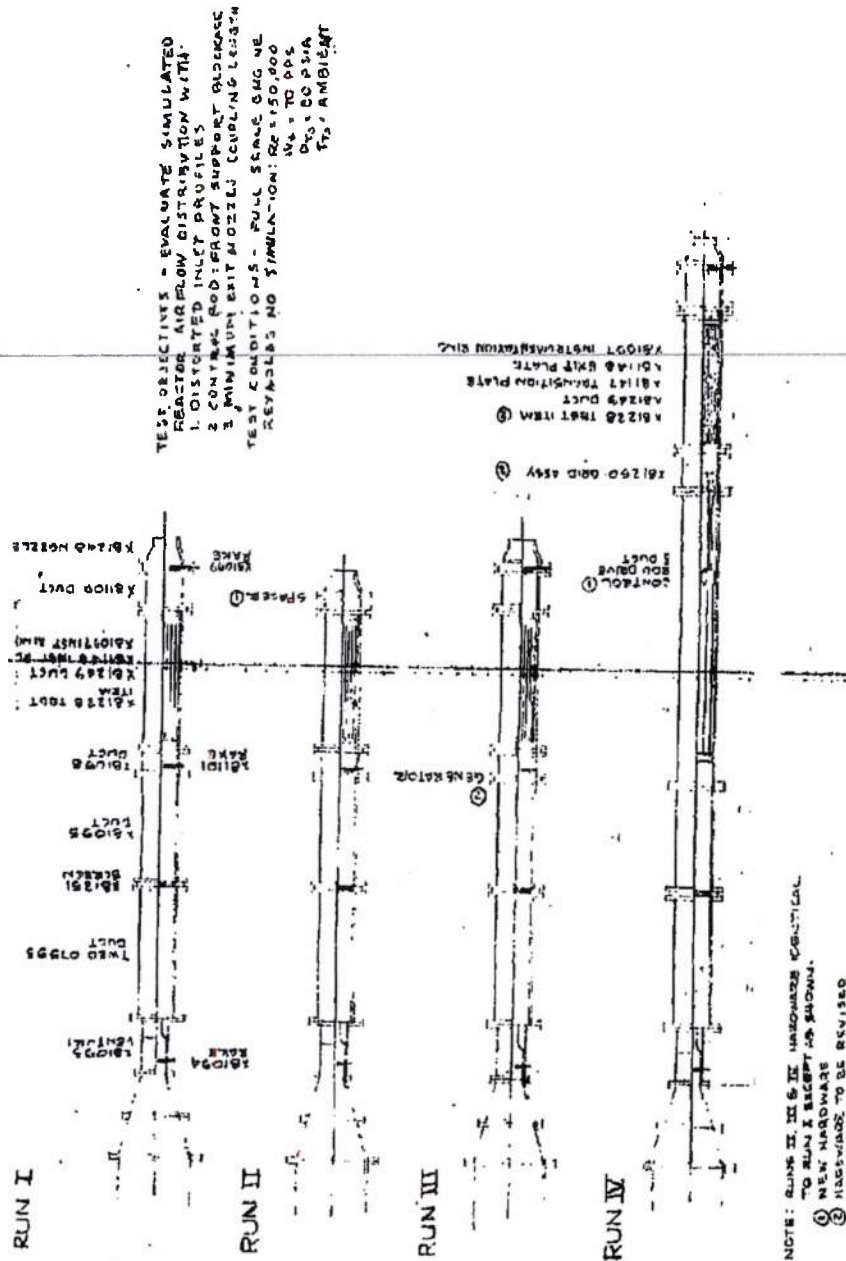


FIGURE 124. Testing of Aerodynamic Coupling with $1/3$ Scale Engine
(Phase I - Direct Connect Test)

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

POST 6003

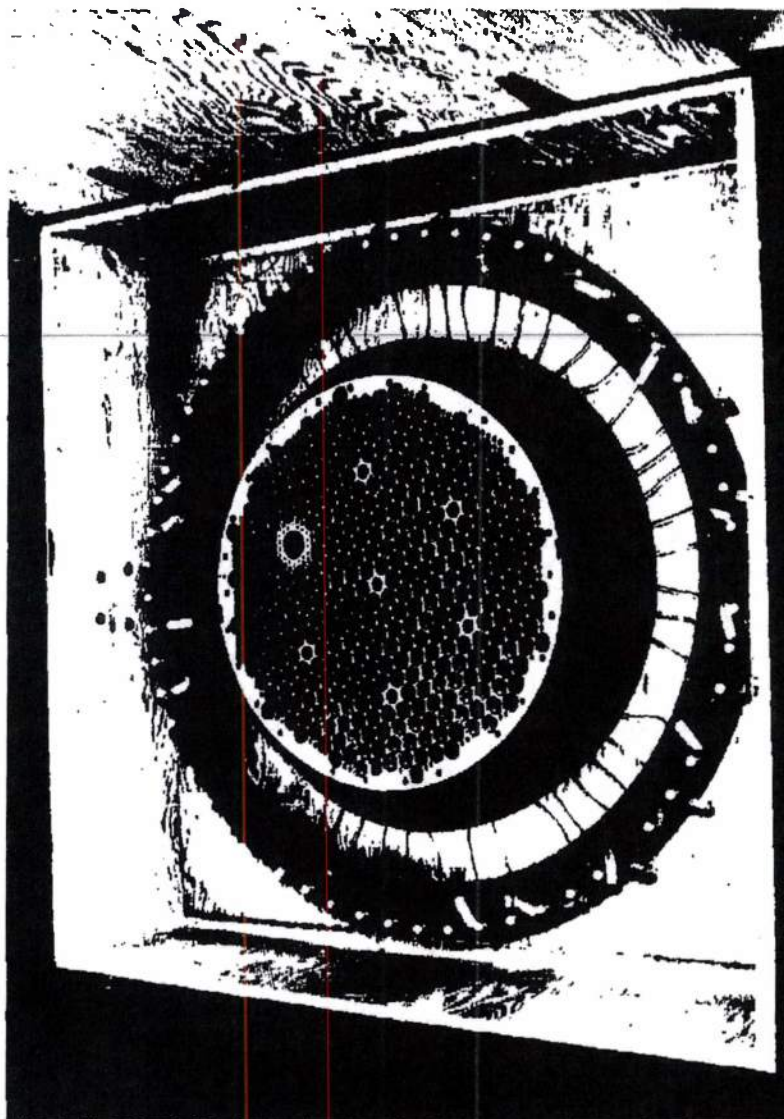


FIGURE 125. Aft Transition Section for Aerodynamic Coupling Test

NEB. C 4267-3

MAC 1422

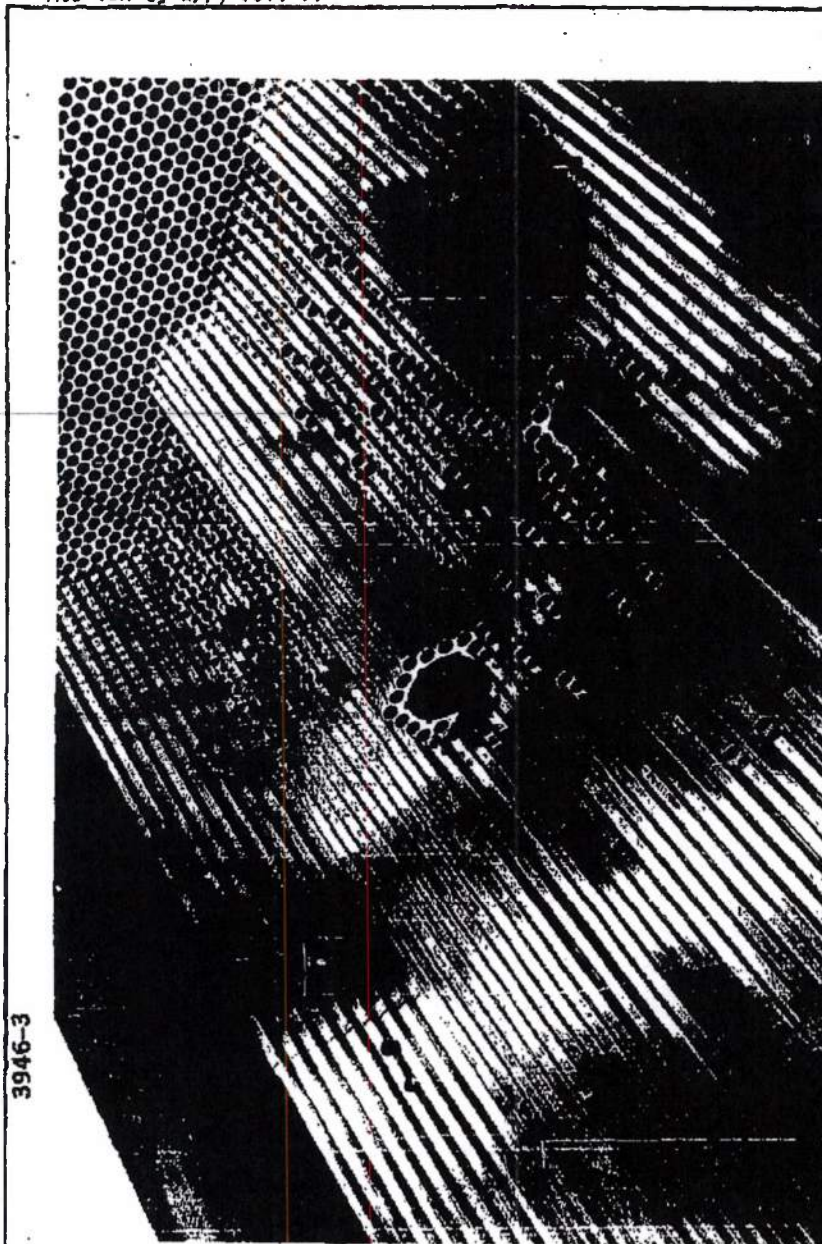
~~CONFIDENTIAL RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003



3946-3

FIGURE 126. Buildup of Simulated Reactor Segment for Aerodynamic Coupling Test (Closeup)

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 657

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~CONFIDENTIAL RESTRICTED DATA~~
~~ASD-TDR-63-277, Vol. 24~~

ASD-TDR-63-277, Vol. 24

REP 6003



FIGURE 127. Front Support Section of Aerodynamic Coupling (Looking Forward)

NEO, C 4007-5

MAC 407

~~CONFIDENTIAL RESTRICTED DATA~~
~~ASD-TDR-63-277, Vol. 24~~

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. 1V

REPORT 6003

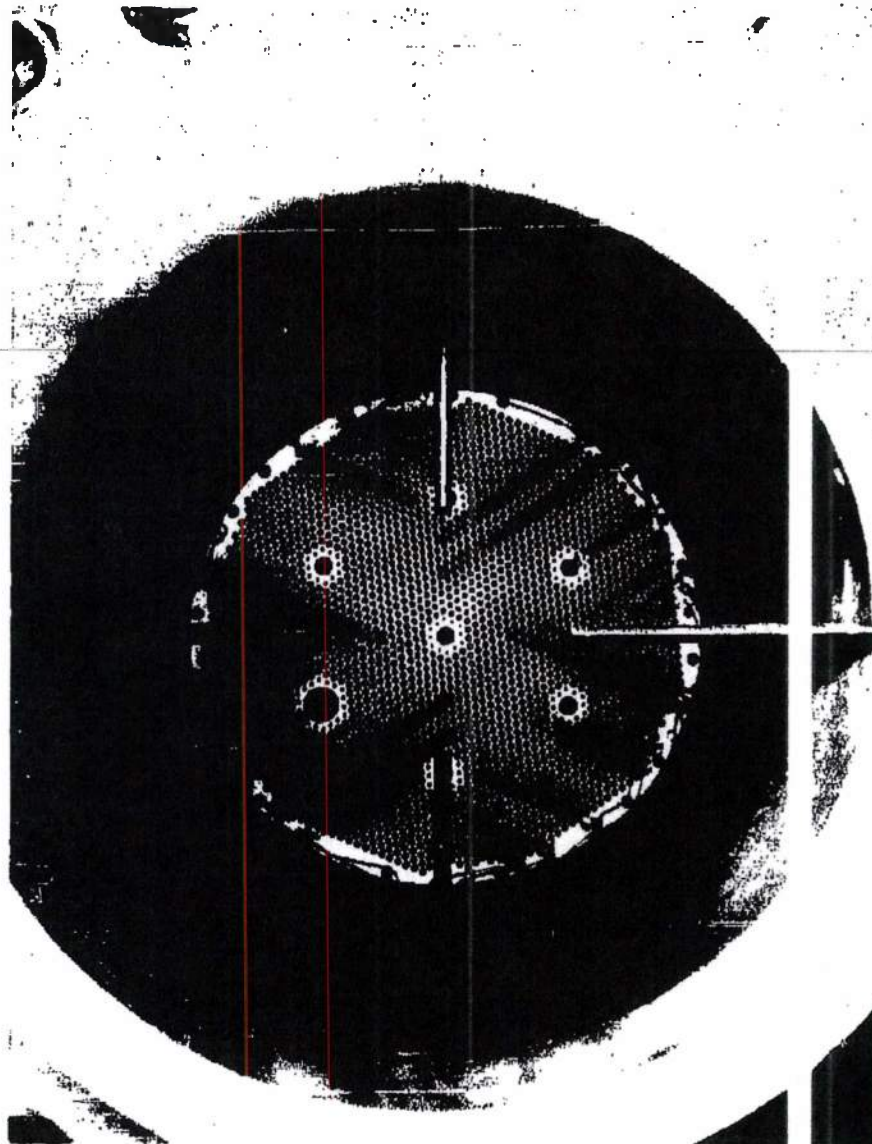


FIGURE 126. Aft End of Simulated Reactor Segment (Viewed Up Exit Nozzle)

~~CONFIDENTIAL RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

1

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

SECRET, UNCLASS, 1982

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

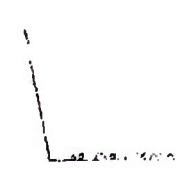
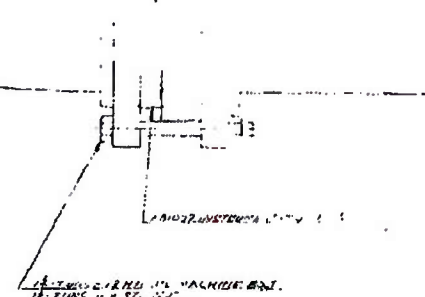
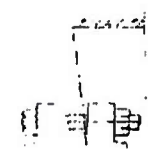
11

10

9

~~SECRET~~

3

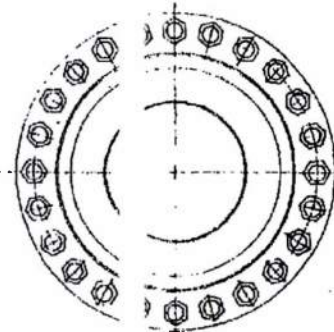
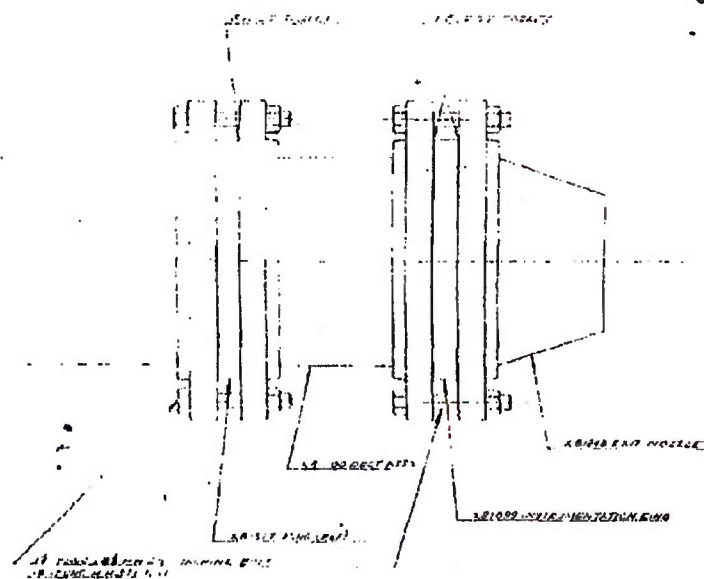


Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

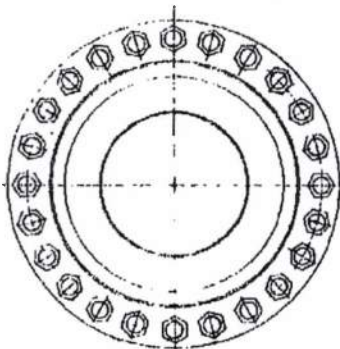
4

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date:

OCT 02 2015



5



DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

FIGURE 129: Engine Assembly
Aerodynamic Coupling, Free Jet



UNCLASSIFIED

ASD-TDR-63-277, vol. 10

REPORT 6003

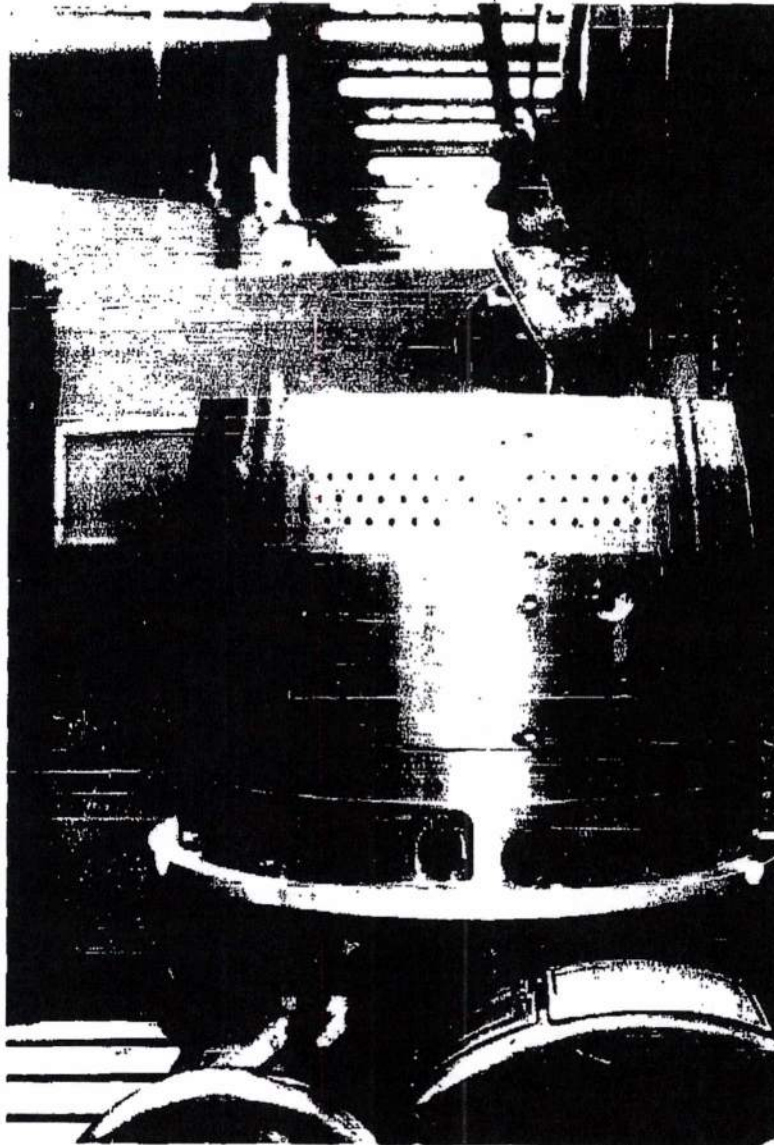


FIGURE 130. CCA Assembly for Free Jet Aerodynamic Coupling Test

UNCLASSIFIED

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

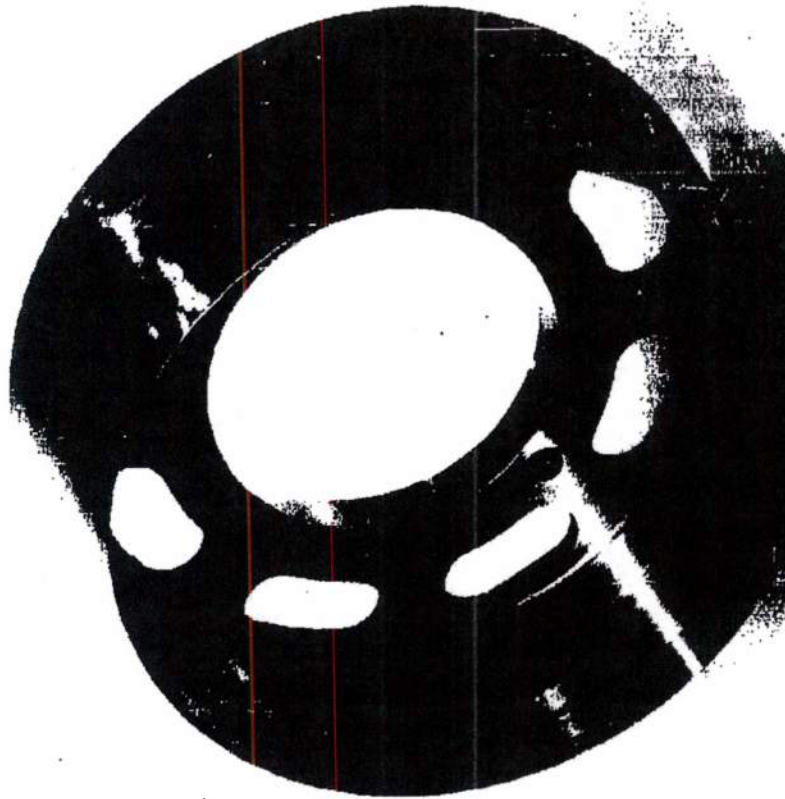


FIGURE 131. Spike Adapter for Free Jet Aerodynamic Coupling Test

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

NEG - 131-3

MAC ADP

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

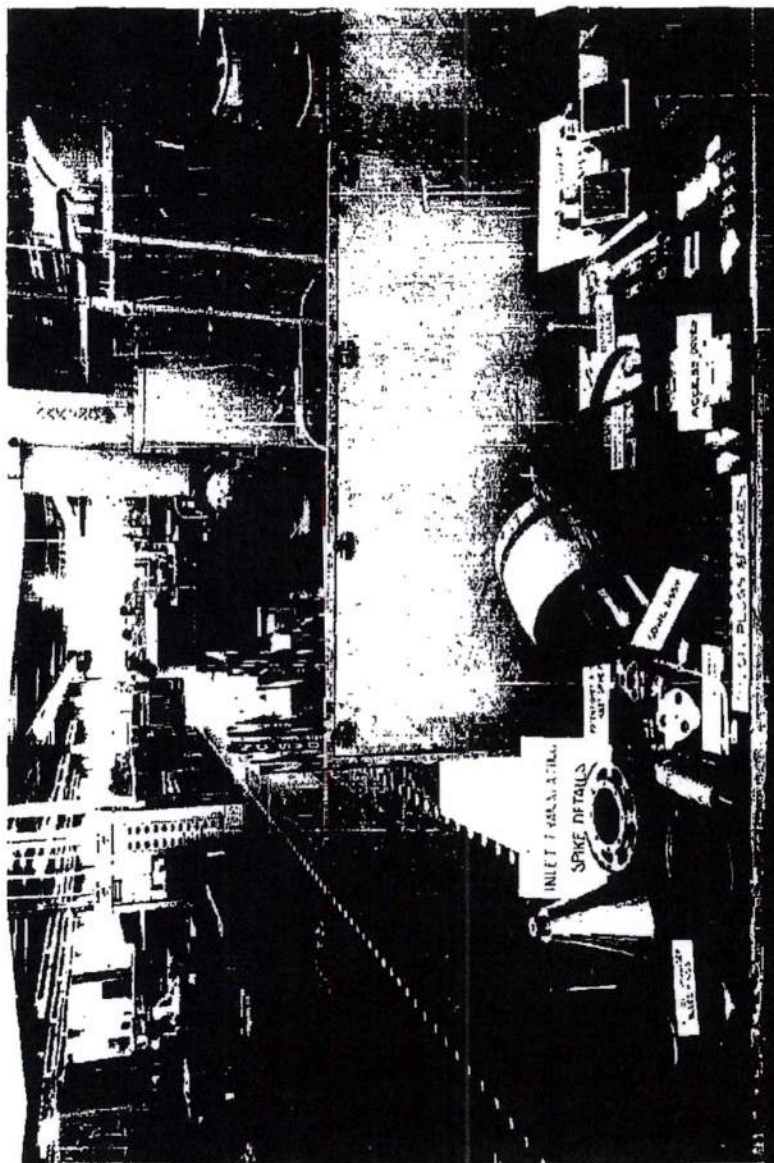


FIGURE 152. Display of Components for Free Jet Aerodynamic Coupling Test

UNCLASSIFIED

- 289 -

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

ASD-TOR-63-277, Vol. 14

6003

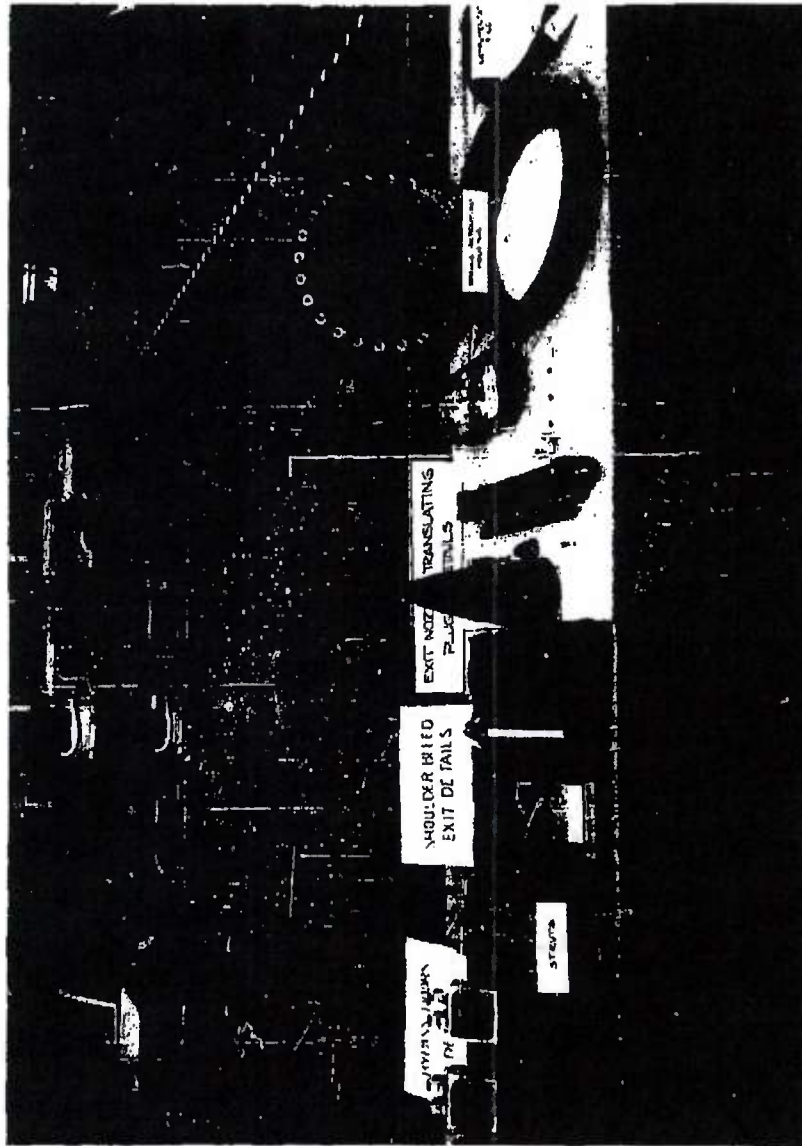


FIGURE 133. Closeup of Components for Free Jet Aerodynamic Coupling Test

UNCLASSIFIED

29

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

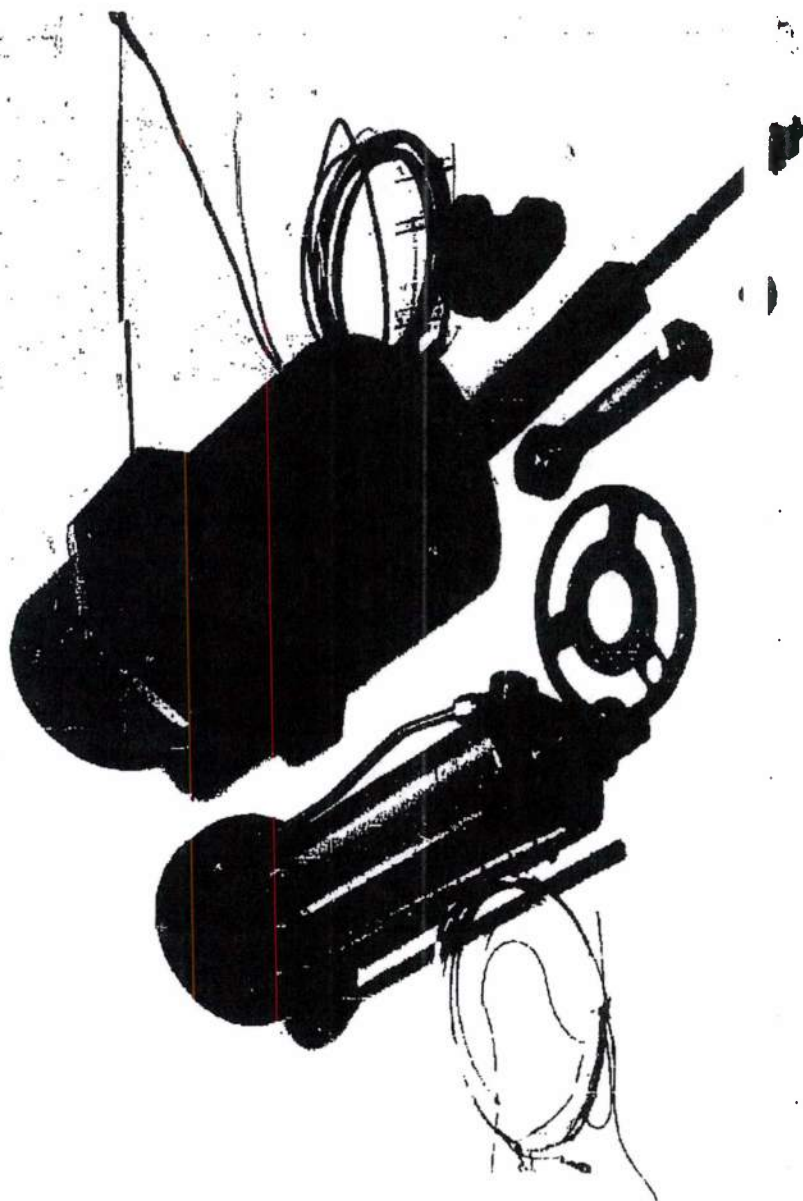


FIGURE 134. Spike Actuator and Housing Details for Free Jet Aerodynamic Coupling Test

UNCLASSIFIED

291 -

Page determined to be unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

ASD-TDR-63-277 Vol. 33

FORM 6003

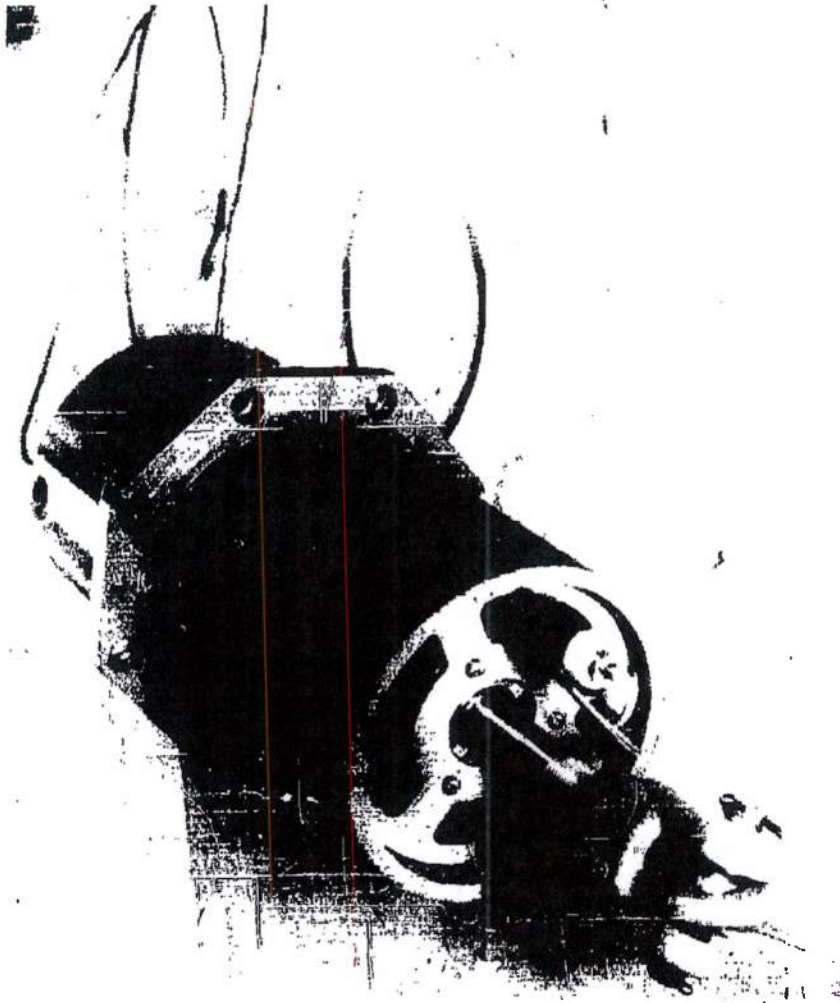


FIGURE 135. Spike Actuator and Housing Assembly for Free Jet Aerodynamic Coupling Test

NEG. 434A-18

MAC A-53

UNCLASSIFIED

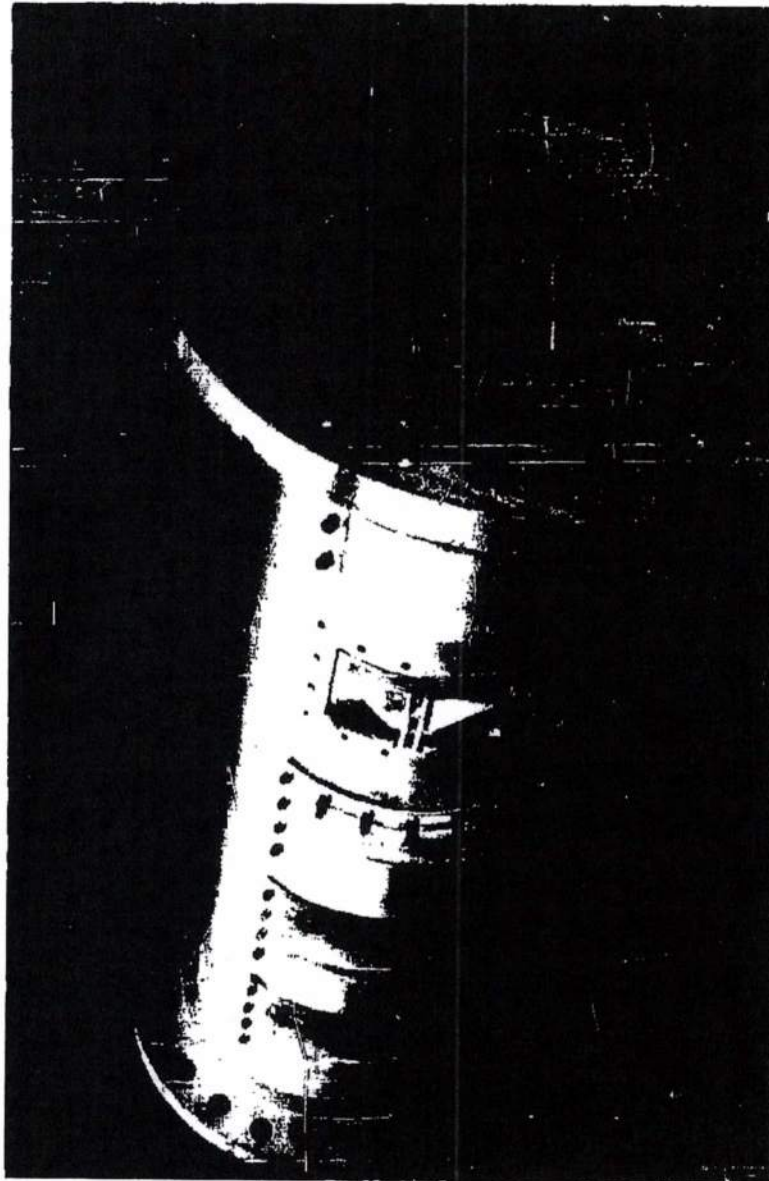
232 -

Page determined to be unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003



NEG. 4344-19

FIGURE 136. Simulated Reactor and Housing for Free Jet Aerodynamic Coupling Test

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

MAC 453

~~SECRET RESTRICTED DATA~~
~~ARMED FOR EXPORT ACT OF 1954~~

ASD-TDR-63-277 Vol. 17

PORT 6003



FIGURE 137. Test Item Assembly (Axial Load Spring Retention) for Free Jet Aerodynamic Coupling Test

MEG. 4344-21

MAC A673

~~SECRET RESTRICTED DATA~~
~~ARMED FOR EXPORT ACT OF 1954~~

~~CONFIDENTIAL~~

ASC-TR-63-277 Vol. 1

REPORT 6003



FIGURE 138. Centerbody and Inlet Duct for Free Jet Aerodynamic Coupling Test

NEG. 4344-24

MAC 1133

~~CONFIDENTIAL~~

235

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~CONFIDENTIAL~~

ASD-TDR-63-277 Vol. 13

REPORT 6003

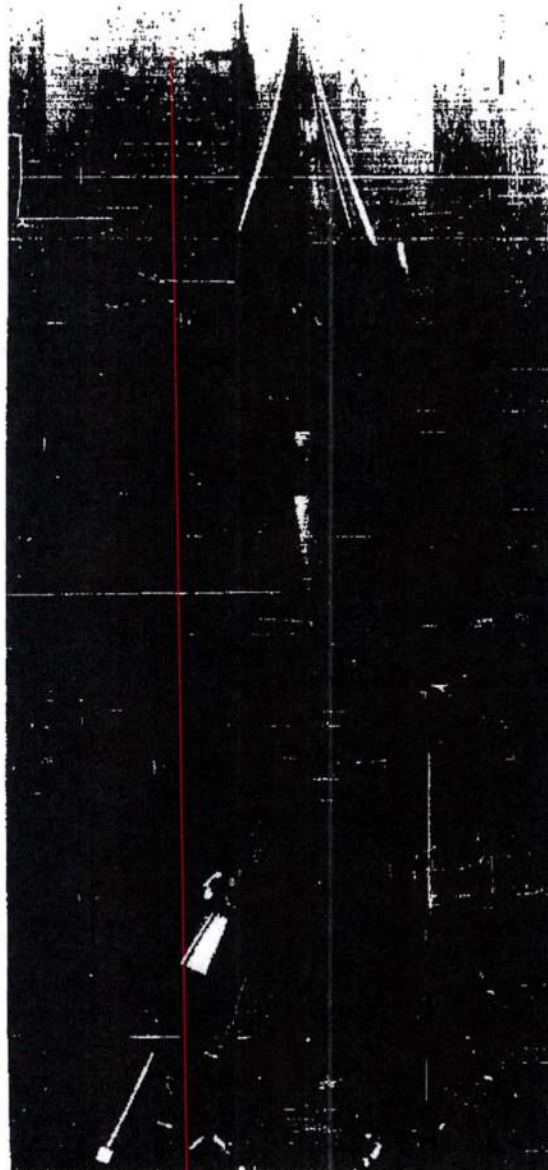


FIGURE 139. Centerbody and Inlet Duct Assembly for Free Jet Aerodynamic Coupling Test

NEG. 4344-26

MAC 8573

~~CONFIDENTIAL~~

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

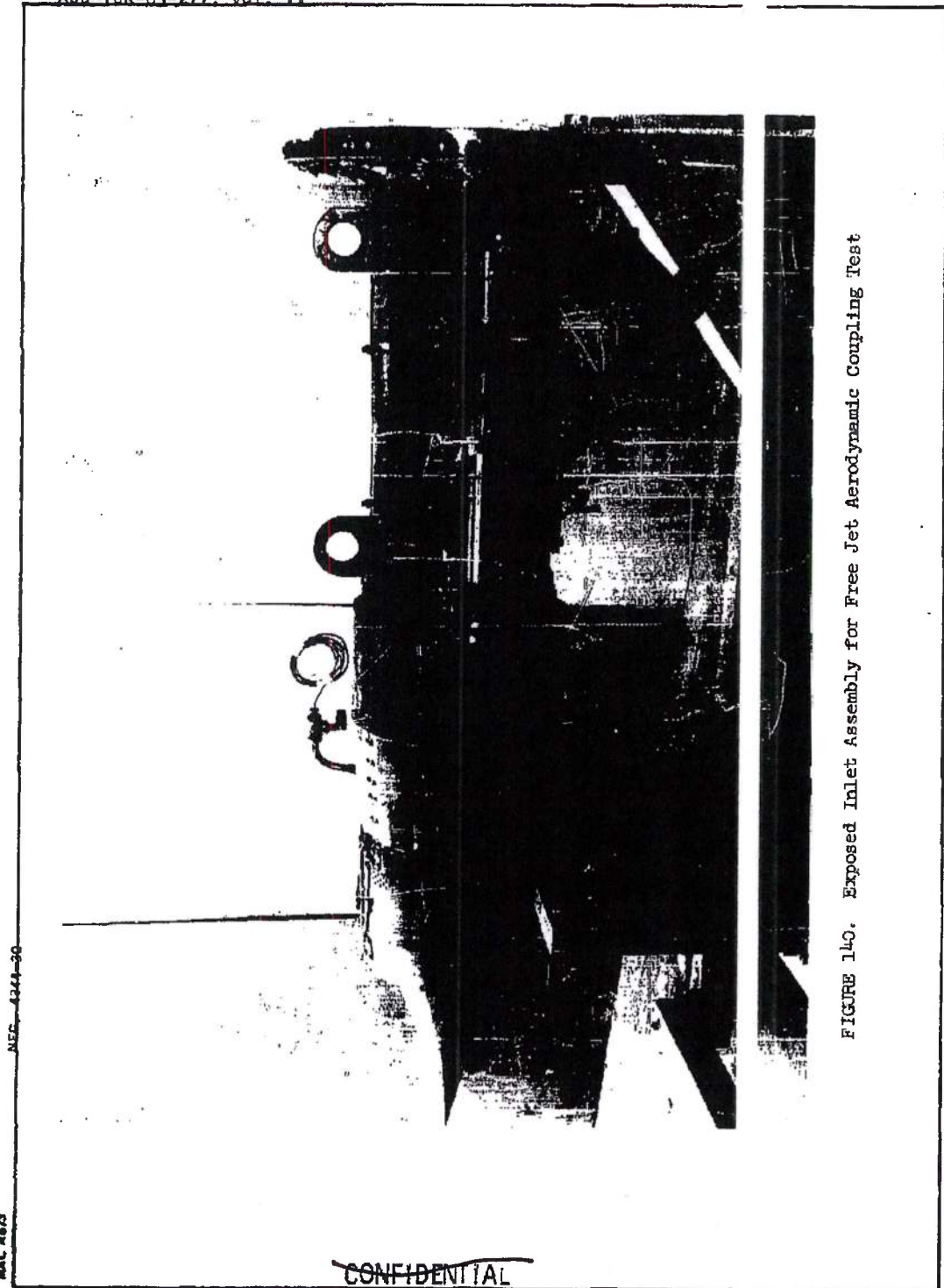


FIGURE 140. Exposed Inlet Assembly for Free Jet Aerodynamic Coupling Test

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~CONFIDENTIAL~~

ASD-TDR-63-277, Vol. 34

6003

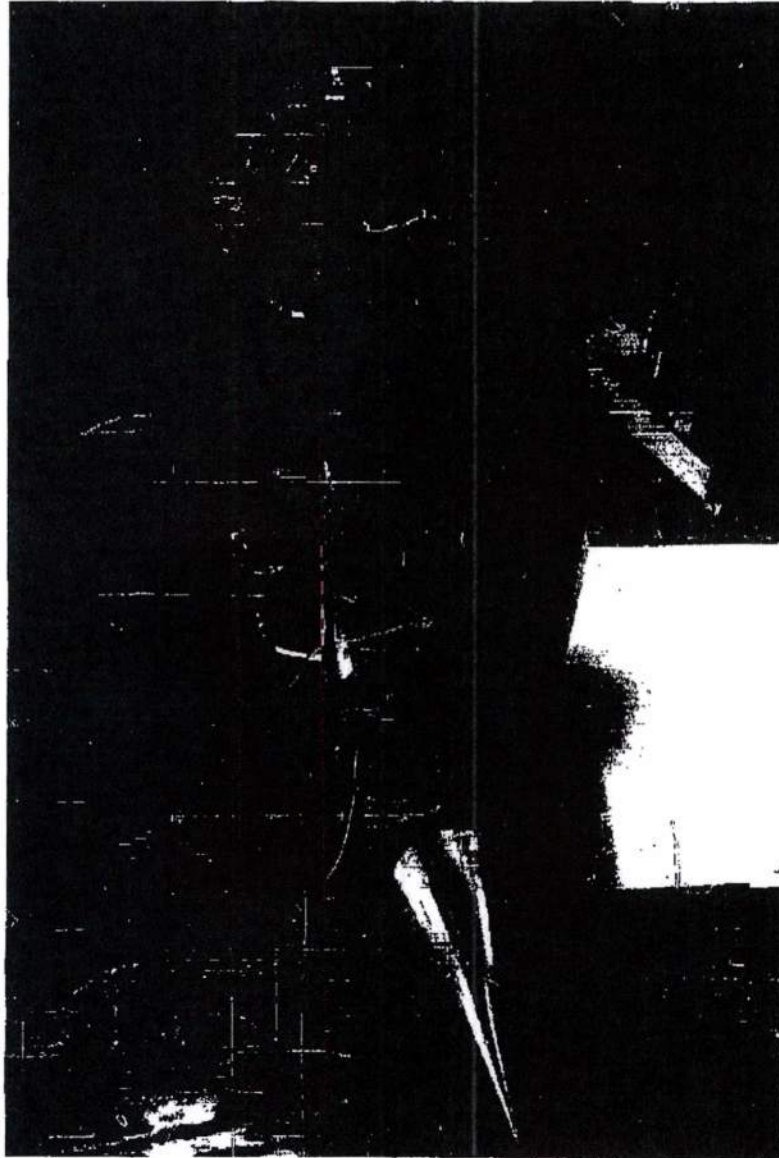


FIGURE 141. Inlet Assembly for Free Jet Aerodynamic Coupling Test
(Three Quarter View)

NEG. 4344-31

MAC 6672

~~CONFIDENTIAL~~

293

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

NEG. T-3066-1

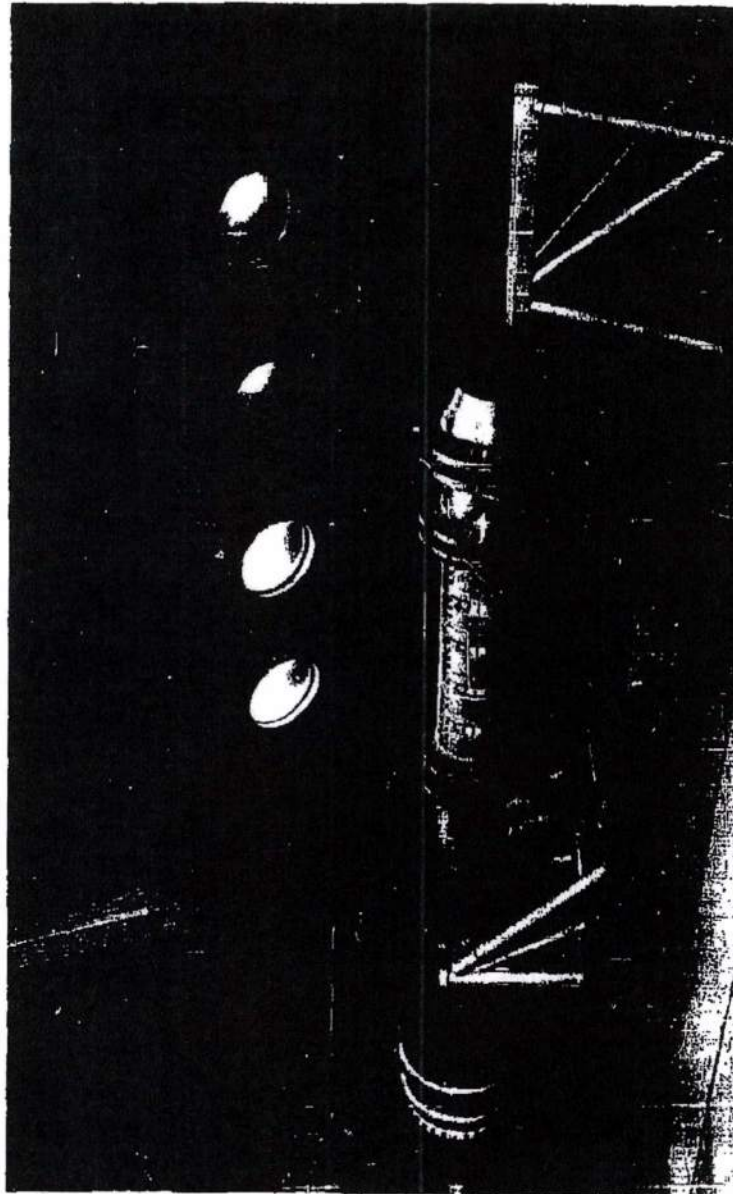


FIGURE 142. Setup of Plute Propulsion System Model for Free Jet Aerodynamic Coupling Test

MAC 403

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

APPENDIX A

~~CLASSIFIED MATERIALS STUDIES~~

Cyclic Stress Studies on Rene' 41

To evaluate the behavior of Rene' 41 under conditions of operation, creep specimens of 0.050 in. thick sheet were subjected to tensile stresses at 1400°F in accordance with the stress-time profile in Figure A-1. The results of deformations produced by cyclic loading in comparison with deformations produced on control specimens subjected to a standard creep test at the same temperature, subjected to a constant stress of 52 Ksi and for the same total time are presented in Table A-I. A comparison of results between test and control specimens shows that cycled specimens elongated less by a factor 1/4 to 1/9, indicating strengthening occurred as a result of this particular cyclic test. Standard tensile tests were also performed at a strain rate of 0.001 in./in./sec at 1400°F, the principle strain rate for cyclic testing. The results of these tensile tests are compared in Table A-II. Cycled specimens show tensile values intermediate to noncycled and creep tested specimens; the creep tested specimens having the highest tensile values.

Table A-III and Figure A-2 present results of creep rupture tests performed on control specimens at 1400°F under a 52 Ksi stress. The time to rupture in 10 hours was approximately 0.2% much higher than the cycled specimens. Tensile tests were also performed at 1400°F under stress rates which produced strain rates of 0.001, 0.01, and 0.1 in./in./sec. The results of these tests are presented in Table A-IV and Figures A-3 and A-4. As expected, the short time tensile properties increased with the strain rate.

The strengthening effect shown by these cyclic tests may be due to a combination of additional precipitation and/or solid solution strengthening stimulated by repeated strains, at the temperature of testing. Considerable additional testing will be needed to resolve the effects to be expected from service cycles. A program of cyclic testing of Rene' 41 and the other alloys should be undertaken to study the effects of temperature and stress cycling on precipitation and nonprecipitation hardening alloys.

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

TABLE A-I

CREEP OF RENE' 41 DURING CYCLIC TESTING

Specimens - Standard, 0.050 in. thick sheet
Creep Measurements - Before and after cyclic testing.
Specimens at room temperature.
Scribe marks measured with
Guertner toolmaker's microscope
Heating - Resistance

Specimen Number	Length (Inches)		Change in Length In 2 Inches (in.)	Plastic Deformation In 2 Inch Gage Length (%)
	Before Cycling	After Cycling		
G-1	--	1.9978	--	--
G-3	1.9990	2.0008	0.0018	0.09
G-13	1.9988	1.9993	0.0005	0.03
G-14	1.9999	2.0012	0.0013	0.07

CREEP OF RENE' 41 DURING STATIC TESTING

Load, Temperature, - Load - 52,000 psi, constant
and Time - Same temperature as above
- time 5 hours

Specimen Number	Length (Inches)		Change in Length In 2 Inches (in.)	Plastic Deformation In 2 Inch Gage Length (%)
	Before Cycling	After Cycling		
G-10	1.9980	2.0068	0.0088	0.44
G-11	1.9990	2.0079	0.0086	0.43

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REF 6003

TABLE A-II
 TENSILE PROPERTIES OF RENE' 41 SHEET AFTER CYCLIC TESTING

Heat Treatment - 1950°F, 1/2 hr, rapid air cool,
 Age at 1400°F, 16 hrs
 Heating - Resistance
 Hold Time - 5 minutes
 Specimens - Standard, 0.050 inch thick sheet

Specimen Number	Strain Rate Start to Rupture (in/in/sec)	Test Temperature (°F)	Proportional Limit (ksi)	0.2% Yield Strength (ksi)	Ultimate Tensile Strength (ksi)	Elongation In 2 Inches (%)	Young's Modulus (x 10 ⁶ psi)
G-1	0.001	1400	97.0	125.0	(127.5)*	(0.9)*	24.0
G-3	0.001	1400	88.0	127.7	141.0	2.4	22.2
G-13	0.001	1400	86.0	126.0	147.0	3.7	23.8
G-14	0.001	1400	90.0	126.0	146.0	3.3	23.1
Average of 3 specimens not cyclic tested.		Average	90.2	126.2	144.7	3.1	23.3
	0.001	1400	77.0	119.7	134.0	2.5	25.3
TENSILE PROPERTIES OF RENE' 41 SHEET AFTER STATIC TESTING SIMULATING CYCLIC TESTING							
G-10	0.001	1400	79.0	133.0	153.8	4.0	24.0
G-11	0.001	1400	81.0	131.8	151.0	4.0	24.0

* Broke at thermocouple weld, not valid.

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

TABLE A-III

CREEP-RUPTURE OF RENE' 41 SHEET CONTROL SPECIMENS

Test Conditions:

- Rene' 41, 0.050 inch thick sheet
- Heat Treatment - 1950°F 30 minutes
- Age 1400°F, 16 hours
- Heating - Resistance
- Gage Length - 1.5 inches
- Atmosphere - Air
- Specimens - Standard

Specimen Number	Test Temperature (°F)	Creep Stress (ksi)	Time to Produce Indicated Plastic Creep (hours)						Time to Rupture (hours)	Elongation In 1.5 Inches (%)
			0.5%	0.1%	0.2%	0.5%	1.0%	2.0%		
G-18	1400	52	0.3	3.6	13.2	30.1	49.1	63.1	73.35	4.0
G-21	1400	52	2.8	8.6	23.3	48.1	64.5	81.0	87.5	2.3

DECLASSIFIED IN FULL
Authority: EO 13526
Chief, Records & Declass Div, WHS
Date: OCT 02 2015

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

PORT 6003

TABLE A-IV
 TENSILE PROPERTIES OF RENE' 41 SHEET AT SELECTED STRAIN RATES

Test Conditions:

Heat Treatment - 1950°F, 1/2-hour, rapid air cool
 Age at 1400°F, 16 hours
 Heating
 - Resistance
 Specimens - 0.050 inch thick sheet, per Dwg. XL4281
 Hold Time - 5 minutes
 Gage Length - 2 Inches

Specimen Number	Strain Rate Start to Rupture (in/in/sec)	Test Temperature (°F)	Proportional Limit (ksi)	0.2% Yield Strength (ksi)	Ultimate Tensile Strength (ksi)	Elongation In 2 Inches (%)	Young's Modulus (x 10 ⁶ psi)
G-2	0.001	1400	78.0	120.0	134.0	2.5	26.3
G-4	0.001	1400	77.0	118.0	134.0	2.5	25.0
G-6	0.001	1400	76.0	121.0	134.0	2.5	25.3
		Average	77.0	119.7	134.0	2.5	25.3
G-7	0.01	1400	80.0	120.0	145.0	4.0	26.6
G-8	0.01	1400	82.0	123.0	148.0	4.0	24.0
G-9	0.01	1400	81.0	123.0	146.0	3.5	22.0
G-28	0.01	1400	78.0	121.0	149.0	4.5	26.7
		Average	80.0	121.8	147.0	4.0	24.8
G-12	0.1	1400	--	--	152.0	8.5	--
G-15	0.1	1400	--	--	160.0	8.5	--
G-5	0.1	1400	--	--	161.0	9.0	--
G-25	0.1	1400	88.0	--	150.0	16.5	24.0
G-33	0.1	1400	98.0	131.5	154.0	7.0	29.0

~~SECRET RESTRICTED DATA~~

~~ATOMIC ENERGY ACT OF 1954~~

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

ASD-TDR-63-277, Vol. IV

REPORT 6003

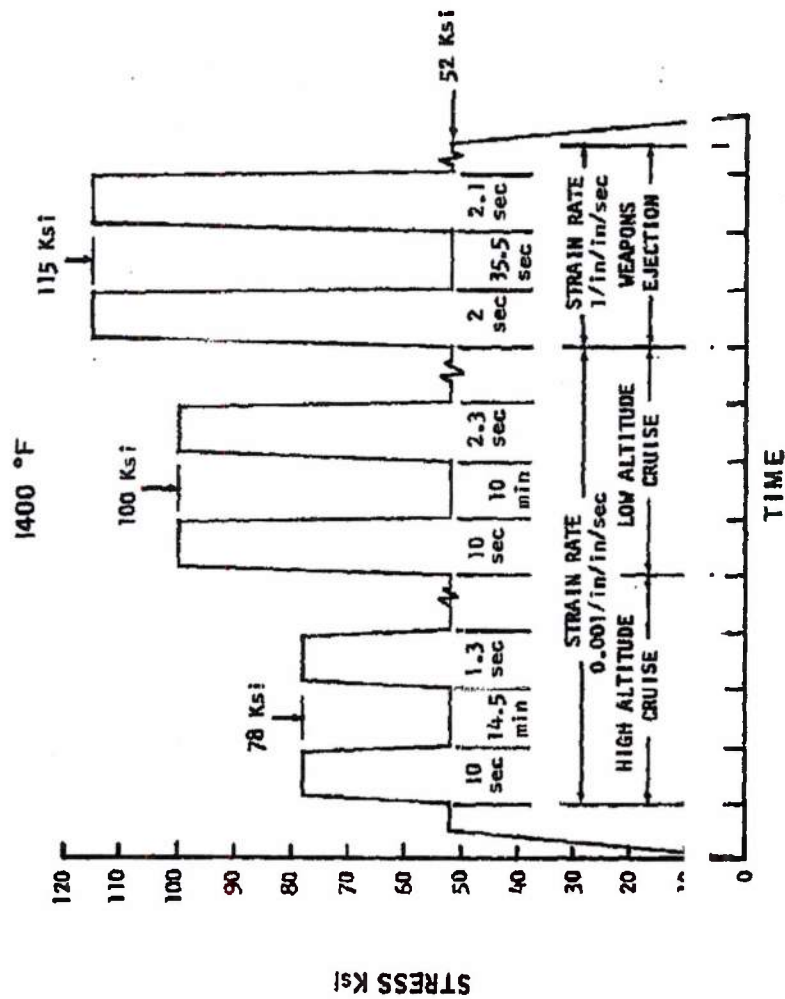


FIGURE A-1. Stress-Time Profile for Rene' 41

~~SECRET RESTRICTED DATA~~
~~ATOMIC ENERGY ACT OF 1954~~

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

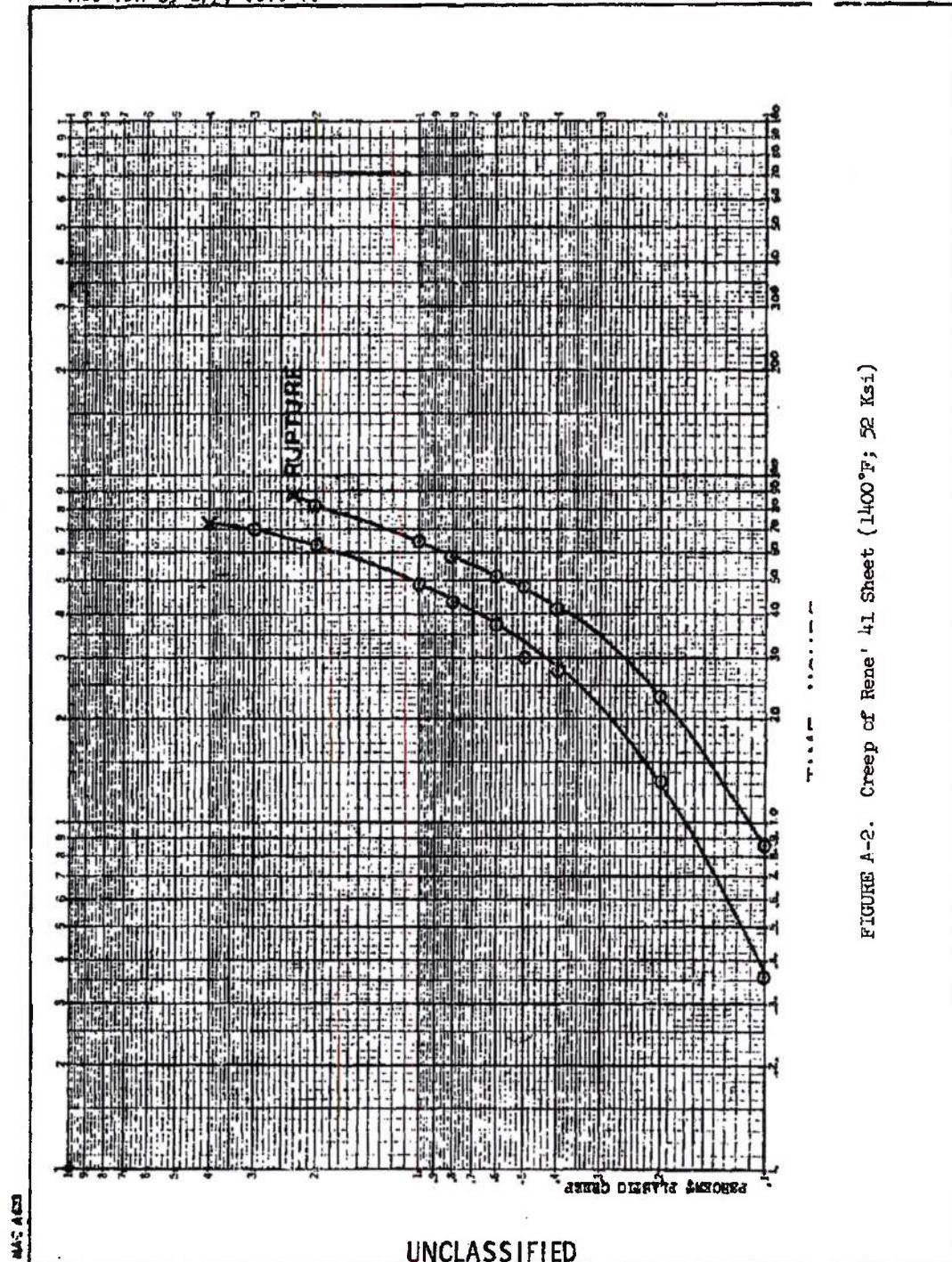


FIGURE A-2. Creep of Rene 41 Sheet (1400°F; 52 Ksi)

UNCLASSIFIED

28B364

- 306 -

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

REPORT 6003

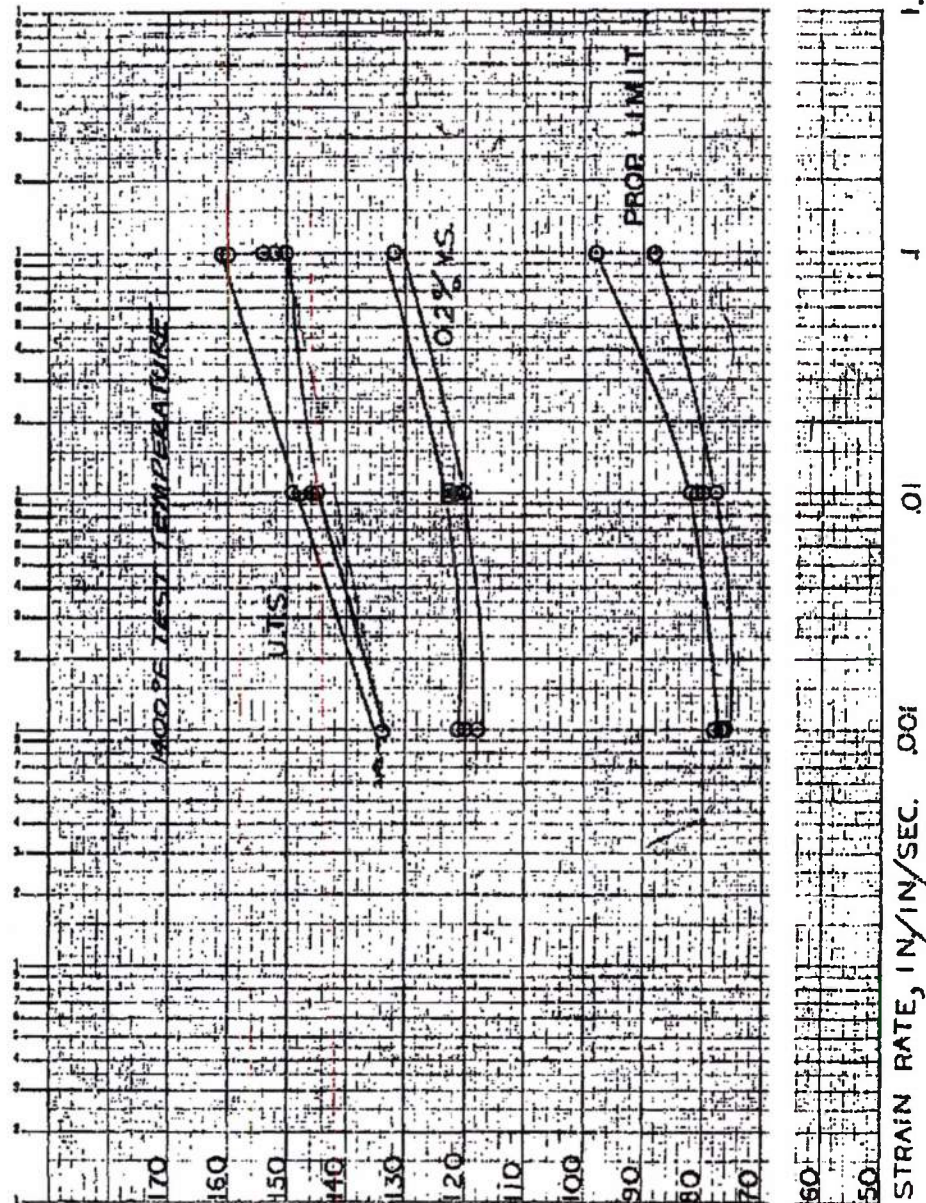


FIGURE A-3. Ultimate Tensile Strength and Yield Strength of Rene' 41 Sheet at Selected Strain Rates

UNCLASSIFIED

28A526

- 307 -

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5
Date: OCT 02 2015

UNCLASSIFIED

ASD-TDR-63-277, Vol. IV

6003

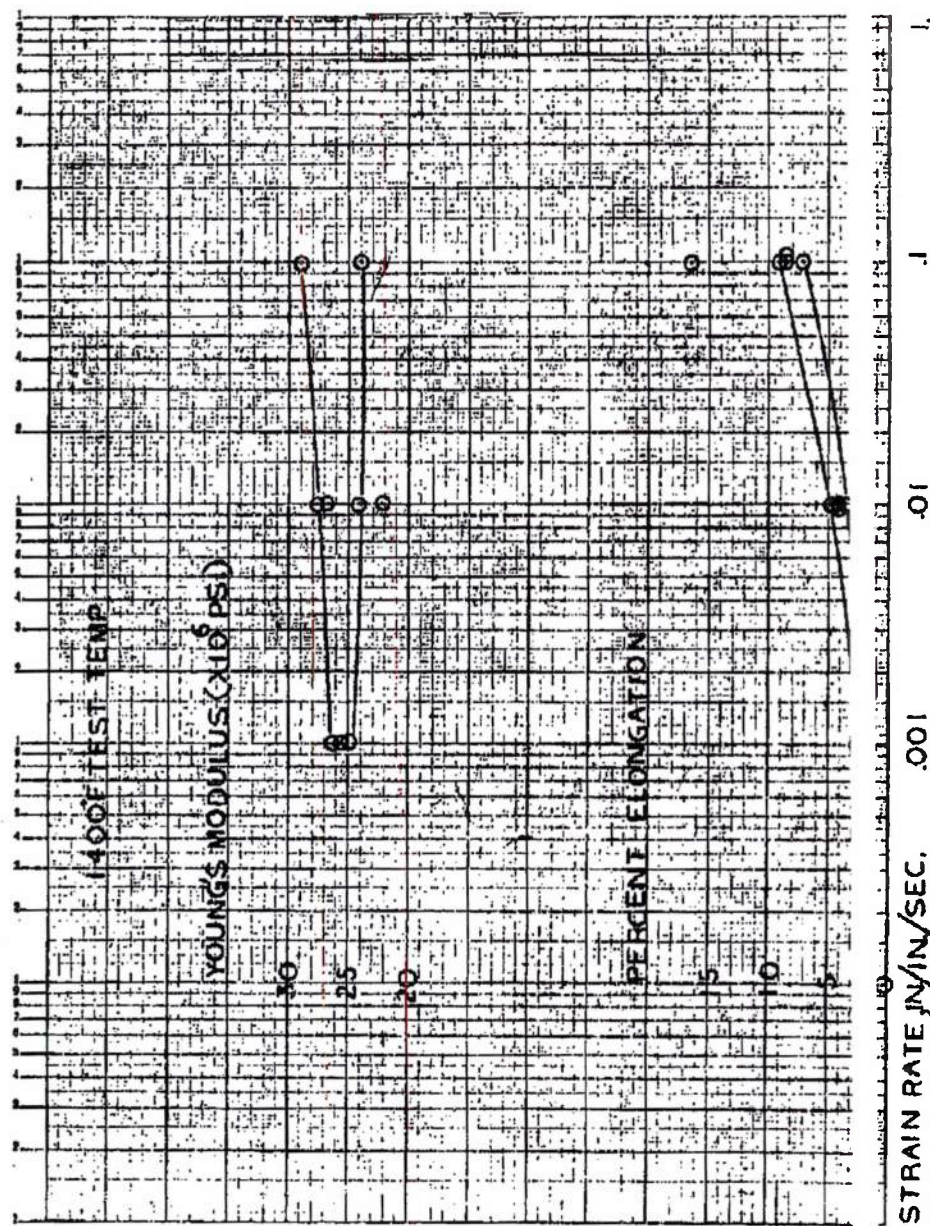


FIGURE A-4. Young's Modulus and Percent Elongation of Rene 41 Sheet at Selected Strain Rates

MAC 4673

UNCLASSIFIED

Page determined to be Unclassified
Reviewed Chief, RDD, WHS
IAW EO 13526, Section 3.5

Date: OCT 02 2015

ASD-TDR-63-277, Vol. IV	UNCLASSIFIED	Report 6003
UNCLASSIFIED 1. Nuclear ramjet propulsion systems 2. Propulsion systems 3. Aircraft nuclear power propulsion 4. Nuclear powerplants 5. Ramjet engines I. AFSC Project 655A, Task 1 and 5 II. Contract AF 33(657)-8123 III. The Marquardt Corp., Van Nuys, Calif. IV. R. D. Grossman V. TMC Report 6003 VI. Not eval fr OPS UNCLASSIFIED	UNCLASSIFIED 1. Nuclear ramjet propulsion systems 2. Propulsion systems 3. Aircraft nuclear power propulsion 4. Nuclear powerplants 5. Ramjet engines I. AFSC Project 655A, Task 1 and 5 II. Contract AF 33(657)-8123 III. The Marquardt Corp., Van Nuys, Calif. IV. R. D. Grossman V. TMC Report 6003 UNCLASSIFIED	UNCLASSIFIED 1. Nuclear ramjet propulsion systems 2. Propulsion systems 3. Aircraft nuclear power propulsion 4. Nuclear powerplants 5. Ramjet engines I. AFSC Project 655A, Task 1 and 5 II. Contract AF 33(657)-8123 III. The Marquardt Corp., Van Nuys, Calif. IV. R. D. Grossman V. TMC Report 6003 UNCLASSIFIED
Aeronautical Systems Division, Dir/Aeromechanics, Propulsion Lab, Wright-Patterson AFB, Ohio Rpt Nr ASD-TDR-63-277, vol. IV. NUCLEAR RAMJET PRODUCTION SYSTEM APPLIED RESEARCH AND TECHNOLOGY (1962 TECHNICAL SUMMARY): VOL. IV. Propulsion System Design and Structural Analysis Summary, Annual Summary Report, 15 Feb 63. P Incl. illus., tables, 25 refs. Volume contains results of design, material studies, and structures component testing of a nuclear propulsion system for the Pluto reactor program. Design concepts, structural analysis of steady state and dynamic loads, material evaluation, and recommended dynamic and structural test programs are studied. Methods of analysis are outlined. Unclassified Abstract	Aeronautical Systems Division, Dir/Aeromechanics, Propulsion Lab, Wright-Patterson AFB, Ohio Rpt Nr ASD-TDR-63-277, vol. IV. NUCLEAR RAMJET PRODUCTION SYSTEM APPLIED RESEARCH AND TECHNOLOGY (1962 TECHNICAL SUMMARY): VOL. IV. Propulsion System Design and Structural Analysis Summary, Annual Summary Report, 15 Feb 63. P Incl. illus., tables, 25 refs. Volume contains results of design, material studies, and structures component testing of a nuclear propulsion system for the Pluto reactor program. Design concepts, structural analysis of steady state and dynamic loads, material evaluation, and recommended dynamic and structural test programs are studied. Unclassified Abstract	Aeronautical Systems Division, Dir/Aeromechanics, Propulsion Lab, Wright-Patterson AFB, Ohio Rpt Nr ASD-TDR-63-277, vol. IV. NUCLEAR RAMJET PRODUCTION SYSTEM APPLIED RESEARCH AND TECHNOLOGY (1962 TECHNICAL SUMMARY): VOL. IV. Propulsion System Design and Structural Analysis Summary, Annual Summary Report, 15 Feb 63. P Incl. illus., tables, 25 refs. Volume contains results of design, material studies, and structures component testing of a nuclear propulsion system for the Pluto reactor program. Design concepts, structural analysis of steady state and dynamic loads, material evaluation, and recommended dynamic and structural test programs are studied. Unclassified Abstract

UNCLASSIFIED